

LG-24-066

June 10, 2024

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Limerick Generating Station, Unit 1
Renewed Facility Operating License No. NPF-39
NRC Docket No. 50-352

Subject: LER 2024-002-00 Invalid Actuation Due to a Trip of 1B Reactor Protection System
Circuit Breaker

In accordance with the requirements of 10 CFR 50.73(a)(2)(iv)(A), Limerick Generating Station hereby submits the enclosed Licensee Event Report.

There are no commitments contained in this letter.

If you have any questions, please contact Jordan Rajan at (610) 718-3400.

Respectfully,



Michael F. Gillin
Vice President – Limerick Generating Station
Constellation Energy Generation, LLC

cc: Administrator Region I, USNRC
USNRC Senior Resident Inspector, Limerick Generating Station



LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)

(See NUREG-1022, R.3 for instruction and guidance for completing this form
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollections.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503. The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.

1. Facility Name Limerick Generating Station Unit 1	<input checked="" type="checkbox"/> 050 <input type="checkbox"/> 052	2. Docket Number 352	3. Page 1 OF 3
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4. Title
 Invalid Actuation Due to a Trip of 1B Reactor Protection System Circuit Breaker

5. Event Date			6. LER Number			7. Report Date			8. Other Facilities Involved	
Month	Day	Year	Year	Sequential Number	Revision No.	Month	Day	Year	Facility Name	Docket Number
04	11	2024	2024	002	00	06	10	2024	<input type="checkbox"/> 050	
									<input type="checkbox"/> 052	

9. Operating Mode 5	10. Power Level 0
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11. This Report is Submitted Pursuant to the Requirements of 10 CFR §: (Check all that apply)

10 CFR Part 20	<input type="checkbox"/> 20.2203(a)(2)(vi)	10 CFR Part 50	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	<input type="checkbox"/> 73.1200(a)
<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	<input type="checkbox"/> 73.1200(b)
<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)	<input type="checkbox"/> 73.1200(c)
<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.36(c)(2)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)	<input type="checkbox"/> 73.1200(d)
<input type="checkbox"/> 20.2203(a)(2)(i)	10 CFR Part 21	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	10 CFR Part 73	<input type="checkbox"/> 73.1200(e)
<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 21.2(c)	<input type="checkbox"/> 50.69(g)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.77(a)(1)	<input type="checkbox"/> 73.1200(f)
<input type="checkbox"/> 20.2203(a)(2)(iii)		<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> 73.77(a)(2)(i)	<input type="checkbox"/> 73.1200(g)
<input type="checkbox"/> 20.2203(a)(2)(iv)		<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> 73.77(a)(2)(ii)	<input type="checkbox"/> 73.1200(h)
<input type="checkbox"/> 20.2203(a)(2)(v)		<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)		

OTHER (Specify here, in abstract, or NRC 366A).

12. Licensee Contact for this LER

Licensee Contact Jordan Rajan	Phone Number (Include area code) 610-718-3400
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13. Complete One Line for each Component Failure Described in this Report

Cause	System	Component	Manufacturer	Reportable to IRIS	Cause	System	Component	Manufacturer	Reportable to IRIS
B	JC	52	W121	Yes					

14. Supplemental Report Expected

<input checked="" type="checkbox"/> No	<input type="checkbox"/> Yes (If yes, complete 15. Expected Submission Date)
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15. Expected Submission Date

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16. Abstract (Limit to 1326 spaces, i.e., approximately 13 single-spaced typewritten lines)

On April 11, 2024, Limerick Generating Station (LGS) Unit 1 was in Operational Condition (OPCON) 5, refueling, performing a surveillance test on the 1B Reactor Protection System (RPS) Power Monitoring System. Following the completion of the testing, when returning the system to the normal plant alignment, an invalid isolation occurred due to a trip of the 1B-Y160 RPS panel.

Troubleshooting on circuit breaker 52-DY24801 found the circuit breaker contacts were not closed even though the handle was in the closed position. The circuit breaker was replaced. This resulted in containment isolation signals that actuated containment isolation valves in more than one system; therefore, the loss of the RPS bus is reportable in accordance with 10 CFR 50.73(a)(2)(iv)(A).



