



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

May 28, 2024

Rod L. Penfield  
Site Vice President  
Vistra Operations Company LLC  
Perry Nuclear Power Plant  
10 Center Rd., Mail Stop A-PY-A290  
Perry, OH 44081-0097

SUBJECT: PERRY NUCLEAR POWER PLANT, UNIT NO. 1 – ISSUANCE OF  
AMENDMENT NO. 204 TO ADOPT TECHNICAL SPECIFICATIONS TASK  
FORCE TRAVELER TSTF-264, REVISION 0 (EPID L-2024-LLA-0009)

Dear Rod Penfield:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 204 to Facility Operating License No. NPF-58 for Perry Nuclear Power Plant, Unit No. 1. This amendment consists of changes to the license and technical specifications (TSs) in response to your application dated January 24, 2024. The amendment revises technical specification (TS) 3.3.1.1, "RPS Instrumentation," in accordance with Technical Specifications Task Force (TSTF) Traveler TSTF-264, Revision 0, "3.3.9 and 3.3.10 - Delete Flux Monitors Specific Overlap Requirement SRs [Surveillance Requirements]." Specifically, the proposed changes delete SRs which verify the overlap between the source range monitor and the intermediate range monitor, and between the intermediate range monitor and the average power range monitor.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's monthly *Federal Register* notice.

Sincerely,

**/RA/**

Scott P. Wall, Senior Project Manager  
Plant Licensing Branch III  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-440

Enclosures:

1. Amendment No. 204 to NPF-58
2. Safety Evaluation
3. Notice and Environmental Finding

cc: Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VISTRA OPERATIONS COMPANY LLC  
ENERGY HARBOR NUCLEAR GENERATION, LLC  
DOCKET NO. 50-440  
PERRY NUCLEAR POWER PLANT, UNIT NO. 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 204  
License No. NPF-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by Energy Harbor Nuclear Corp., et al.,<sup>12</sup> dated January 24, 2024, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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<sup>1</sup> Effective March 1, 2024, Facility Operating License No. NPF-58 was transferred from Energy Harbor Nuclear Generation LLC and Energy Harbor Nuclear Corp. to Energy Harbor Nuclear Generation LLC and Vistra Operations Company LLC, as the licensed owner and operator, respectively. In a letter dated February 23, 2024, Vistra Operations Company LLC requested that the NRC continue the regulatory reviews and actions on the outstanding licensing actions and applications on Docket No. 50-440 (Agencywide Documents Access and Management System Accession No. ML24054A498).

<sup>2</sup> At the time of the application, Energy Harbor Nuclear Corp. was authorized to act as an agent for Energy Harbor Nuclear Generation, LLC and had exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 204, are hereby incorporated into the license. Vistra Operations Company LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of its issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Jeffrey A. Whited, Chief  
Plant Licensing Branch III  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility Operating  
License No. NPF-58 and  
Technical Specifications

Date of Issuance: May 28, 2024

ATTACHMENT TO LICENSE AMENDMENT NO. 204

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Facility Operating License No. NPF-58

Replace the following page of Facility Operating License No. NPF-58 with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

REMOVE

INSERT

-4-

-4-

Technical Specifications

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

3.3-4

3.3-4

3.3-7

3.3-7

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Vistra Operations Company LLC is authorized to operate the facility at reactor core power levels not in excess of 3758 megawatts thermal (100% power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 204, are hereby incorporated into the license. Vistra Operations Company LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Antitrust Conditions

a. Energy Harbor Nuclear Generation LLC shall comply with the antitrust conditions delineated in Appendix C to this license; Appendix C is hereby incorporated into this license.

