



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION IV
1600 EAST LAMAR BOULEVARD
ARLINGTON, TEXAS 76011-4511

May 08, 2024

Doug Pehrson, Site Vice President
Arkansas Nuclear One
Entergy Operations, Inc.
N-TSB-58
1448 S.R. 333
Russellville, AR 72802-0967

**SUBJECT: ARKANSAS NUCLEAR ONE – INTEGRATED INSPECTION
REPORT 05000313/2024001 AND 05000368/2024001**

Dear Doug Pehrson:

On March 31, 2024, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Arkansas Nuclear One. On April 11, 2024, the NRC inspectors discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

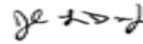
One finding of very low safety significance (Green) is documented in this report. This finding involved a violation of NRC requirements. We are treating this violation as a non-cited violation (NCV) consistent with section 2.3.2 of the Enforcement Policy.

If you contest the violation or the significance of the violation documented in this inspection report, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region IV; the Director, Office of Enforcement; and the NRC Resident Inspector at Arkansas Nuclear One.

If you disagree with a cross-cutting aspect assignment in this report, you should provide a response within 30 days of the date of this inspection report, with the basis for your disagreement, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region IV; and the NRC Resident Inspector at Arkansas Nuclear One.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,



Signed by Dixon, John
on 05/08/24

John L. Dixon, Jr., Chief
Reactor Projects Branch D
Division of Operating Reactor Safety

Docket Nos. 05000313, 05000368
License Nos. DPR-51, NPF-6

Enclosure:
As stated

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ARKANSAS NUCLEAR ONE – INTEGRATED INSPECTION REPORT 05000313/2024001 AND 05000368/2024001 – DATED MAY 08, 2024

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Nos. 05000313, 05000368

License Nos. DPR-5, NPF-6

Report Nos. 05000313/2024001, 05000368/2024001

Enterprise Identifier: I-2024-001-0008

Licensee: Entergy Operations, Inc.

Facility: Arkansas Nuclear One

Location: Russellville, AR

Inspection Dates: January 1, 2024, to March 31, 2024

Inspectors: T. DeBey, Resident Inspector
R. Kopriva, Senior Project Engineer
T. McGowan, Acting Senior Resident Inspector
A. Sanchez, Senior Project Engineer

Approved By: John L. Dixon, Jr., Chief
Reactor Projects Branch D
Division of Operating Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee’s performance by conducting an integrated inspection at Arkansas Nuclear One, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC’s program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

Failure to Properly Evaluate Reactor Coolant System Leakage for Emergency Plan Declarations			
Cornerstone	Significance	Cross-Cutting Aspect	Report Section
Emergency Preparedness	Green NCV 05000313/2024001-01 Open/Closed	[H.12] - Avoid Complacency	71111.11Q
The inspectors identified a Green finding and associated non-cited violation of 10 CFR Part 50, appendix E, section IV.C(2), for the licensee’s failure to establish and maintain the capability to assess, classify, and declare an emergency condition for reactor coolant system leakage that would be timely and accurate. Specifically, the licensee failed to properly calculate reactor coolant system leakage when the water volume being assessed has a density that is significantly different from the reactor coolant system water that is lost from the system.			

Additional Tracking Items

Type	Issue Number	Title	Report Section	Status
LER	05000313/2023-001-00	Automatic Reactor Trip due to Reactor Protection System Actuation	71153	Closed

PLANT STATUS

Unit 1 began the period at full power. On January 21, 2024, Unit 1 experienced an automatic down power to approximately 78 percent due to a false indication of a failed reactor coolant pump input to the integrated control system. The erroneous signal was removed, and the unit was returned to full power later that same day. Unit 1 operated at full power for the remainder of the inspection period.

Unit 2 began the period at full power and remained there for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed activities described in IMC 2515, Appendix D, "Plant Status," observed risk significant activities, and completed on-site portions of IPs. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.01 - Adverse Weather Protection

Impending Severe Weather Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated the adequacy of the overall preparations to protect risk-significant systems from impending severe cold weather on January 11, 2024.

