

South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

June 19, 2024 NOC-AE-24004048 10 CFR 50.73 STI: 35610159 File No. G26

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ATTN: Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

South Texas Project Unit 1 Docket No. STN 50-498 Licensee Event Report 2023-003-01 Supplement to Two Essential Chilled Water Trains Inoperable Resulting in a Condition That <u>Could Have Prevented Fulfillment of a Safety Function</u>

Reference: Letter; J. Tomlinson (STP) to Document Control Desk (NRC); "Licensee Event Report 2023-003-00 Two Essential Chilled Water Trains Inoperable Resulting in a Condition That Could Have Prevented Fulfillment of a Safety Function;" January 09, 2024; (NOC-AE-24004007)(ML24009A282)

Pursuant to reporting requirements of 10 CFR 50.73(a)(2)(v)(D), STP Nuclear Operating Company (STPNOC) hereby submits the supplemented South Texas Project (STP) Unit 1 Licensee Event Report 2023-003-01.

The event did not have an adverse effect on the health and safety of the public.

There are no commitments in this letter.

If there are any questions on this submittal, please contact Derrick Johnson at 361-972-7088 or me at 361-972-8945.

GATO

Jason Tomlinson Site Vice President

Attachment:

Licensee Event Report 2023-003-01, Two Essential Chilled Water Trains Inoperable Resulting in a Condition That Could Have Prevented Fulfillment of a Safety Function

NOC-AE-24004048 Page 2 of 2

CC:

Regional Administrator, Region IV U.S. Nuclear Regulatory Commission 1600 E. Lamar Boulevard Arlington, TX 76011-4511

NOC-AE-24004048 Attachment

Attachment

Licensee Event Report 2023-003-01

Two Essential Chilled Water Trains Inoperable Resulting in a Condition That Could Have Prevented Fulfillment of a Safety Function

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (04-02-2024)						ON APPRO	APPROVED BY OMB: NO. 3150-0104 EXPIRES: 04/30/2027								
(See NUREG-1022, R.3 for instruction and guidance for completing this form http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/)						learned ar estimate Commissie at: OMB Commissie not require	Estimated burden per response to comply with this mandatory collection request. 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects.Resource@nrc.gov, and the OMB reviewer at OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503. The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.								
1. Facility Name South Texas Ur	nit 1							050	0 2. Docket Number 3. Page						
South Texas Of								052		00498		1	OF	7	
4. Title Two Essential (Function	wo Essential Chilled Water Trains Inoperable Resulting in a Condition That Could Have Prevented Fulfillment of a Safety														
5. Event Da	te		6. LER Number		7.	Report I	Date		8	. Other Fac	ilities Involv	ved			
Month Day	Year	Year	Sequential Number	Revision No.	Month	Day	Year	Facility Nar	ne		050	Docket N	Number		
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10 CFR Part	11. This Report is Submitted Pursuant to the Requirements of 10 CFR §: (Check all that apply) 10 CFR Part 20 20.2203(a)(2)(vi) 10 CFR Part 50 50.73(a)(2)(ii)(A) 50.73(a)(2)(viii)(A) 73.1200(a)									200(2)					
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20.2201(d)	20.2201(b) 20.2203(a)(3)(i) 50.36(c)(1)(i)(A)								50.73(a)(2)(ix)(A) 73.1200(c)						
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20.2203(a)(2	2)(v)			50).73(a)(2)(i)	(C)		3(a)(2)(vii)	L						
OTHER (Spe	ecify here,	in abstract	t, or NRC 366A).											
				12	2. License	e Conta	ct for this L	.ER							
Licensee Contact Derrick Johnso	n / Licen	ising Sp	ecialist			-					Phone Num 36		clude a 2-708		
			13. Complete	One Line	for each C	ompon	ent Failure	Describe	d in this R	eport			*******		
Cause	System	Compor	nent Manufac	turer Rep	ortable to IF	RIS	Cause	Syst	tem C	Component Manu		acturer Repor		rtable to IRIS	
В	KM	SEA	L	Í.	Y		Х	KI	M	TCV	B066	5		Y	
	14.	Suppleme	ental Report Exp	ected		-	15	15. Expected Submission Date Month Day					ay	Year	
V No	Ye	es (If yes, c	complete 15. E	xpected S	ubmission	Date)	15.	Expected	Submission	Date					
On 11/10/23 at water temperat declared inope out of the three required by 10 compressor sh inoperability ca check and the	✓ No Yes (If yes, complete 15. Expected Submission Date) 16. Abstract (Limit to 1326 spaces, i.e., approximately 13 single-spaced typewritten lines) On 11/10/23 at 0642 Essential Chilled Water 'B' train and cascading equipment was declared inoperable due to chill water temperature exceeding limits. On 11/10/23 at 1413 Essential Chilled Water 'C' train and cascading equipment was declared inoperable due to discharge pressure exceeding limits. This condition resulted in an inoperable condition on two out of the three safety trains for the accident mitigating function. This condition was determined to be reportable as required by 10 CFR 50.73(a)(2)(v)(D). Essential Chilled Water 'C' train inoperability was due to the Essential Chilled Water 'B' train inoperability cause was indeterminate. Corrective actions include a Performance Assessment to evaluate proper flatness check and the proper installation of the compressor main shaft mechanical seal, evaluate materials needed to clean shaft seal parts during reassembly, and revise chiller maintenance procedure to include cleaning guidance material.														