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.3.1.1.5	Perform CHANNEL FUNCTIONAL TEST.	In accordance with the Surveillance Frequency Control Program
SR 3.3.1.1.6	Not used.	
SR 3.3.1.1.7	Not used.	
SR 3.3.1.1.8	Calibrate the local power range monitors.	In accordance with the Surveillance Frequency Control Program
SR 3.3.1.1.9	Perform CHANNEL FUNCTIONAL TEST.	In accordance with the Surveillance Frequency Control Program
SR 3.3.1.1.10	Calibrate the trip unit.	In accordance with the Surveillance Frequency Control Program

(continued)

Table 3.3.1.1-1 (page 1 of 3)  
Reactor Protection System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION D.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Intermediate Range Monitors					
a. Neutron Flux - High	2	3	H	SR 3.3.1.1.1 SR 3.3.1.1.4 SR 3.3.1.1.13 SR 3.3.1.1.15	≤ 122/125 divisions of full scale
	5 <sup>(a)</sup>	3	I	SR 3.3.1.1.1 SR 3.3.1.1.13 SR 3.3.1.1.15 SR 3.3.1.1.19	≤ 122/125 divisions of full scale
b. Inop	2	3	H	SR 3.3.1.1.4 SR 3.3.1.1.15	NA
	5 <sup>(a)</sup>	3	I	SR 3.3.1.1.15 SR 3.3.1.1.19	NA
2. Average Power Range Monitors					
a. Neutron Flux-High, Setdown	2	3	H	SR 3.3.1.1.1 SR 3.3.1.1.4 SR 3.3.1.1.8 SR 3.3.1.1.11 SR 3.3.1.1.15	≤ 20% RTP
b. Flow Biased Simulated Thermal Power - High	1	3	G	SR 3.3.1.1.1 SR 3.3.1.1.2 SR 3.3.1.1.3 SR 3.3.1.1.8 SR 3.3.1.1.9 SR 3.3.1.1.11 SR 3.3.1.1.14 SR 3.3.1.1.15 SR 3.3.1.1.17 SR 3.3.1.1.18	≤ 0.628 W + 63.8% RTP and ≤ 113% RTP <sup>(b)</sup>

(continued)

(a) With any control rod withdrawn from a core cell containing one or more fuel assemblies.

(b) Allowable Value is ≤ 0.628 W + 43.5% RTP when reset for single loop operation per LCO 3.4.1, "Recirculation Loops Operating."



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO ADOPTION OF TECHNICAL SPECIFICATION TASK FORCE TRAVELER

TSTF-264, REVISION 0,

“3.3.9 AND 3.3.10 – DELETE FLUX MONITORS SPECIFIC OVERLAP

REQUIREMENT SRS [SURVEILLANCE REQUIREMENTS]”

AMENDMENT NO. 204 TO FACILITY OPERATING LICENSE NO. NPF-58

VISTRA OPERATIONS COMPANY LLC

ENERGY HARBOR NUCLEAR GENERATION, LLC

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

DOCKET NO. 50-440

<u>Application (i.e., initial and supplement)</u> <ul style="list-style-type: none"><li>January 24, 2024 ADAMS Accession No. ML24025A011</li></ul>	<u>Safety Evaluation Date</u> May 28, 2024
	<u>Principal Contributors to Safety Evaluation</u> <ul style="list-style-type: none"><li>Scott Wall, NRR</li><li>Tarico Sweat, NRR</li></ul>

1.0 INTRODUCTION

Energy Harbor Nuclear Corp. (EHNC) requested changes to the technical specifications (TSs) for Perry Nuclear Power Plant (PNPP) by license amendment request (LAR, application). The proposed changes would modify TS 3.3.1.1, “RPS Instrumentation,” by deleting surveillance requirements (SRs) 3.3.1.1.6 and 3.3.1.1.7, which verify the overlap between the source range monitor (SRM) and the intermediate range monitor (IRM), and between the IRM and the average power range monitor (APRM).

The proposed changes are based on Technical Specifications Task Force (TSTF) traveler TSTF-264, Revision 0, “3.3.9 and 3.3.10 - Delete Flux Monitors Specific Overlap Requirement SRs”, dated April 28, 1998 (Agencywide Documents Access and Management System Accession No. ML040620165), and the associated U.S. Nuclear Regulatory Commission (NRC, the Commission) staff safety evaluation (SE) of TSTF-264 dated July 26, 1999 (ML19067A141).



