

March 14, 2024

Docket No. 052-050

U.S. Nuclear Regulatory Commission
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SUBJECT: NuScale Power, LLC Submittal of Presentation Material Entitled “ACRS Subcommittee Meeting (Open Session) Chapters 2, 10, 11, 13, 17, and 18,” PM-157982, Revision 0

The purpose of this submittal is to provide presentation materials for use during the upcoming Advisory Committee on Reactor Safeguards (ACRS) NuScale Subcommittee Meeting on March 19, 2024. The materials support NuScale’s presentation of the subject chapters of the US460 Standard Design Approval Application.

The enclosure to this letter is the nonproprietary presentation entitled “ACRS Subcommittee Meeting (Open Session) Chapters 2, 10, 11, 13, 17, and 18,” PM-157982, Revision 0.

This letter makes no regulatory commitments and no revisions to any existing regulatory commitments.

If you have any questions, please contact Jim Osborn at 541-360-0693 or at josborn@nuscalepower.com.

Sincerely,



Thomas Griffith
Manager, Licensing
NuScale Power, LLC

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Enclosure 1: “ACRS Subcommittee Meeting (Open Session) Chapters 2, 10, 11, 13, 17, and 18,” PM-157982, Revision 0

Enclosure 1:

“ACRS Subcommittee Meeting (Open Session) Chapters 2, 10, 11, 13, 17, and 18,” PM-157982, Revision 0

NuScale Nonproprietary



ACRS Subcommittee Meeting

(Open Session)

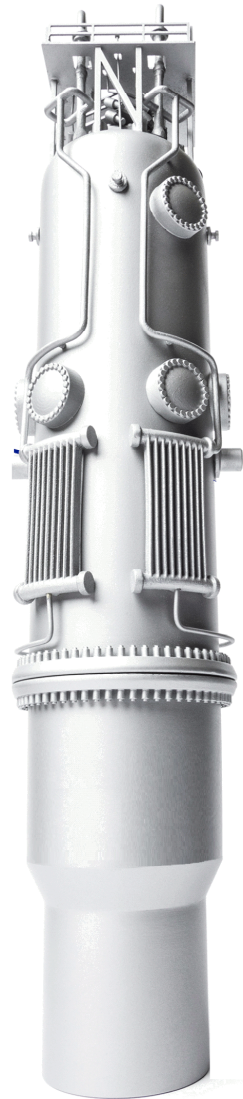
3/19/2024

Chapters 2, 10, 11, 13, 17, and 18

Acknowledgement and Disclaimer

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Chapter 10

Steam and Power Conversion System

3/19/2024

Presenters:

Tyler Beck and Mara Swanson

Chapter 10 - Steam and Power Conversion System

- Section 10.1 - Summary Description
- Section 10.2 - Turbine Generator
- Section 10.3 - Main Steam System
- Section 10.4 - Other Features of Steam and Power Conversion System

Introduction

- Chapter 10 Overview
 - Section 10.1 – Summary Description (No slide included)
- RAI 10.1-1 and Audit Items
- ACCS Summary
- Radiation Protection and DBE Mitigation Features

Section 10.2 - Turbine Generator

- TGS – includes turbine, turbine gland seal, turbine lube oil, turbine control oil, generator, and generator air coolers
 - Main steam feeds turbine through the turbine control valve and stop valve
 - **TGS provides extraction steam to FW heaters**
 - TGS provides gland sealing steam – described in Section 10.4
 - TGS includes the turbine bypass system comprised of desuperheater and turbine bypass valve
- System is SC-III
- Generally Quality Group D, with limited exceptions in Table 10.2-2
- No safety-related or risk-significant SSC
- No major design changes from DCA

Section 10.3 - Main Steam System

- MSS includes piping immediately downstream of the MSIVs up to the TG skid
 - Includes nonsafety-related secondary MSIVs (and associated bypass valves)
 - Includes nonsafety-related MSSVs
- MSS is generally Quality Group D and SC-III
 - Limited exceptions identified in Table 10.3-4
 - Secondary MSIVs (and associated bypass valves) are SC-I
- No safety-related or risk-significant SSC
- Secondary MSIVs – included in TSs
- Design changes from DCA:
 - **Main steam of operating module is preferred auxiliary steam source for startup module**
 - **Extraction steam lines are now part of the TGS**