NRC FORM 366B (04-02-2024) LICENSEE EVENT REPORT (LER) (FAILURE CONTINUATION) (See NUREG-1022, R.3 for instruction and guidance for completing this form http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/)					Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17 th Street NW,							
1. FACILITY NAME				2. DOCKET	NUMBER	YEAR	3. LER NUMBER SEQUENTIAL REV					
South Texas Unit 1			052	00498			NUMBER 003	NO. - 01				
-	13. COMP	LETE ONE LINE FOR E	ACH COMPON	ENT FAILU	JRE DESC	RIBED IN TH	IIS REPORT					
CAUSE		SYSTEM	COMPONI	ENT	MANU	ACTURER	REPORT	ABLE TO IRIS				
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NRC FORM 366B (04-02-2024)

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NRC FORM 366A U.S. NUCLEAR REGULATOR	MISSION								
LICENSEE EVENT REPOR	ER)	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects. Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory							
CONTINUATION SHE			Affairs, (3150-0104), Attn: Desk Officer	for the Nucle	ar Regu	latory Commission,	725 17th	Street NW,	
(See NUREG-1022, R.3 for instruction and guidance for cor http://www.nrc.gov/reading-rm/doc-collections/nuregs/st			Washington, DC 20503. The NRC may not conduct or sponsor, and a person is not required respond to, a collection of information unless the document requesting or requiring the collect displays a currently valid OMB control number.						
1. FACILITY NAME		050	2. DOCKET NUMBER 3. LER NUMBER YEAR SEQUENTIAL REV						
outh Texas Unit 1 052 00498 2023 - 003 -								REV NO.	
			l]		<u> </u>		ļļ. I		
I. Description of Event									
A. Reportable Event Classification									
This event is reportable per 50.73(a)(2)(v)(D) as a condition that could have prevented fulfillment of a safety function of structures or systems that are needed to: (D) Mitigate the consequences of an accident. This event resulted in an inoperable condition on two out of the three safety trains for the accident mitigating function including 'B' and 'C' train High Head Safety Injection, Low Head Safety Injection, Containment Spray, Control Room Envelope HVAC, and Essential Chilled Water.									
B. Plant Operating Conditions Prior To Event									
Prior to the event on November 10, 2023, Unit 1	was ir	MODE	1 at 100% power.						
C. Status Of Structures, Systems, and Components That Were Inoperable At the Start of the Event and That Contributed To the Event									
There were no inoperable Structures, Systems,	or Con	nponents	s at the start of the event	that co	ntribu	uted to the	event.		
D. Narrative Summary of the Event Timeline (Note: All times Are Central Standard	Time)								
11/10/23 (0642) - Essential Chilled Water 'B' tra temperature exceeding limits.	ain and	l cascadi	ing equipment were decl	ared inc	pera	ble due to o	chill w	ater	
11/10/23 (1413) - Essential Chilled Water 'C' tra discharge pressure.	ain decl	ared ino	perable due to exceedin	g the ma	aximı	um compre	ssor		
11/10/23 (2127) - Shift Manager determined that two inoperable Essential Chilled Water Trains resulted in a condition that could have prevented the fulfillment of a safety function.									
11/10/23 (2152) - Event Notification # 56848 wa	as mad	e to the	NRC Headquarters Ope	rations (Office	er (HOO).			
11 /11 /23 (1330) - Essential Chilled Water 'B' to for the accident mitigation function.	rain de	clared O	PERABLE which also re	stored t	he sa	afety trains	relied	on	
E. Method of Discovery									
The Chill Water exceeding temperature limits in Essential Chilled Water Train 'B' and the discharge pressure exceeding limits in Essential Chilled Water Train 'C ' were self-revealing .									