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (2 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 2 reactor protection system during work on the control element drive system on February 6, 2024
- (2) Unit 2 emergency diesel generator train A, during train B inoperability for surveillance activities on March 3, 2024

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) Unit 2 vital dc equipment room, fire zone FZ-2056, on January 3, 2024
- (2) Unit 2 corridor to emergency diesels and battery rooms, fire zone FZ-2049, on January 4, 2024
- (3) Unit 1 main control room, fire zone FZ-1052, on February 6, 2024
- (4) Unit 1 intake structure, fire zones FZ-1030 and FZ-1061, on February 9, 2024
- (5) Unit 1 refuel floor, fire zones FZ-1054 and FZ-1039, on February 26, 2024

Fire Brigade Drill Performance Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated the onsite fire brigade training and performance during an announced fire brigade drill, with personnel injury, in the Unit 1 south emergency diesel generator room, fire zone FZ-87-H, on February 13, 2024.

71111.06 - Flood Protection Measures

Flooding Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated internal flooding mitigation protections in the Unit 2 auxiliary building (535-foot elevation) on March 21, 2024.

71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (2 Samples)

- (1) The inspectors observed and evaluated licensed operator performance in the control room during Unit 2 control element drive troubleshooting, control element movements, and re-pressurization of safety injection tanks on February 14, 2024.
- (2) The inspectors observed and evaluated licensed operator performance in the control room during Unit 1 emergency diesel generator endurance testing on February 17, 2024.

Licensed Operator Requalification Training/Examinations (IP Section 03.02) (2 Samples)

- (1) The inspectors observed and evaluated the Unit 1 simulator exercise involving loss of subcooling margin and emergency system actuation on January 12, 2024.
- (2) The inspectors observed and evaluated the Unit 2 simulator exercises involving excess steam demand on February 27 and 28, 2024.

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (1 Sample)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) Unit 1 125 Vdc battery charger (D03B) maintenance to clean and inspect and load test on January 31, 2024

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) Unit 1 emergent power runback to 78 percent due to integrated control system detecting reactor coolant pump trip on January 21, 2024
- (2) Unit 2 vital inverter 2Y-2224 planned work on March 12, 2024
- (3) Unit 1 emergent work to fix seal leakage on high pressure injection pump P-36B on March 28, 2024

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (4 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) operability assessment of emergency diesel generator 2K-4B after failure and replacement of the 'C' current potential transformer on January 23, 2024
- (2) functionality assessment of site seismic monitors after failure of two of the monitors on January 24, 2024
- (3) functionality assessment of Unit 1 diesel room exhaust fan VEF-24C on March 20, 2024
- (4) functionality assessment of the alternate ac generator on March 26, 2024

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (3 Samples)

The inspectors evaluated the following temporary or permanent modifications:

- (1) Unit 2 temporary modification for emergency diesel generator hallway heater per operating procedure 2106.032, "Unit 2 Freeze Protection Guide" revision 33 on January 10, 2024
- (2) Unit 1 temporary modification for integrated control system runback per engineering change EC-54105613, "Defeat Loss of Single RCP Runback Feature in ICS" on

- February 29, 2024
- (3) Unit 2 temporary modification for control element drive motor control system power after blown fuses and circuit breaker malfunction on March 8, 2024

71111.24 - Testing and Maintenance of Equipment Important to Risk

The inspectors evaluated the following testing and maintenance activities to verify system operability and/or functionality:

Post-Maintenance Testing (PMT) (IP Section 03.01) (3 Samples)

- (1) Unit 2 emergency diesel generator 2K-4B testing after replacement of associated current potential transformer on January 24, 2024
- (2) Unit 2 service water isolation valve to containment cooling (2CV-1511-1) after relay replacement and major preventive maintenance on March 5, 2024
- (3) Unit 1 sluice gate 6 after major preventive maintenance on March 14, 2024

Surveillance Testing (IP Section 03.01) (8 Samples)