By order dated September 28, 2023 (ML23237B427), the NRC staff approved the license transfer of several EHNC-operated plants, including PNPP, from Energy Harbor Nuclear Generation LLC and EHNC to Energy Harbor Nuclear Generation LLC and Vistra Operations Company LLC (VistraOps, the licensee), as the licensed owner and operator, respectively. By letter dated February 23, 2024 (ML24054A498), EHNC informed the NRC that:

Upon completion of the license transfer, VistraOps will adopt and endorse the outstanding commitments, licensing actions, applications, and similar items on the aforementioned docket numbers. VistraOps requests NRC continuation of the regulatory reviews and actions on these items.

On February 29, 2024 (ML24061A100), the EHNC informed the NRC that the transfer transaction would close on March 1, 2024. On March 1, 2024 (ML24057A092), the NRC staff issued Amendment No. 203 to reflect the license transfer. Accordingly, VistraOps is now authorized to act as an agent for Energy Harbor Nuclear Generation, LLC, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility at PNPP.

#### 1.1 Proposed TS Changes to Adopt TSTF-264

The licensee proposes to delete SRs 3.3.1.1.6 and 3.3.1.1.7 from TS 3.3.1.1, "Reactor Protection System (RPS) Instrumentation."

#### 1.2 Additional Proposed TS Changes

TSTF-264 deletes the SRs and all subsequent surveillances are renumbered, with corresponding cross-references to the renumbered surveillances being revised. The licensee is choosing to retain the SR numbers and show them as "Not Used" instead of renumbering the remaining SRs. These differences are editorial and do not affect the applicability of TSTF-264 to the proposed LAR with respect to these facilities.

#### 2.0 REGULATORY EVALUATION

The regulation at paragraph 50.36(c)(3) of Title 10 of the *Code of Federal Regulations* (10 CFR) requires that TSs include items in the category of SRs, which are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation (LCOs) will be met.

Under 10 CFR 50.92(a), determinations on whether to grant an applied-for license amendment are to be guided by the considerations that govern the issuance of initial licenses to the extent applicable and appropriate. Both the common standards for licenses in 10 CFR 50.40(a) (regarding, among other things, consideration of the operating procedures, the facility and equipment, the use of the facility, and other technical specifications, or the proposals) and those specifically for issuance of operating licenses in 10 CFR 50.57(a)(3), provide that there must be reasonable assurance that the activities at issue will not endanger the health and safety of the public, and that the applicant will comply with the Commission's regulations.

The NRC staff's guidance for the review of TSs is provided in chapter 16.0, "Technical Specifications," of NUREG-0800, Revision 3, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR [Light-Water Reactor] Edition" (SRP), March 2010 (ML100351425). As described therein, as part of the regulatory standardization effort, the NRC staff has prepared standard technical specifications (STS) for each of the LWR nuclear designs. Accordingly, the NRC staff's review includes consideration of whether the proposed changes are consistent with the STSs<sup>1</sup>, as modified by NRC approved travelers.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Proposed TS Changes to Adopt TSTF-264

The NRC staff compared the licensee's proposed TS changes in section 1.1 of this SE against the changes approved in TSTF-264. In accordance with SRP, chapter 16.0, the NRC staff determined that the STS changes approved in TSTF-264 are applicable because Perry is a boiling-water reactor (BWR)-design plant, and the NRC staff approved the TSTF-264 changes for BWR designs. The NRC staff finds that the licensee's proposed changes, as stated in section 1.1 of this SE are consistent with those found acceptable in TSTF-264.

##### 3.1.1. Current TS Requirements for Nuclear Instrumentation Neutron Flux Indication Overlap Verification

TS 3.3.1.1, "Reactor Protection System (RPS) Instrumentation," SRs 3.3.1.1.6 and 3.3.1.1.7, require verification of overlap for the SRM and IRM indications, and for the IRM and APRM indications, respectively. The PNPP TS Bases for these SRs (ML23303A136) explain what constitutes acceptable overlap in each case and state, in part, that:

Overlap (nominally 1/2 decade) between IRMs and APRMs exists when sufficient IRMs and APRMs concurrently have onscale readings such that the transition between MODE 1 and MODE 2 can be made without either APRM downscale rod block, or IRM upscale rod block. Overlap (nominally 1/2 decade) between SRMs and IRMs similarly exists when, prior to withdrawing the SRMs from the fully inserted position, IRMs are above 10/125 on range 1 before SRMs have reached the upscale rod block.