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NRC FORM 366A U.S. NUCLEAR REGULATOR	RY COMMISSION	N APPROVED BY OMB: NO. 3150-0104 EXPIRES: 04/30/2027								
(04-02-2024) LICENSEE EVENT REPOR CONTINUATION SHE (See NUREG-1022, R.3 for instruction and guidance for con http://www.nrc.gov/reading-rm/doc-collections/nuregs/sta	Estimated burden per response to complessons learned are incorporated into the regarding burden estimate to the FOIA, Nuclear Regulatory Commission, Infocollects.Resource@nrc.gov, and the Affairs, (3150-0104), Attn: Desk Officer Washington, DC 20503. The NRC no respond to, a collection of information displays a currently valid OMB control no section.	ne licensing proc Library, and Inf Washington, e OMB reviewer for the Nuclear may not conduc n unless the do	cess and fed ba formation Colled DC 20555 at: OMB Offic r Regulatory Co t or sponsor, a	ack to indust ctions Brand 5-0001, or e of Informa ommission, 7 and a perso	try. Sen h (T-6 / by tion and 725 17th n is no	d comments A10M), U. S. email to d Regulatory h Street NW, t required to				
1. FACILITY NAME		2. DOCKET NUMBER		3. LER	NUMBER					
	■ 050		YEAR	SEQUE NUM			REV NO.			
South Texas Unit 1							01			
NARRATIVE										
 II. Component Failures A. Failure Mode, Mechanism, and Effects of Failed Component On 11/10/23 at 1448, Essential Chiller 12C compressor discharge pressure was reading higher than allowable due to the Essential Chiller's Compressor Shaft Mechanical Seal becoming degraded to the point of losing seal integrity. The Motor Shaft Compressor Mechanical Seal was a significant source of air in-leakage due to being in a vacuum condition whether the chiller is running or idle. The degradation to failure created a gap which allowed air in-leakage to occur and consequently raised compressor discharge pressure above the allowable limit. 										
The Essential Chiller 12B temperature control loop controls the Leaving Water Temperature. On 11/10/23 at 0045, at least one component in the Essential Chiller 12B temperature control loop failed to operate properly which resulted in the inability to maintain proper refrigerant pressure and flow to maintain the chilled water discharge/outlet temperature at the pre-established setting.										
B. Cause of Component or System Failure										
Essential Chilled Water Train 'C' The source of the air in-leakage that resulted in the high compressor discharge pressure was determined to be the compressor main shaft seal. After it was replaced, compressor discharge pressure was observed to be normal during post-maintenance testing. The shaft seal ring had recently been replaced on 7/19/2023, under Preventative Maintenance Work Order 659148. The PM activity is on a five-year replacement schedule. The shaft seal had been in service for four months when it was identified as the path of air in-leakage. Per PM Work Order 659148, the new seal face passed a required optical flatness check prior to installation. The shaft seal ring was replaced on 11/10/2023. The Maintenance technician that removed the seal under the Corrective Maintenance Work Order reported the as-found conditions during seal removal as no apparent evidence of installation issues or noticeable seal defects.										