Section 10.4 - Other Features of Main Steam and Power Conversion System

- **ACCS serves as the main condenser**
 - ACCS condenses steam and provides adequate capacity for the FWS
 - ACCS includes capability for 100% load rejection
 - Not credited for DBE
 - Eliminated need for circulating water system
- FWS supplies feedwater with necessary flow, temperature, and pressure to the SGs
 - No substantial change from DCA
- TGSS provides gland seal steam to prevent leakage into/out of TGS
- **ABS supplies steam to auxiliary steam users when main steam is not available**
 - **DCA: low pressure and high pressure subsystems**
 - High pressure for module heatup system heat exchangers
 - Low pressure for gland seal steam, deaerator, condensate polishing regeneration system
 - **SDAA: auxiliary boiler and chemical skid subsystems**
 - Serves as the low pressure system of the previous ABS when no module is available
- SSC in above systems are generally Quality Group D, SC-III
 - Limited exceptions identified in Table 10.4-4
 - FWRVs and FCVs are SC-I
- No safety-related or risk-significant SSC

RAI 10.1-1 and Audit Items

- RAI 10.1-1:
 - NuScale revised RAI 10.1-1 and provided the nominal heat balance case in FSAR Section 10.1.
- 21 audit items successfully resolved
 - Only unresolved audit item was the heat balance request

ACCS Summary

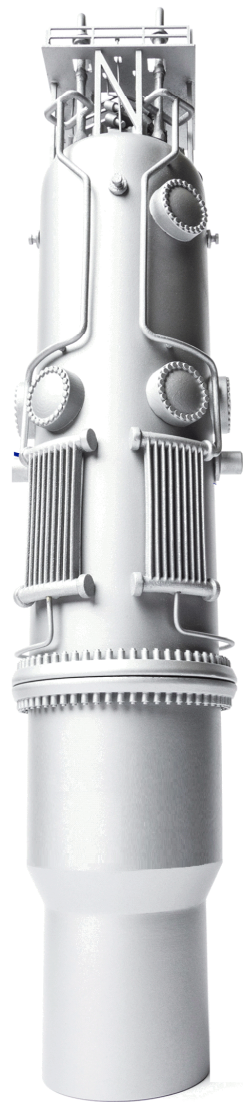
- Air-cooled condensers were selected to allow for the licensee to place the US460 standard plant design in locations where water access is limited.
 - Eliminated the circulating water system
- Principal functions remain consistent with water-cooled condensers
 - Condense exhaust steam from turbine exhaust
 - Reduce dissolved oxygen level in feedwater
 - Maintain ACC vacuum condition by removing air and noncondensibles from the main condenser
 - Provide adequate capacity for condensate and feedwater system during normal operation

Radiation Protection and DBE Mitigation Features

- Effluent and process radiation monitoring is functionally similar to the DCA
 - Radiation monitors allow automatic system isolations and detection of primary-to-secondary leakage.
- Nonsafety-related equipment is credited for event mitigation by functioning as backup protection
 - This is unchanged from the DCA (e.g., secondary MSIVs).
- MPS Interfaces:
 - MPS actuation signals and PAM variables for steam and power conversion systems are unchanged from the DCA.

Acronyms

ABS	Auxiliary Boiler System	MSIV	Main Steam Isolation Valve
ACCS	Air Cooled Condenser System	MSSV	Main Steam Safety Valve
CARS	Condenser Air Removal System	PAM	Post Accident Monitoring
DBE	Design Basis Event	SC	Seismic Classification
DCA	Design Certification Application	SG	Steam Generator
FCV	Feedwater Check Valve	SSC	Systems, Structures, and Components
FW	Feedwater	TBS	Turbine Bypass System
FWIV	Feedwater Isolation Valve	TGS	Turbine Generator System
FWRV	Feedwater Regulation Valve	TS	Technical Specification
FWS	Condense and Feedwater System		
MPS	Module Protection System		
MSIBV	Main Steam Isolation Bypass Valve		



Chapter 11

Radioactive Waste Management

3/19/2024

Presenters:

Seth Robison, Chelsea Lockwood, and Freeda Ahmed

Chapter 11 - Radioactive Waste Management

- Section 11.1 - Source Terms
- Section 11.2 - Liquid Waste Management System
- Section 11.3 - Gaseous Waste Management System
- Section 11.4 - Solid Waste Management System
- Section 11.5 - Process and Effluent Radiological Monitoring and Sampling Systems
- Section 11.6 - I&C Design Features for Radiation Monitoring

Section 11.1 - Source Terms

- Same methodology as DCA
- Updated source term information in Table 11.1-1 through Table 11.1-8
 - Resulting from the change in cycle length, increase in burnup, and change in thermal power
- Audit results
 - NRC review of the dose input and output files associated with the LADTAP and GASPAR code runs (A-11.1-1)
 - Explanation between the differences in the DCA and SDAA source term information (A-11.1-2)
 - Difference between the full power steam flow rate in Chapter 10 and the secondary coolant flow rate in Table 11.1-2. The more conservative secondary coolant flow rate was used in the dose calculation (A-11.1-3)

Section 11.2 - Liquid Waste Management System

- Changes from DCA
 - Removed COL Item on mobile equipment
 - The design allows for at least 30 days of holdup capacity. Description of mobile equipment is needed if there is less than 2 days holdup capacity.
 - Some component changes to the LRWS – concept remains unchanged
 - Similar to DCA - Use of filters, ion exchangers, and reverse osmosis
- Audit results
 - Removal of the COL Item (A-11.2-1)

Section 11.3 - Gaseous Waste Management System

- No changes from DCA
- No audit questions

Section 11.4 - Solid Waste Management System

- Minor system design changes
 - Hard piped connections between the RWBV and LRW and SRW tanks changed to hooded connections
- Audit results
 - Wording clarified regarding the two phase separator tanks and two spent resin tanks (A-11.4-1 and follow-up)
 - Use of mobile equipment wording change. Not needed for SRWS to meet processing requirements (A-11.4-2 and follow-up)
 - Review of the amount of storage space available for Class A waste (A-11.4-3)
 - Clarifying description of reverse osmoses filter membranes (A-11.4-4)
 - Clarification added to Figure 11.4-1 to differentiate between the drum dryer skid and dewatering skid (A-11.4-5)

Section 11.5 - Process and Effluent Radiological Monitoring and Sampling Systems

- Changes from DCA
 - Reduction in the number of modules and associated design changes between the DCA and SDA resulted in a net reduction in the number of radiation monitors
 - Auxiliary Boiler System
 - Circulating Water System eliminated
 - Dry Cooling (Air-cooled condensers)

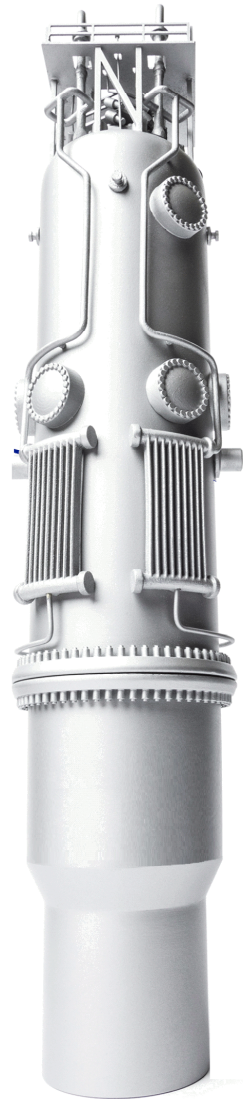
- Audit results
 - Ar-41 for leak detection (A-11.5-1)
 - Monitor Alarms in the main control room (A-11.5-2)
 - Calibration requirement for radiation monitors (A-11.5-3)
 - Sampling Points (A-11.5-4)

Section 11.6 - I&C Design Features for Radiation Monitoring

- Design changes from DCA captured in Section 11.5

Acronyms

COL	Combined License
DCA	Design Certification Application
I&C	Instrument and Controls
LRW	Liquid Radioactive Waste
RWBV	Radioactive Waste Building HVAC
SDAA	Standard Design Approval Application
SDA	Standard Design Approval
SRW	Solid Radioactive Waste
SRWS	Solid Radioactive Waste System