**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

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1. FACILITY NAME Limerick Generation Station Unit 1	<input checked="" type="checkbox"/> 050	2. DOCKET NUMBER 352	3. LER NUMBER		
	<input type="checkbox"/> 052		YEAR 2024	SEQUENTIAL NUMBER 002	REV NO. 00

NARRATIVE

Initial Plant Condition

Prior to the event, Unit 1 was in Operational Condition (OPCON) 5 (Refueling) with reactor coolant temperature 99 degrees Fahrenheit with a time to boil of 12 hours.

Event Description

On April 11, 2024, a surveillance test was being performed to test the 1B Reactor Protection System (RPS) [EIS System JC] Power Monitoring System by simulating undervoltage, overvoltage, and underfrequency signals for the 1B RPS series circuit breakers. During the testing, power is maintained to the 1B RPS system through a maintenance bypass circuit which is parallel to the series breakers.

On April 11, 2024, at 13:48 EST the station experienced a loss of the RPS panel 1B-Y160 which resulted in a 1B RPS half scram signal and isolation of the following systems: Drywell Chilled Water (DWCW) [EIS System KM], Reactor Enclosure Chilled Water (RECW) [EIS System KM], Reactor Water Cleanup (RWCU) [EIS System CE], Primary Containment Instrument Gas (PCIG) [EIS System LD, and Primary Containment Leak Detector System [EIS System IJ]. Shutdown Cooling [EIS System BO] received an isolation signal but did not isolate because the Shutdown Cooling isolation instruments were bypassed in accordance with station procedures. The B train of Standby Gas Treatment System (SGTS) [EIS System BH] started and was isolated due to drawing the fuel floor to a high negative pressure. The A train of SGTS was started and operated as designed. The cause of the B train of SGTS drawing a high negative pressure was the panel 1B-Y160 supplies power to the refuel floor slide gate damper position. Without this position indication the B train of SGTS will draw excessively from the fuel floor. The plant responded as expected to the loss of the 1B RPS Panel 1B-Y160.

The investigation identified that one of the series circuit breakers, 52-DY24801, appeared to be closed but troubleshooting determined the circuit breaker was open. Additional troubleshooting was performed which cycled the breaker several times to ensure the expected continuity was achieved. The station was able to re-energize the 1B RPS power panel to restore normal ventilation. On April 13, 2024, circuit breaker 52-DY24801 was replaced.

Analysis of the Event

The loss of the RPS bus and associated actuations is considered invalid because it was not the result of a valid signal and not an intentional manual action. The loss of the RPS Panel 1B-Y160 was a result of opening the maintenance bypass switch with one of the series breakers open (52-DY24801). This resulted in containment isolation signals that actuated containment isolation valves in more than one system. Therefore, the loss of the RPS bus is reportable in accordance with 10 CFR 50.73(a)(2)(iv)(A).

Safety Consequences

The system response did not result in a safety system functional failure (SSFF) as defined in accordance with NEI 99-02, "Regulatory Assessment Performance Indicator Guideline." The RPS logic safety function was satisfied by its actuation and responded as expected to the loss of the 1B RPS Panel 1B-Y160.

(04-02-2024)



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NARRATIVE

Cause of the Event

The circuit breaker contacts did not close even though the handle indicated the circuit breaker was in the closed position. Failure analysis on circuit breaker 52-DY24801 identified a mechanical problem with the circuit breaker closing handle. The handle failed to reposition to the tripped position during testing of the circuit breaker. Failure of the circuit breaker to close could not be reproduced in the post-event testing.

Corrective Actions Completed

Circuit breaker 52-DY24801 was replaced.

Corrective Actions Planned

Evaluate options to determine if the series circuit breakers are closed prior to opening the maintenance switch during testing.

Previous Similar Occurrences

There have been no previous similar occurrences for an actuation due to a circuit breaker that appeared to be closed, but was open, within the past five years.

Component Data

System: JC (Plant Protection System)
Component: 52 (Circuit Breaker, AC)
Manufacturer: W121 (Westinghouse)