- (1) Unit 1 emergency feedwater pump P-7A valve strokes on January 4, 2024
- (2) Unit 1 low pressure injection pump P-34A quarterly run on January 10, 2024
- (3) Unit 2 emergency diesel generator 2K-4B on February 18, 2024
- (4) Unit 2 core protection calculator B temperature inputs calibration for B reactor coolant system temperature input loops on February 28, 2024
- (5) Unit 1 emergency diesel generator K-4A on March 7, 2024
- (6) Unit 2 emergency diesel generator 2K-4B on March 13, 2024
- (7) Unit 2 plant protection system channel A test on March 19, 2024
- (8) Unit 1 engineered safeguards actuation system analog channel 2 test on March 20, 2024

Inservice Testing (IST) (IP Section 03.01) (1 Sample)

- (1) Unit 2 service water isolation valves to spent fuel pool cooling on January 3, 2024

71114.06 - Drill Evaluation

Required Emergency Preparedness Drill (1 Sample)

- (1) ERO Red team (radiological release drill) on February 13, 2024

Additional Drill and/or Training Evolution (1 Sample)

The inspectors evaluated:

- (1) Unit 1 simulator scenario for turbine control system failures on February 1, 2024

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

IE01: Unplanned Scrams per 7000 Critical Hours Sample (IP Section 02.01) (2 Samples)

- (1) Unit 1 (January 1, 2023, through December 31, 2023)
- (2) Unit 2 (January 1, 2023, through December 31, 2023)

IE03: Unplanned Power Changes per 7000 Critical Hours Sample (IP Section 02.02) (2 Samples)

- (1) Unit 1 (January 1, 2023, through December 31, 2023)
- (2) Unit 2 (January 1, 2023, through December 31, 2023)

IE04: Unplanned Scrams with Complications (USwC) Sample (IP Section 02.03) (2 Samples)

- (1) Unit 1 (January 1, 2023, through December 31, 2023)
- (2) Unit 2 (January 1, 2023, through December 31, 2023)

71153 - Follow Up of Events and Notices of Enforcement Discretion

Event Report (IP Section 03.02) (1 Sample)

The inspectors evaluated the following licensee event reports (LERs):

- (1) LER 05000313/2023-001-00, Automatic Reactor Trip due to Reactor Protection System Actuation (Agencywide Documents Access and Management System [ADAMS] Accession No. ML23104A162). The inspection conclusions associated with this LER are documented in Inspection Report 05000313/2023004 (ML24039A084) under Inspection Results Section 71153. This LER is Closed.

INSPECTION RESULTS

Failure to Properly Evaluate Reactor Coolant System Leakage for Emergency Plan Declarations			
Cornerstone	Significance	Cross-Cutting Aspect	Report Section
Emergency Preparedness	Green NCV 05000313/2024001-01 Open/Closed	[H.12] - Avoid Complacency	71111.11Q
The inspectors identified a Green finding and associated non-cited violation of 10 CFR Part 50, appendix E, section IV.C(2), for the licensee’s failure to establish and maintain the capability to assess, classify, and declare an emergency condition for reactor coolant system leakage that would be timely and accurate. Specifically, the licensee failed to properly calculate reactor coolant system leakage when the water volume being assessed has a density that is significantly different from the reactor coolant system water that is lost from the system.			
<u>Description:</u> The inspectors observed a simulator training scenario involving reactor coolant system (RCS) leakage from the pressurizer to the quench tank and noted that no density			

correction was made even though the leaking water cooled significantly when collecting in the quench tank. The leak rate determination (gpm) was made by assuming that the rate of water level increase in the quench tank (gpm) was equal to the rate of RCS water lost from the pressurizer (gpm). The density change in the water as it moved from the pressurizer to the quench tank volume caused a nonconservative assessment of the RCS leak rate. The inspector-identified deficiency stemmed from inadequate procedures for identifying RCS leakage that would exceed either 10 gpm or 25 gpm, requiring the declaration of an Unusual Event (UE). Specifically, determination of RCS leakage by water volume changes is done by site Procedures OP-1103.013, "RCS Leak Detection," revision 48, and OP-1203.039, "Excess RCS Leakage," revision 22.