Chapter 18

Human Factors Engineering

3/19/2024

Presenter: Doug Bowman

Chapter 18 - Human Factors Engineering

- Section 18.1 – Human Factors Engineering (HFE) Program Management
- Section 18.2 – Operating Experience Review (OER)
- Section 18.3 – Functional Requirements Analysis and Function Allocation (FRA/FA)
- Section 18.4 – Task Analysis (TA)
- Section 18.5 – Staffing and Qualifications (S&Q)
- Section 18.6 – Treatment of Important Human Actions (IHAs)
- Section 18.7 – Human-System Interface Design
- Section 18.8 – Procedure Development
- Section 18.9 – Training Program Development
- Section 18.10 – Human Factors Verification and Validation (V&V)
- Section 18.11 – Design Implementation (DI)
- Section 18.12 – Human Performance Monitoring

Introduction

- Chapter 18 Overview
- There are not slides for the areas that have not changed
- Other Items

Section 18.3 - Functional Requirements Analysis and Function Allocation

- The purpose of this element is to verify those functions needed to satisfy the plant's safety and commercial goals, and assign those functions to personnel and automation.
 - Takes advantage of human and machine strengths and avoids human and machine limitations
- **As the FRA/FA process has matured, a single, combined and interlinked database has been developed that aligns the HFE task analysis, FRA/FA database, the Operator Training Task Analysis and the Plant Operating Procedure set**

Section 18.4 - Task Analysis

- TA identifies specific tasks (human actions) required to satisfy the functions from the FRA/FA element
- Similar process to DCA
 - Now combined in a single, interlinked database as described in Section 18.3

Section 18.5 - Staffing and Qualifications

- S&Q determines the number and qualification of licensed operators required for safe and reliable plant operation
 - **Minimum staffing requirement is one licensed RO and two licensed SROs**
- Changes from the DCA:
 - For the DCA: Minimum staffing requirements are located in Part 52 Appendix G
 - **For the SDAA: Technical basis and approach for minimum staffing requirements is the “NuScale Control Room Staffing Plan,” TR-0420-69456-NP-A**
 - Previously reviewed by ACRS (ML21139A226 and ML21139A232 [April 2021])

Section 18.6 - Treatment of Important Human Actions

- Identification of IHAs within the scope of Chapters 7, 15, and 19
- PRA (Chapter 19) determines risk-important human actions
- Deterministic human actions are those credited in Chapter 15 and D3 (Diversity and Defense-in-Depth) coping analyses of Ch 7 (e.g., those required for long-term decay heat removal or reactivity control)
- US460 Standard Design does not have IHAs
- Changes from the DCA:
 - DCA included two IHAs → No longer RIHAs in the SDA (see Chapter 19)
 - **The DCA IHAs are still addressed and mitigating strategies accounted for in the current SDA generic technical guidelines, HSI design, HFE and training task analysis**

Section 18.7 - Human-System Interface Design

- HSI design element establishes the HSI design
- Substantially similar main control room/HSI as the DCA
 - **US460 changes are in response to ISV and design changes from the US600 design**

Other Items

New HFE-related ITAAC:

<u>02.</u>	<u>The MCR design incorporates HFE principles that reduce the potential for operator error.</u>	<u>An integrated system validation (ISV) test is performed in accordance with the Verification and Validation Implementation Plan.</u>	<u>A report exists and concludes that acceptance criteria associated with each ISV test scenario are satisfied upon initial performance of the scenarios or upon remediation of failures.</u>
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SDAA Audit

- 19 audit items successfully resolved
- Included virtual demonstration of simulator and staff review of databases

Acronyms

DI	Design Implementation	SRO	Senior Reactor Operator
FRA/FA	Functional Requirements Analysis and Function Allocation	TA	Task Analysis
HFE	Human Factors Engineering	V&V	Verification and Validation
HSI	Human-System Interface		
HPM	Human Performance Monitoring		
IHA	Important Human Action		
IP	Implementation Plan		
ISV	Integrated System Validation		
MCR	Main Control Room		
OER	Operating Experience Review		
PRA	Probabilistic Risk Assessment		
RIHA	Risk Important Human Action		
RO	Reactor Operator		
RSR	Results Summary Report		
S&Q	Staffing and Qualifications		

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ACRS Subcommittee Meeting

(Open Session)

