



2807 West County Road 75  
Monticello, MN 55362

August 27, 2024

L-MT-24-024  
10 CFR 50.73

ATTN: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

Monticello Nuclear Generating Plant  
Docket No. 50-263  
Renewed Facility Operating License No. DPR-22

Monticello Nuclear Generating Plant Licensee Event Report 2024-002-00

Northern States Power Company, a Minnesota corporation, doing business as Xcel Energy (hereafter "NSPM"), hereby submits Licensee Event Report (LER) 50-263/2024-002-00 per 10 CFR 50.73(a)(2)(v)(B) and 10 CFR 50.73(a)(2)(v)(D).

If you have any questions about this submittal, please contact Carrie Seipp, Senior Regulatory Engineer, at 612-330-5576.

Summary of Commitments

This letter makes no new commitments and no revisions to existing commitments.

A handwritten signature in black ink, appearing to read 'Greg D Brown', written over a horizontal line.

Greg D Brown  
Plant Manager, Monticello Nuclear Generating Plant  
Northern States Power Company – Minnesota

Enclosure

cc: Administrator, Region III, USNRC  
Project Manager, Monticello, USNRC  
Resident Inspector, Monticello, USNRC  
State of Minnesota

**ENCLOSURE**

**MONTICELLO NUCLEAR GENERATING PLANT  
LICENSEE EVENT REPORT 50-263/2024-002-00**

3 pages follow



# LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)

(See NUREG-1022, R.3 for instruction and guidance for completing this form  
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to [Infocollections.Resource@nrc.gov](mailto:Infocollections.Resource@nrc.gov), and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503. The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.

<b>1. Facility Name</b> Monticello Nuclear Generating Plant	<input checked="" type="checkbox"/> <b>050</b> <input type="checkbox"/> <b>052</b>	<b>2. Docket Number</b> 263	<b>3. Page</b> 1 OF 3
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**4. Title**  
 Low Pressure Coolant Injection Inoperable Due To Motor Valve Failure

5. Event Date			6. LER Number			7. Report Date			8. Other Facilities Involved	
Month	Day	Year	Year	Sequential Number	Revision No.	Month	Day	Year	Facility Name	Docket Number
06	28	24	2024	002	00	08	27	24	<input type="checkbox"/> <b>050</b> <input type="checkbox"/> <b>052</b>	Docket Number Docket Number

<b>9. Operating Mode</b> 1	<b>10. Power Level</b> 100
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**11. This Report is Submitted Pursuant to the Requirements of 10 CFR §: (Check all that apply)**

<b>10 CFR Part 20</b>	<input type="checkbox"/> 20.2203(a)(2)(vi)	<b>10 CFR Part 50</b>	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	<input type="checkbox"/> 73.1200(a)
<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	<input type="checkbox"/> 73.1200(b)
<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)	<input type="checkbox"/> 73.1200(c)
<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)	<input type="checkbox"/> 73.1200(d)
<input type="checkbox"/> 20.2203(a)(2)(i)	<b>10 CFR Part 21</b>	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<b>10 CFR Part 73</b>	<input type="checkbox"/> 73.1200(e)
<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 21.2(c)	<input type="checkbox"/> 50.69(g)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.77(a)(1)	<input type="checkbox"/> 73.1200(f)
<input type="checkbox"/> 20.2203(a)(2)(iii)		<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> 73.77(a)(2)(i)	<input type="checkbox"/> 73.1200(g)
<input type="checkbox"/> 20.2203(a)(2)(iv)		<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> 73.77(a)(2)(ii)	<input type="checkbox"/> 73.1200(h)
<input type="checkbox"/> 20.2203(a)(2)(v)		<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)		

**OTHER** (Specify here, in abstract, or NRC 366A).

**12. Licensee Contact for this LER**

<b>Licensee Contact</b> Carrie Seipp, Senior Nuclear Regulatory Engineer	<b>Phone Number (Include area code)</b> 612-330-5576
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**13. Complete One Line for each Component Failure Described in this Report**

Cause	System	Component	Manufacturer	Reportable to IRIS	Cause	System	Component	Manufacturer	Reportable to IRIS
B	BO	ISV	L200	Y					

**14. Supplemental Report Expected**

No  Yes (If yes, complete 15. Expected Submission Date)

**15. Expected Submission Date**

Month	Day	Year

**16. Abstract** (Limit to 1326 spaces, i.e., approximately 13 single-spaced typewritten lines)

On June 28, 2024 at 0110 CDT Monticello Nuclear Generating Plant was in Mode 1 at 100 percent power and performing OSP-RHR-0556 "Residual Heat Removal (RHR) Water Fill Verification" Technical Specification Surveillance Procedure. When the RHR Low Pressure Coolant Injection (LPCI) Motor Valve MO-2012 "RHR Division 1 LPCI Injection Outboard Valve" was attempted to be cycled, the Motor Valve opened approximately one inch then stopped. With the Motor Valve incapable of opening, the A LPCI injection path was inoperable for injection as an Emergency Core Cooling system; and due to the plant design of the LPCI Loop Select Logic, this rendered both subsystems of LPCI inoperable.

The cause of the event was that the Motor Valve's motor pinion was not secured to the motor shaft because an incorrect-sized set screw was installed. The shorter motor pinion set screw relaxed, allowing the motor pinion to move axially along the motor shaft until it no longer engaged the worm shaft clutch gear.

On June 29, 2024, the motor pinion was adjusted back into position on the motor shaft. On August 7, 2024, the incorrect motor pinion set screw was replaced with the correct set screw.