These procedures may produce leak rate determinations that are significantly nonconservative due to not making an appropriate water density correction. For example, if the rate of RCS leakage from a pressurizer instrument tap leak is determined by the rate of water decrease in the RCS makeup tank, the change of density from the ~110 F makeup tank to the ~650 F pressurizer water will cause a nonconservative error of ~65 percent in the leak rate determination. If unidentified RCS leakage was determined to be causing a makeup tank level decrease of 8 gpm (not a UE entry), the actual RCS (density corrected) leak rate would be ~13.2 gpm (a UE entry for unidentified leakage greater than 10 gpm). Procedure OP-1903.010, "Emergency Action Level Classification," revision 62, states that, for RCS leakage, "The emergency director should declare the event promptly upon determining that the time limit has been exceeded or will likely be exceeded." The nonconservative leak rate evaluation could delay or prevent the required emergency action level declaration.

Corrective Actions: The licensee documented the RCS leakage assessment concern and is evaluating changes to site procedures and training programs.

Corrective Action References: condition reports CR-ANO-C-2023-03340 and CR-ANO-1-2024-00274

Performance Assessment:

Performance Deficiency: The licensee's failure to maintain an appropriate capability to assess, classify, and declare an emergency condition involving excess RCS leakage is a performance deficiency.

Screening: The inspectors determined the performance deficiency was more than minor because it was associated with the Procedure Quality attribute of the Emergency Preparedness cornerstone and adversely affected the cornerstone objective to ensure that the licensee is capable of implementing adequate measures to protect the health and safety of the public in the event of a radiological emergency. Specifically, it could impact the ability to correctly classify a loss of reactor coolant system leakage.

Significance: The inspectors assessed the significance of the finding using IMC 0609, Appendix B, "Emergency Preparedness SDP." The finding was determined to be of very low safety significance (Green) because it could result in a Notice of Unusual Event not being declared when it should have been, resulting in a failure to comply with a regulatory requirement associated with a risk-significant planning standard that did not result in the loss or degradation of that risk-significant planning standard function.

Cross-Cutting Aspect: H.12 - Avoid Complacency: Individuals recognize and plan for the possibility of mistakes, latent issues, and inherent risk, even while expecting successful

outcomes. Individuals implement appropriate error reduction tools. The inspectors determined that the most significant contributing cause of this performance deficiency was failure of the site to recognize the latent issue of density change errors in RCS leakage calculations.

Enforcement:

Violation: Title 10 CFR Part 50, Appendix E, Section IV.C(2), "Activation of Emergency Organization," requires, in part, the licensee to establish and maintain the capability to assess, classify, and declare an emergency condition within 15 minutes after the availability of indications to plant operators that an emergency action level has been exceeded.

Contrary to above, the licensee failed to maintain an appropriate capability to assess, classify, and declare an emergency action level within 15 minutes for an RCS leak rate in excess of emergency action level entry requirements. The ANO Unit 1 Procedures OP-1103.013, "RCS Leak Detection," revision 48, and OP-1203.039, "Excess RCS Leakage," revision 22, for determining leakage from the RCS included potentially significant, nonconservative errors because they did not account for density differences in the water volumes being assessed during RCS leakage determinations. These errors could lead the operators to conclude that an emergency action level had not been exceeded when it had been exceeded.