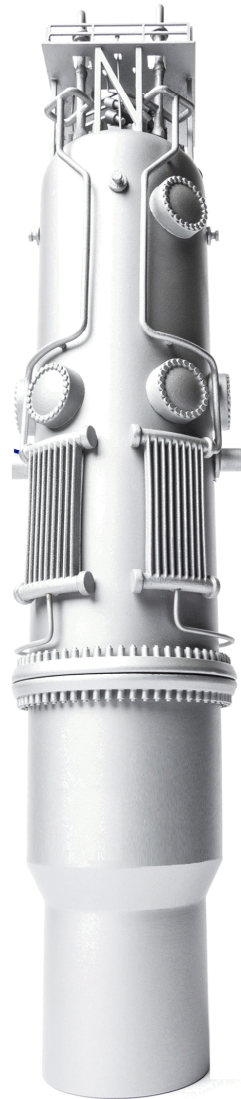
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Chapters 2, 13, and 17

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Chapter 2

Site Characteristics and Site Parameters

3/19/2024

Presenter: Tyler Beck

Chapter 2 - Site Characteristics and Site Parameters

- Section 2.0 - Site Characteristics and Site Parameters
- Section 2.1 - Geography and Demography
- Section 2.2 - Nearby Industrial, Transportation, and Military Facilities
- Section 2.3 - Meteorology
- Section 2.4 - Hydrologic Engineering
- Section 2.5 - Geology, Seismology, and Geotechnical Engineering

Note: Chapter 2 scope is largely site-specific. Each Section includes a COL item to ensure the site-specific values are provided.

Section 2.0 - Site Characteristics and Site Parameters

- The NuScale Power Plant US460 standard design uses site parameters that are representative of a reasonable number of potential plant site locations in the United States. Table 2.0-1 summarizes these parameters
- Applicants will verify the site-specific parameters

Section 2.1 - Geography and Demography

- The NuScale Power Plant US460 standard design considers the exclusion area boundary and low population zone outer boundary are as close as 369 feet from the nearest release point (i.e., site boundary)
 - This is a change from 400 feet in the DCA
- The only change from the DCA is the exclusion area boundary and low population zone outer boundary

Section 2.2 - Nearby Industrial, Transportation, and Military Facilities

- The SDAA does not postulate hazards from nearby industrial, transportation, or military facilities
 - Nearby facilities and potential resulting hazards are entirely in the scope of COL Item 2.2-1
- No change from the DCA

Section 2.3 - Meteorology

- The NuScale Power Plant US460 standard design uses meteorological parameters that are representative of a reasonable number of potential plant sites in the US
 - Includes precipitation, design basis tornado/hurricane, snow loads, and other similar parameters
- Applicants will describe the site-specific meteorology
- Changes from the DCA:
 - SDAA considers more limiting design basis tornado than the DCA
 - Difference in atmospheric dispersion values pertains to a different source-to-receptor distance for the US460
 - Atmospheric dispersion values at the exclusion area boundary are similar to the DCA values (see Section 2.1 for change in exclusion area boundary)
 - Atmospheric dispersion values at MCR are lower than the DCA values
 - Routine release values are lower than the DCA values

Section 2.4 - Hydrologic Engineering

- The NuScale Power Plant US460 standard design does not rely on external water supply for the ultimate heat sink or safety-related makeup water. The design reduces the need for local hydrologic features for plant safety
- COL Item 2.4-1 requires an applicant to describe the site-specific hydrologic characteristics for sections 2.4.1 through Section 2.4.14, except Section 2.4.8, Section 2.4.10, and Section 2.4.11
 - 2.4.8 – Cooling Water Canals and Reservoirs
 - 2.4.10 – Flood Protection Requirements
 - 2.4.11 – Low Water Considerations
- No major changes to Section 2.4
 - Exclusion of Section 2.4.11 from COL Item 2.4-1