Post-event initial inspections of the removed shaft seal: -The removed shaft seal ring did not pass the optical flatness when it was checked post-event several days later. -An Engineering inspection revealed a gouge on the spring seating location upon inspection by Engineering. It is indeterminate when the gouge occurred. -An Engineering inspection also revealed wear cracks and chipping on the leading edge of the shaft seal ring upon inspection by Engineering. -Enhanced visual inspection by a Materials Engineer found no indications of material defects. -The Pump Engineer reviewed the vibration data on Chiller 12C from July 22nd and November 19th and found no trends of concern. -The chiller Engineer identified purge counts increasing following July 19, 2023. The compressor shaft seal was sent to an off-site lab for failure analysis. The lab analyst report had concluded that the carbon stationary ring and iron rotating ring both exhibited circumferential scoring and wear on the sealing surface caused by silica contamination. The advanced wear and scoring on the sealing surfaces lead to elevated seal leakage. Essential Chilled Water Train 'B' Neither the Hot Gas Bypass Motor Operated Valve Motor nor the Temperature Current Module were bench tested; therefore, cause determination is indeterminate.										
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NRC FORM 366A U.S. NUCLEAR REGULATOF		APPROVED BY OMB: NO.	. 3150-010	4 EXPIRES	: 04/30/2027			
(04-02-2024) LICENSEE EVENT REPOR CONTINUATION SHE (See NUREG-1022, R.3 for instruction and guidance for com http://www.nrc.gov/reading-rm/doc-collections/nuregs/sta	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commission, Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commission, Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commission, Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commission, Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Nuclear Regulatory Commendation and Regulatory Commission, 725 17th Street NW, Street							
1. FACILITY NAME	050	2. DOCKET NUMBER 3. LER NUMBER						
South Texas Unit 1		00498	2023	NUMBER	NO.			
	052			- 003				
C. Systems or Secondary Functions That Were	Affected by the	Enilure of Components	with Mul	Itiple Eurotions				
Failure of the components in Essential Chilled W systems inoperable: High Head Safety Injection, HVAC, and Essential Chill Water.	later 'B' and 'C	' trains rendered the 'B' a	and 'C' tr	ains of the follow				
D. Failed Component Information System: Chilled Water System {KM} Component: Compressor Shaft Seal {SEAL} Manufacturer: Pure Carbon Company Model:								
System: Chilled Water System {KM} Component: Temperature Current Limit Control {} Manufacture: Borg-Warner {B350} Model: UCI-TCM-1A								
System: Chilled Water System {KM} Component: Hot Gas Bypass Valve Motor Operated Valve{MO} Manufacture: Barber-Colman{B066} Model: MP-487-119-0-1								
III. Analysis of Event								
A. Safety System Responses that Occurred								
No safety system responses occurred because	of this event.							
B. Duration of Safety System Inoperability								
Essential Chilled Water Trains 'B' and 'C' were I 2023, when Essential Chilled Water Train 'B' wa								
C. Safety Consequences and Implications								
The accumulated incremental risk during the time period with two trains inoperable for Incremental Core Damage Probability (ICDP) and Incremental Large Early Release Probability (ILERP) are within normal work controls, but compensatory Risk Management Actions were implemented per the requirements of the STP Configuration Risk Management Program to offset the risk impacts of the configuration.								
The event did not result in any offsite release of personnel injuries or damage to any other safet					no			
Therefore, there was no adverse effect on the h	nealth and safe	ety of the public.						