The Frequency of SR 3.3.1.1.6 (IRM-SRM overlap verification) is, "Prior to withdrawing SRMs from the fully inserted position." The surveillance column NOTE in SR 3.3.1.1.7 (APRM-IRM overlap verification) states, "Only required to be met during entry into MODE 2 from MODE 1." The Frequency of SR 3.3.1.1.7 is "In accordance with the Surveillance Frequency Control Program."

SR 3.3.1.1.1 requires a CHANNEL CHECK of IRM and APRM channels "In accordance with the Surveillance Frequency Control Program."

As specified in TS Table 3.3.1.1-1, in Mode 2, SR 3.3.1.1.6 applies to RPS Function 1.a, IRM Neutron Flux - High, and SR 3.3.1.1.7 applies to RPS Function 1.a, and Function 2.a, APRM

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<sup>1</sup> U.S. Nuclear Regulatory Commission, "Standard Technical Specifications, General Electric Plants, BWR/6," NUREG-1434, Volume 1, "Specifications," and Volume 2, "Bases," Revision 5.0, September 2021 (ML21271A582 and ML21271A596, respectively).

Neutron Flux - High (Setdown). In addition, SR 3.3.1.1.1 applies to both Function 1.a and Function 2.a.

### 3.1.2. Proposed TS Requirements for Nuclear Instrumentation Neutron Flux Indication Overlap Verification

The proposed change revises TS 3.3.1.1, "RPS Instrumentation," by deleting SR 3.3.1.1.6 and SR 3.3.1.1.7, which verify the SRM-IRM neutron flux indication overlap and the IRM-APRM neutron flux indication overlap, respectively. TS 3.3.1.1 retains the surveillance number designators but replaces the surveillance statement and any associated surveillance NOTE with the phrase "Not Used" and leaves the frequency column blank in the SRs table rows for existing SR 3.3.1.1.6 and SR 3.3.1.1.7.

A CHANNEL CHECK is currently required for Functions 1.a and 2.a with a Frequency of "In accordance with the Surveillance Frequency Control Program." TS Section 1.1 defines a CHANNEL CHECK as follows (the same as the STS definition):

A CHANNEL CHECK shall be the qualitative assessment, by observation, of channel behavior during operation. This determination shall include, where possible, comparison of the channel indication and status to other indications or status derived from independent instrument channels measuring the same parameter.

As explained in TSTF-264, this definition of a CHANNEL CHECK in the STS requires SRM-IRM and IRM-APRM neutron flux indication overlap verification for the IRM-and APRM-associated RPS Functions that are required to be operable in Mode 2 (i.e., Functions 1.a and 2.a). Therefore, consistent with TSTF-264, the present indication overlap verifications may be considered redundant to the channel check SR.

The licensee stated that it will relocate the existing discussions of the SR 3.3.1.1.6 and SR 3.3.1.1.7 overlap verification requirements from the existing Bases to the Bases for SR 3.3.1.1.1. Specifically, the licensee plans to add the following paragraph to SR 3.3.1.1.1 Bases:

The agreement criteria includes an expectation of one decade of overlap when transitioning between neutron flux instrumentation. The overlap between SRMs and IRMs must be demonstrated prior to withdrawing SRMs from the fully inserted position since indication is being transitioned from the SRMs to the IRMs. This will ensure that reactor power will not be increased into a neutron flux region without adequate indication. The overlap between IRMs and APRMs is of concern when reducing power into the IRM range (entry into MODE 2 from MODE 1). On power increases, the system design will prevent further increases (by initiating a rod block) if adequate overlap is not maintained. Overlap between IRMs and APRMs exists when sufficient IRMs and APRMs concurrently have onscale readings such that the transition between MODE 1 and MODE 2 can be made without either APRM downscale rod block, or IRM upscale rod block. Overlap between SRMs and IRMs similarly exists when, prior to withdrawing the SRMs from the fully inserted position, IRMs are above 6/125 on Range 1 before SRMs have reached the upscale rod block.

If overlap for a group of channels is not demonstrated (e.g., IRM/APRM overlap), the reason for the failure of the Surveillance should be determined and the

appropriate channel(s) declared inoperable. Only those appropriate channels that are required in the current MODE or condition should be declared inoperable.