Enforcement Action: This violation is being treated as a non-cited violation, consistent with section 2.3.2 of the Enforcement Policy.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On April 11, 2024, the inspectors presented the integrated inspection results to Doug Pehrson, Site Vice President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.01	Corrective Action Documents	CR-ANO-	2-2023-02765, C-2022-00735, C-2023-00746	
71111.01	Procedures	OP-1104.039	Plant Heating and Cold Weather Operations	41
71111.01	Procedures	OP-2106.032	Unit 2 Freeze Protection Guide	33
71111.04	Corrective Action Documents	CR-ANO-	2-2020-01467, 2-2024-00167	
71111.04	Drawings	M-2217, Sheet 2	Piping and Instrument Diagram Emergency Diesel Generator Starting Air System	34
71111.04	Drawings	M-2217, Sheet 3	Piping and Instrument Diagram Emergency Diesel Generator Auxiliary Systems	19
71111.04	Miscellaneous	STM 2-31	Emergency Diesel Generators (System Training Manual)	03/03/2024
71111.04	Miscellaneous	ULD-2-SYS-15	ANO-2 Reactor Protection and Core Protection Calculator Systems	5
71111.04	Procedures	OP-2203.030	Remote Shutdown	20
71111.04	Procedures	OP-2304.037	Unit 2 Plant Protection System Channel A Test	56
71111.04	Work Orders	WO	514400, 54108727	
71111.05	Fire Plans	FZ-1030	Intake Structure	3
71111.05	Fire Plans	FZ-1039	Refuel Floor	4
71111.05	Fire Plans	FZ-1052, Fire Zone 129-F	Turbine Building Area Control Room (Unit 1)	4
71111.05	Fire Plans	FZ-1054	Refuel Floor	4
71111.05	Fire Plans	FZ-1061	Intake Structure	3
71111.05	Fire Plans	PFP-U1	Unit 1 Prefire Plans	22
71111.05	Fire Plans	PFP-U2	Unit 2 Prefire Plans	18
71111.05	Miscellaneous	FHA	Fire Hazards Analysis	23
71111.05	Procedures	EN-OP-125	Fire Brigade Drills	0
71111.06	Calculations	CALC-15-E-0007-20	Reduced Latching Requirements for Watertight Doors	0
71111.06	Calculations	CALC-ANOC-CS-15-00003	ANO Flood Protection Design Basis	8
71111.06	Drawings	A-7001, Sheet 1	Flood Protection Drawing Index Penetration Drawing Index, General Notes and Legend	0

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.06	Drawings	A-7001. Sheet 5	Unit 2 Auxiliary Building Flood Protection (EL 335'-0")	2
71111.06	Miscellaneous		Unit 2 Operations Logs	03/22/2024
71111.06	Miscellaneous	Fire Impairment (FI-11537)	Door 455 (opened) in support of Reverse Osmosis Skid Installation (T-MOD)	03/22/2024
71111.06	Procedures	1000.120	ANO Fire Impairment Program	26
71111.06	Procedures	1015.052	Passive Barrier Breach Permitting Process	7
71111.11Q	Corrective Action Documents	CR-ANO-	2-2024-00167	
71111.11Q	Miscellaneous	SES-2-018	Arkansas Nuclear One Unit 2 Dynamic Exam Scenario	14
71111.11Q	Procedures	1903.010	Emergency Action Level Classification	62
71111.11Q	Procedures	2202.001	Standard Post Trip Actions	20
71111.11Q	Procedures	2202.005	Excess Steam Demand	19
71111.11Q	Procedures	2203.024	Loss of Turbine Load	16
71111.11Q	Procedures	EN-OP-115	Conduct of Operations	34
71111.11Q	Procedures	EN-OP-123	Time Critical Action Program Standard	7
71111.11Q	Procedures	EN-TQ-210	Conduct of Simulator Training	17
71111.11Q	Procedures	OP-1104.036	Emergency Diesel Generator Operation	90
71111.11Q	Procedures	OP-1202.002	Loss of Subcooling Margin	12
71111.11Q	Procedures	OP-1202.010	ESAS	15
71111.11Q	Procedures	OP-2105.009	CEDM Control System Operation	46
71111.11Q	Work Orders	WO	53013692, 53033927, 54108727	
71111.12	Procedures	1307.013	Unit 1 Class 1E Battery Charger Load Test	7
71111.12	Procedures	1412.81	Battery Chargers Cleaning and Inspection	22
71111.12	Work Orders	53007808-01	D-03B Perform 18 Month Load Test IAW OP-1307.013	01/30/2024
71111.12	Work Orders	53013380-01	D-03B Thermography Clean Inspect Lube & CK Meter (preventive maintenance)	01/31/2024
71111.13	Calculations	CALC-97-R-1002-01	ECCS Leakage Quantities to Aux Bldg	0
71111.13	Corrective Action Documents	CR-ANO-	1-2024-00084, 1-2024-00400, 2-2021-01235, 2-2024-00380	
71111.13	Procedures	COPD-024	Risk Assessment Guidelines	75
71111.13	Procedures	EN-OP-119	Protected Equipment Postings	18
71111.13	Procedures	EN-WM-104	Online Risk Assessment	27