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050	2. DOCKET NUMBER	L				APPROVED BY OMB: NO. 3150-0104 EXPIRES: 04/30/2027 Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects. Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503. The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.					
052		YEAR	T								
						ds					
ample for silica in t	ne essential chiller oil. Sy	/stem co	onta	amination is co	onside	ered					
2. Handling on-site: Maintenance Personnel are instructed via the work instructions on how to handle the seals with latex gloves and avoid touching the seal face. Chemistry test the latex gloves for silica and has consistently had a negative reading. The glass used for the light band test is also cleaned prior to every use to ensure contaminants in the air are not being placed between the glass and the seal face.											
3. Shipping: These boxes are not shipped with a silica based desiccant which would rule that out as a potential source. The seals are 100% receipt inspected by QC and again by the Chiller Technicians in the Maintenance Shop. The seal is carried to the job site inside its box and still wrapped in its original packaging, which could rule out environmental factors while traversing from the shop to the job site.											
ufacturing process	es were a factor.										
5. There was no need to perform any grinding, sanding, or lapping of the shaft area during replacement. Part of the shaft for installation of the seal green "Scotch Brite #96" is used to clean the area. The Safety Data Sheet for Scotch Brite, lists silica quartz in its make-up. Chemistry performed validation testing, while the concentration of silica was low, it was present. While Maintenance Personnel are extremely diligent with cleaning the area after the use of Scotch Brite to include cleaning the shaft area with alcohol and using chiller oil in a spray bottle to clean the cavity, this is the most probable source of contamination. A condition report action was written to evaluate the need for using an abrasive in area preparation. If it is determined that this is a necessary step, then Maintenance will look to use alternatives to Scotch Brite to mitigate the introduction of any contaminants that could result in premature failure of a seal.											
n inoperability was	indeterminate.										
1											
	ample for silica in the report ample for silica in the are instructed via the mistry test the lates st is also cleaned point of the silica based do and again by the wrapped in its origination of the silica based to cleaned and again by the settermely diligent water and using chiller of the settermely diligent water and using chiller of the setter streamely step, the ants that could results	ample for silica in the essential chiller oil. Sy are instructed via the work instructions on h emistry test the latex gloves for silica and h st is also cleaned prior to every use to ensu- l face. If a silica based desiccant which would rul C and again by the Chiller Technicians in the wrapped in its original packaging, which con- standing, or lapping of the shaft area dur e #96" is used to clean the area. The Safet rmed validation testing, while the concentrate extremely diligent with cleaning the area aft and using chiller oil in a spray bottle to clean on report action was written to evaluate the necessary step, then Maintenance will look	ample for silica in the essential chiller oil. System co are instructed via the work instructions on how to ha emistry test the latex gloves for silica and has consist is also cleaned prior to every use to ensure conta l face. ith a silica based desiccant which would rule that of C and again by the Chiller Technicians in the Mainte wrapped in its original packaging, which could rule that of a subject of the shaft area during replies wrapped in its original packaging, which could rule the strend validation testing, while the concentration of s extremely diligent with cleaning the area after the us and using chiller oil in a spray bottle to clean the cas on report action was written to evaluate the need for necessary step, then Maintenance will look to use a ants that could result in premature failure of a seal.	rred and the report stated Manufacturing, Handling on- ample for silica in the essential chiller oil. System conta are instructed via the work instructions on how to hand emistry test the latex gloves for silica and has consiste st is also cleaned prior to every use to ensure contami l face. ith a silica based desiccant which would rule that out a C and again by the Chiller Technicians in the Maintena wrapped in its original packaging, which could rule out any again by the Chiller Technicians in the Maintena wrapped in its original packaging, which could rule out any again by the chiller Technicians in the Maintena wrapped in its original packaging, which could rule out any again by the clean the area. The Safety Data She rmed validation testing, while the concentration of silic extremely diligent with cleaning the area after the use of and using chiller oil in a spray bottle to clean the cavity on report action was written to evaluate the need for us necessary step, then Maintenance will look to use alter ants that could result in premature failure of a seal.	rred and the report stated Manufacturing, Handling on-site, or from the ample for silica in the essential chiller oil. System contamination is consistently the latex gloves for silica and has consistently had a neg st is also cleaned prior to every use to ensure contaminants in the ail face. ith a silica based desiccant which would rule that out as a potential so and again by the Chiller Technicians in the Maintenance Shop. The wrapped in its original packaging, which could rule out environmenta and a state of the shaft area during replacement. Part of the #96" is used to clean the area. The Safety Data Sheet for Scotch Erred validation testing, while the concentration of silica was low, it we extremely diligent with cleaning the area after the use of Scotch Brite and using chiller oil in a spray bottle to clean the cavity, this is the mon report action was written to evaluate the need for using an abrasin becessary step, then Maintenance will look to use alternatives to Scot ants that could result in premature failure of a seal.	emistry test the latex gloves for silica and has consistently had a negative st is also cleaned prior to every use to ensure contaminants in the air are I face. ith a silica based desiccant which would rule that out as a potential source C and again by the Chiller Technicians in the Maintenance Shop. The seal wrapped in its original packaging, which could rule out environmental factor. Indiacturing processes were a factor. Indiacturing processes were a factor. In the seal of the shaft area during replacement. Part of the sh e #96" is used to clean the area. The Safety Data Sheet for Scotch Brite, I rmed validation testing, while the concentration of silica was low, it was extremely diligent with cleaning the area after the use of Scotch Brite to and using chiller oil in a spray bottle to clean the cavity, this is the most on report action was written to evaluate the need for using an abrasive in necessary step, then Maintenance will look to use alternatives to Scotch E ants that could result in premature failure of a seal.					