**LICENSEE EVENT REPORT (LER)  
CONTINUATION SHEET**

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1. FACILITY NAME  Monticello Nuclear Generating Plant	<input checked="" type="checkbox"/> 050	2. DOCKET NUMBER  263	3. LER NUMBER		
	<input type="checkbox"/> 052		YEAR 2024	SEQUENTIAL NUMBER 002	REV NO. 0

**NARRATIVE**

**EVENT DESCRIPTION**

On June 28, 2024 at 0110 CDT Monticello Nuclear Generating Plant (MNGP) was in Mode 1 at 100 percent power and performing OSP-RHR-0556 "Residual Heat Removal (RHR) Water Fill Verification" Technical Specification Surveillance Procedure. When the RHR Low Pressure Coolant Injection (LPCI) [EIS CODE: BO] Motor Valve MO-2012 "RHR Division 1 LPCI Injection Outboard Valve" was attempted to be opened via the control room hand switch, the Motor Valve opened approximately one inch then stopped. With the Motor Valve incapable of opening, the A LPCI injection path was inoperable for injection as an Emergency Core Cooling system; and due to the plant design of the LPCI Loop Select Logic, this rendered both subsystems of LPCI inoperable. Specifically, if the B Recirc loop was determined to be broken, the automatic logic would be incapable of opening the path to the A LPCI injection path and neither division would automatically inject.

This event is reportable per 10 CFR 50.73(a)(2)(v)(B) and 10 CFR 50.73(a)(2)(v)(D) for an event or condition that could have prevented the fulfillment of a safety function of structures or systems that are needed to remove residual heat and mitigate the consequences of an accident based on the safety function at the RHR LPCI system level. This issue was evaluated for a Safety System Functional Failure (SSFF) in accordance with NEI 99-02. This event is a Safety System Functional Failure (SSFF) due to meeting reporting criteria per 10 CFR 50.73(a)(2)(v)(B) and 10 CFR 50.73(a)(2)(v)(D).

**EVENT ANALYSIS**

On June 27, 2024 at 2158 CDT, Technical Specification 3.5.1 "ECCS – Operation" Condition D "Two LPCI subsystems inoperable for reasons other than Condition C or G" was entered for performance of Technical Specification Surveillance Procedure 0255-04-IA-1-2 "RHR Loop B Quarterly Pump and Valve Tests." Prior to exiting Condition D of Technical Specification 3.5.1, OSP-RHR-0556 was performed. While the Motor Valve was attempted to be opened via the control room hand switch, the Motor Valve opened approximately one inch then stopped, resulting in the inability to exit Condition D of Technical Specification 3.5.1. The plant had no other Structures, Systems, or Components that were inoperable at the start of the event that contributed to the event.

While this was later determined to be incorrect, initial troubleshooting identified that the most likely cause of the Motor Valve failure to stroke was the clutch mechanism was disengaged partially from the motor pinion as a result of foreign material from failure of the tripper spring. The Motor Valve was repaired and satisfactorily retested, then Condition D of Technical Specification 3.5.1 was exited within the required completion time on June 29, 2024 at 1533 CDT.

During performance of the root cause evaluation interviews, it was identified that during the Motor Valve repairs performed on June 29, 2024, the motor pinion was found extended on the motor shaft and prior to reassembly, it was adjusted back into position. The vendor technical manual specified that the motor pinion should have a set screw and lock wire installed, so it should not have been able to move.

Further physical investigation on the Motor Valve and specifically the motor pinion was performed on August 7, 2024. The Motor Valve was successfully stroked open and closed, then the motor pinion was found to be held in place with a set screw and lock wire, with no movement along the motor shaft. The set screw was removed for inspection, and it was determined to be the incorrect set screw. Additionally, indications on the motor shaft revealed that motor pinion had potentially moved axially in the past. A review of past maintenance records did not identify a time where the set screw had been replaced.

An incorrect set screw would have allowed the motor pinion to travel axially along the motor shaft and eventually disengage with the worm shaft clutch gear. When the motor pinion disengaged with the worm shaft clutch gear, the Motor Valve could not be operated via the control room hand switch which starts the motor actuator.

(04-02-2024)



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**NARRATIVE**

**ASSESSMENT OF SAFETY CONSEQUENCES**

Actual safety consequences of the event were minimal for both normal operation and design basis event operation based on the operability during this time of all High Pressure Emergency Core Cooling System (ECCS), as well as both loops of the Low Pressure ECCS subsystem of Core Spray. The remaining functions of RHR including Drywell Spray and Suppression Pool Cooling remained operable despite the loss of the LPCI subsystem ability to inject into A LPCI Injection Path. Based on the operability of these additional sources of coolant to the core, the consequences of this event were minimal. There was no impact on the health and safety of the public or plant personal. There were no radiological or industrial impacts associated with this event. The health and safety of the public and site personnel were not impacted during this event.

**CAUSE OF THE EVENT**

The Motor Valve failed to operate because the motor pinion was not secured to the motor shaft because an incorrect-sized set screw was installed. The shorter motor pinion set screw relaxed, allowing the motor pinion to move axially along the motor shaft until it no longer engaged the worm shaft clutch gear.

**CORRECTIVE ACTIONS**

On June 29, 2024, the Motor Valve's motor pinion was adjusted back into position on the motor shaft. On August 7, 2024, the incorrect motor pinion set screw on the Motor Valve was replaced with the correct set screw.

**PREVIOUS SIMILAR EVENTS**

There were no previous similar events in the past three years.