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.13	Procedures	OP-2104.046	Unit 2 120 VAC 10 kVA Inverter Inspection, Test, and Maintenance Instructions	29
71111.13	Work Orders	WO	53012016, 54123608	
71111.15	Corrective Action Documents	CR-ANO-	1-2023-00288, 1-2024-00373, 2-2002-02036, 2-2023-02352, 2-2023-02353, 2-2023-02996, C-2024-00076, C-2024-00077, C-2024-00099, C-2024-00137, C-2024-00497	
71111.15	Engineering Evaluations	DCP 89-2017	Alternate AC Power Source	2
71111.15	Engineering Evaluations	DCP 92-2011	Alternate AC (AAC) Generator System	0
71111.15	Engineering Evaluations	ER-ANO-2003-0198	Functional Test Phase Tests	03/08/2006
71111.15	Miscellaneous	IN 2007-36	Emergency Diesel Generator Voltage Regulator Problems	11/15/2007
71111.15	Miscellaneous	RG 1.12	Instrumentation for Earthquakes	1
71111.15	Miscellaneous	STM 1-72	Seismic Monitoring System	4
71111.15	Miscellaneous	ULD-0-SYS-19	Alternate AC Generator System (AAC)	2
71111.15	Miscellaneous	ULD-0-TOP-03	Seismic	4
71111.15	Miscellaneous	ULD-2-SYS-01	ANO Unit 2 Emergency Diesel Generator System	10
71111.15	Miscellaneous	Unit 1 UFSAR	Chapter 2.7 Seismology	Amendment 25
71111.15	Procedures	EN-DC-310	Predictive Maintenance Program	12
71111.15	Procedures	OP-2104.036	Emergency Diesel Generator Operations	104
71111.15	Procedures	OP-2104.037	Alternate AC Diesel Generator Operations	38
71111.15	Work Orders	WO	53034621, 54066803, 54089370, 54094397, 54095582	
71111.18	Calculations	CALC-85-E-0053-36	Fire Area JJ Combustible Loading Evaluation	
71111.18	Corrective Action Documents	CR-ANO-	1-2024-00084, 2-2003-0041, 2-2014-00345, 2-2024-00167	
71111.18	Engineering Changes	EC-529999	Justification for Additional Non-Q Battery Room Heating	0
71111.18	Engineering Changes	EC-53838	Unit 2 Combustible Loading Calculations for Area JJ	0
71111.18	Engineering Changes			