Section 2.5 - Geology, Seismology, and Geotechnical Engineering

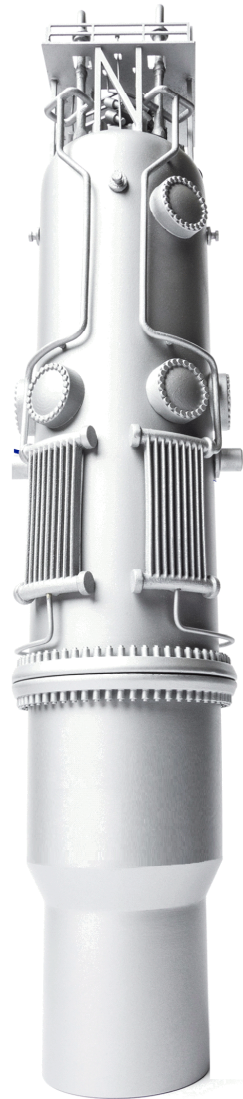
- The NuScale Power Plant US460 standard design uses geologic, seismologic, and geotechnical engineering parameters that are representative of a reasonable number of potential plant site locations in the US.
- Section 2.5 is site-dependent
- 2.5.1 (Basic Geologic and Seismic Information), 2.5.3 (Surface Deformation), and Section 2.5.5 (Stability of Slopes)
 - No change from DCA
- 2.5.2 (Vibratory Ground Motion)
 - Certified seismic design response spectra (CSDRS and CSDRS-HF) are unchanged from the DCA
 - Addressed in Section 3.7
- 2.5.4 (Stability of Subsurface Materials and Foundations)
 - The bearing capacity and settlement values have changed in the SDAA

Audit Summary

- Resolution of 10 audit items during the staff's audit

Acronyms

COL	Combined License
CSDRS	Certified Seismic Design Response Spectra
DCA	Design Certification Application
HF	High Frequency
LPZ	Low Population Zone
SDAA	Standard Design Approval Application
SDA	Standard Design Approval



Chapter 13

Conduct of Operations

3/19/2024

Presenter: Beth Brewer

Chapter 13 - Conduct of Operations

- Section 13.1 - Organizational Structure
- Section 13.2 - Training
- Section 13.3 - Emergency Planning
- Section 13.4 - Operational Programs
- Section 13.5 - Plant Procedures
- Section 13.7 - Fitness for Duty

- NOT INCLUDED: Section 13.6 - Security

Section 13.1 - Organizational Structure

- Changes from the DCA
 - Minor editorial changes
 - No technical changes
- RAIs/Audit
 - No RAIs or Audit Questions

Section 13.2 - Training

- Changes from the DCA
 - Minor editorial changes
 - No technical changes
- RAIs/Audit
 - No RAIs or Audit Questions

Section 13.3 - Emergency Planning

- Changes from the DCA
 - Minor editorial changes
 - TSC changed elevation in the control building and went from seismic category I to seismic category II (fully compliant)
 - Revised the SDA to clearly state that the TSC displays use the same I&C networks used in the MCR but are configured to provide display only, no controls.
 - Went from three COL Items in the DCA to two in the SDAA
 - The DCA had separate COL Items for descriptions of the OSC and EOF and these were combined into one broader COL Item that requires the applicant to describe the emergency response facilities.
 - DCA COL Item 13.3-1 required the applicant to describe direct communication system or systems between the OSC and the control room and this was eliminated from the SDAA COL Items and included directly in the SDAA text.
 - Regulations were updated in the SDAA: 10 CFR 50.47 and 10 CFR 50 Appendix E were removed from the DCA COL Items, and the SDAA COL Items refer to 10 CFR 50 generally due to rulemaking in process during SDA development (new rule 10 CFR 50.160).
 - Removed reference to 10 CFR 52.48 because it is a standard design certification requirement
- RAIs/Audit
 - RAI 10097, Questions 13.3-1, 13.3-2, 13.3-3: Needed additional design descriptions to explain how the TSC meets NUREG-0696 and Supplement 1 to NUREG-0737
 - Section 13.3 was revised to add additional information to address RAIs

Section 13.4 - Operational Programs

- Changes from the DCA
 - Minor editorial changes
 - Removed Reactor Vessel Material Surveillance Program and Motor-Operated Valve Testing Program from the COL Item
 - Not applicable to US460 design
- RAIs/Audit
 - No RAIs or Audit Questions