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NRC FORM 366A U.S. NUCLEAR REGULATOR	RY COMMISSION									
(See NUREG-1022, R.3 for instruction and guidance for con http://www.nrc.gov/reading-rm/doc-collections/nuregs/sta	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503. The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.									
1. FACILITY NAME		2. DOCKET NUMBER			3. LER NUMBER	ER NUMBER				
	■ 050		YEAR		SEQUENTIAL NUMBER		REV NO.			
South Texas Unit 1										
NARRATIVE						•				
V. Corrective Actions										
Essential Chilled Water Train 'C'										
CR 23-10352-2 - On 11/20/23, chiller technicians Pump Mechanical Seal. Post Maintenance testin			anical Se	eal	and the Lube	Oil				
CR 23-10352-14 - Performance Assessment on shaft mechanical Seal.	proper flatness	s check and proper instal	lation of	the	e compressor	mai	n			
CR 23-10352-13 - The compressor shaft seal wa	as sent to an o [.]	ff-site vendor for failure a	nalysis.							
CR 23-10352-17 - Add monitoring points with ale	ert and alarm s	etpoints for MOC.								
CR 23-10352-19 - Evaluate materials needed to	clean shaft se	al parts during reassemb	ly.							
CR 23-10352-20 - Revise Chiller Maintenance p	rocedure to inc	clude cleaning material g	uidance.							
Essential Chilled Water Train 'B'										
CR 23-10345-1 - On 11/11/13, Chiller technician	is replaced the	Hot Gas Bypass Valve N	IOV.							
CR 23-6338-10 - On 11 /11 /13, Chiller technicia	ins replaced th	e existing Temperature a	nd Curr	ent	Limiter Contr	olle	r.			
VI. Previous Similar Events										
There have been no previous similar events with water exceeding the maximum temperature ope recent issues with essential chillers exceeding th	rability limit sin	nultaneously. However, th	ne site h							