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.18	Engineering Changes	EC-54110581	Use Installed Spare Breaker to Re-Energize Power Switch Assembly Associated with CEDMCS Subgroup 6	0
71111.18	Miscellaneous	ER-ANO-2002-0145-000	Justification for Temporary non-Q Battery Room Heating	
71111.18	Miscellaneous	PC 95-8010	Ground Fault Monitoring for MG and CEDMCS	08/23/1995
71111.18	Procedures	OP-2106.032	Unit 2 Freeze Protection Guide	33
71111.18	Work Orders	WO	54105685, 54108727	
71111.24	Corrective Action Documents	CR-ANO-	1-2024-00397, 2-2023-02996, 2-2024-00134, 2-2024-00326, C-2023-00487	
71111.24	Drawings	M-204, Sheet 3	PI&D Unit 1 Emergency Feedwater	37
71111.24	Drawings	M-204, Sheet 4	PI&D Unit 1 Emergency Feedwater	14
71111.24	Drawings	M-204, Sheet 5	PI&D Unit 1 Emergency Feedwater	18
71111.24	Drawings	M-2210, Sheet 3	PI&D Unit 2 Service Water	
71111.24	Engineering Changes	EC-73910	Baseline the Reference Values for the ANO1 IST Components	0
71111.24	Engineering Evaluations	ER-ANO-2004-0507-001	Thermal Binding Evaluation for Stroke Testing CV-1050 and CV-1410 During 1R19 Cooldown	0
71111.24	Miscellaneous		Unit 1 Operations Logs	03/06/2024
71111.24	Miscellaneous	SEP-ANO-1-IST-1	ANO Unit 1 Inservice Testing Bases Document	8
71111.24	Miscellaneous	SEP-ANO-2-IST-1	ANO Unit 2 Inservice Testing Bases Document	10
71111.24	Miscellaneous	ULD-2-TOP-03	ANO-2 Containment Response to Design Basis Accidents	5
71111.24	Procedures	1304.050	Unit 1 ESAS Analog Channel 2 Test	31
71111.24	Procedures	2104.036	Emergency Diesel Generator Operations, Supplement 2C-2DG2 Semi-Annual Test (Fast Start)	104
71111.24	Procedures	2104.036 (Supplement 1A)	2DG1 Monthly Test with 2T-31B Aligned (Slow Start)	104
71111.24	Procedures	2304.037	Unit 2 Plant Protection System Channel A Test	56
71111.24	Procedures	EN-MA-141	Limatorque Valve Operator Model SMB/SB/SBD-000 through 5 MOV and HBC Periodic Inspection	23
71111.24	Procedures	OP-1104.004	Decay Heat Removal Operating Procedure	140
71111.24	Procedures	OP-1104.029	Service Water and Auxiliary Cooling System	128

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.24	Procedures	OP-1104.036	Emergency Diesel Generator Operation	90
71111.24	Procedures	OP-1106.006	Emergency Feedwater Pump Operation	120
71111.24	Procedures	OP-2104.029	Service Water System Operations	127
71111.24	Procedures	OP-2104.033	Containment Atmosphere Control	86
71111.24	Procedures	OP-2104.036	Emergency Diesel Generator Operations	104
71111.24	Procedures	OP-2107.001	Electrical System Operations	139
71111.24	Procedures	STM-2-31	Emergency Diesel Generators (U2 System Training Manual)	03/03/2024
71111.24	Work Orders	53016033-01	Perform 18 Month Test Per 2304.187 CPC Channel B RCE Temp I	02/28/2024
71111.24	Work Orders	54037428-01	Monthly 2 EDG#2 Diesel Test per 2104.036 SUP 2B (Slow Time Allowed)	02/18/2024
71111.24	Work Orders	WO	53018387, 54017970, 54040272, 54058738, 54060059, 54064162, 54094397, 54103261,	
71114.06	Self-Assessments	2024 ANO 24FSD1 Red Team	Radiological Release (simulated)	02/13/2024
71114.06	Self-Assessments	U1 Scenario (DEP)	SES-1-025A (EAL classification)	02/01/2024
71151	Corrective Action Documents	CR-ANO-	1-2023-00743, 2-2023-00026, 2-2023-01929, 2-2023-01938	
71151	Miscellaneous	EN-LI-114, Att 1	NRC Performance Indicator Technique/Data Sheet	20 and 21
71151	Miscellaneous	ESOMS	Narrative Logs	
71151	Miscellaneous	NEI 99-02	Regulatory Assessment Performance Indicator Guideline	7
71153	Corrective Action Documents	CR-ANO-	1-2023-00252, 1-2023-00290, 1-2023-00325, 1-2023-00399, 1-2023-00611, 1-2023-00650, 1-2023-00837	
71153	Miscellaneous	LER 2023-001-00	Automatic Reactor Trip due to Reactor Protection System Actuation	04/14/2023