Section 13.5 - Plant Procedures

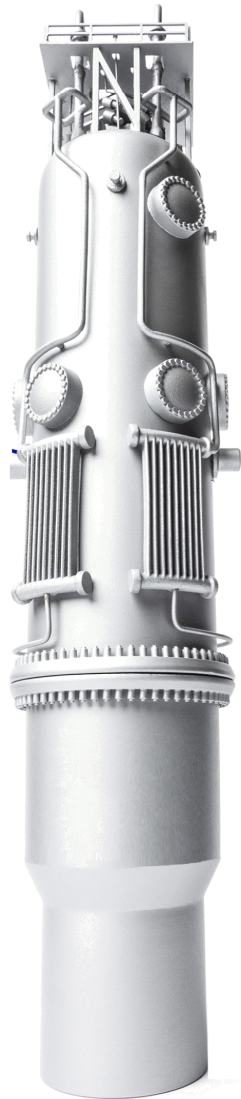
- Changes from the DCA
 - Section 13.5.2.1 – removed discussion about Generic Technical Guidelines
 - Clarified and consolidated the information concerning how the Generic Technical Guidelines will be used to develop site specific emergency operating procedures into SDA COL Item 13.5-5, and then provide a process to maintain them in COL Item 13.5-3
 - The plant specific technical guidelines developed by a COL holder will be nearly identical to the Generic Technical Guidelines provided to the applicant prior to COLA submittal.
 - Renumbered COL Items
- RAIs/Audit
 - No RAIs or Audit Questions

Section 13.7 - Fitness-For-Duty

- Changes from the DCA
 - Removed two COL Items related to the operational and construction Fitness-For-Duty programs
 - An applicant referencing the standard design is responsible for providing an FFD program description and implementation as described in 10 CFR Part 26
- RAIs/Audit
 - No RAIs or Audit Questions

Acronyms

COL	Combined License
COLA	Combined License Application
DCA	Design Certification Application
EOF	Emergency Operations Facility
FFD	Fitness for Duty
I&C	Instrumentation & Control
MCR	Main Control Room
OSC	Operational Support Center
RAI	Request for Additional Information
SDAA	Standard Design Approval Application
SDA	Standard Design Approval
TSC	Technical Support Center



Chapter 17

Quality Assurance and Reliability Assurance

3/19/2024

Presenter: Amanda Bode

Chapter 17 - Quality Assurance and Reliability Assurance

- Section 17.1 – Quality Assurance During the Design Phase
- Section 17.2 – Quality Assurance During the Construction and Operation Phases
- Section 17.3 – Quality Assurance Program Description
- Section 17.5 – Quality Assurance Program Description
- Section 17.6 – Maintenance Rule

- NOT INCLUDED: Section 17.4 – Reliability Assurance Program

Chapter 17

Section	Description	Remarks
17.1	Quality Assurance during Design Phase	Described in Section 17.5
17.2	Quality Assurance during Construction and Operation Phase	Not applicable to SDAA. COL applicant describes the quality assurance program applicable to site-specific design activities and to the construction and operations phases.
17.3	Quality Assurance Program Description	Described in Section 17.5
17.5	Quality Assurance Program Description – Design Certification, Early Site permits, and New License Applicants	Does not address construction and design QA activities that begin at construction
17.6	Maintenance Rule	Not applicable to SDAA. The maintenance rule operational program is the responsibility of an applicant.

Section 17.5 - Quality Assurance Program Description

- Quality Assurance Program Description (QAPD) for the NuScale Power Plant US460 is provided in the NRC approved topical report
 - “NuScale Power, LLC Quality Assurance Program Description” (MN-122626-A, Revision 1)
- The NuScale Quality Assurance Plan is established in accordance with 10 CFR 50, Appendix B, 10 CFR 52, and 10 CFR 21 based on the requirements and recommendations of ASME NQA-1-2008 and NQA-1a-2009 addenda, Parts I and II, as endorsed by Regulatory Guide 1.28, Revision 4
 - Safety Evaluation published December 2023
 - NRC inspection of NuScale’s QA program performed February 26- March 1

RAIs and Audit questions

- No RAIs on Chapter 17 (minus 17.4)
- No Audit questions on Chapter 17 (minus 17.4)

Acronyms

COL	Combined License
QA	Quality Assurance
QAPD	Quality Assurance Program Description
RAI	Request for Additional Information
SDAA	Standard Design Approval Application
SDA	Standard Design Approval