In adopting TSTF-264, the PPNP TS 3.3.1.1 retains the surveillance number designators but replaces the surveillance statement and any associated surveillance note with the phrase "Not Used" and leaves the frequency column blank in the SRs table rows for existing SR 3.3.1.1.6 and SR 3.3.1.1.7. This is in lieu of removing these rows and renumbering the existing subsequent SRs, as done in the traveler's markup of NUREG-1434, Revision 1. In addition, the references to these SRs in TS Table 3.3.1.1-1 are removed. References to SR 3.3.1.1.6 and SR 3.3.1.1.7 in the Bases for TS 3.3.1.2, "Source Range Monitor (SRM) Instrumentation," will be replaced by references to SR 3.3.1.1.1. No changes were needed to TS 3.3.1.2 requirements on SRM channel operability, actions, or surveillances in Mode 2 with IRMs on Range 2 or below.

The NRC staff has determined that the proposed changes are in accordance with NRC-approved TSTF-264, Revision 0, with no technical deviations, are consistent with STS Revision 5.0, and the overlap check will still be performed during surveillance testing.

Additionally, the NRC staff concludes that the requirements of 10 CFR 50.36(c)(3) continue to be met because the revised SR provides the appropriate surveillance to ensure the necessary quality of components is maintained and the LCO will be met. Based on the above, the NRC staff concludes the proposed changes to SR 3.3.1.1.6, SR 3.3.1.1.7, and Table 3.3.1.1 1 of TS 3.3.1.1 to be acceptable.

### 3.2 TS Change Consistency

The NRC staff reviewed the proposed TS changes for technical clarity and consistency with the existing requirements for customary terminology and formatting. The NRC staff finds that the proposed changes are consistent with the terminology and formatting requirements of Chapter 16.0 of the SRP and are therefore acceptable.

### 3.3 Technical Conclusion

The NRC staff reviewed the proposed TS changes for technical clarity and consistency with the existing requirements for customary terminology and formatting. The NRC staff finds that the proposed changes are consistent with chapter 16.0 of the SRP and are therefore acceptable.

## 4.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

NOTICES AND ENVIRONMENTAL FINDINGS  
RELATED TO  
AMENDMENT NO. 204 TO FACILITY OPERATING LICENSE NO. NPF-58  
VISTRA OPERATIONS COMPANY LLC  
ENERGY HARBOR NUCLEAR GENERATION, LLC  
PERRY NUCLEAR POWER PLANT, UNIT NO. 1  
DOCKET NO. 50-440

<u>Application (i.e., initial and supplement)</u> <ul style="list-style-type: none"> <li>January 24, 2024 ADAMS Accession No. ML24025A011</li> </ul>	<u>Safety Evaluation Date</u> May 28, 2024
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1.0 INTRODUCTION

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Upon completion of the license transfer, VistraOps will adopt and endorse the outstanding commitments, licensing actions, applications, and similar items on the aforementioned docket numbers. VistraOps requests NRC continuation of the regulatory reviews and actions on these items.

On February 29, 2024 (ML24061A100), the EHNC informed the NRC that the transfer transaction would close on March 1, 2024. On March 1, 2024 (ML24057A092), the NRC staff issued Amendment No. 203 to reflect the license transfer. Accordingly, VistraOps is now authorized to act as an agent for Energy Harbor Nuclear Generation, LLC, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility at PNPP.

2.0 STATE CONSULTATION

In accordance with the Commission’s regulations, the State of Ohio official was notified of the proposed issuance of the amendment on May 2, 2024. The State official had no comments.

### 3.0 ENVIRONMENTAL CONSIDERATION

The amendment changes requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration in the *Federal Register* on March 19, 2024 (89 FR 19610), and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

SUBJECT: PERRY NUCLEAR POWER PLANT, UNIT NO. 1 – ISSUANCE OF AMENDMENT NO. 204 TO ADOPT TECHNICAL SPECIFICATIONS TASK FORCE TRAVELER TSTF-264, REVISION 0 (EPID L-2024-LLA-0009) DATED MAY 28, 2024

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**ADAMS Accession No.: ML24124A016**

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NAME	DRoth	JWhited	SWall
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