



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 10, 2024

Heath Baldner – Director, Licensing
Engineering
NAC International
2 Sun Court, Suite 220
Peachtree Corners, GA 30092

SUBJECT: ACCEPTANCE OF NAC INTERNATIONAL REQUEST FOR AMENDMENT NO. 9, AND REVISION TO AMENDMENT NOS. 6, 7, AND 8, OF THE CERTIFICATE OF COMPLIANCE NO. 1025 FOR THE NAC MULTI-PURPOSE CANISTER SYSTEM - ENTERPRISE PROJECT IDENTIFICATION NUMBERS L-2023-LLA-0149 AND L-2023-LLA-0148

Dear Heath Baldner:

By letter dated September 7, 2023 (Agencywide Documents Access and Management System [ADAMS] Accession No. ML23250A056), you submitted an application for Amendment No. 9, and Revision to Amendment Nos. 6, 7, and 8, of the Certificate of Compliance (CoC) No. 1025 for the NAC Multi-Purpose Canister System. You requested to revise the description of the Vertical Concrete Cask in the technical specifications to make a distinction between the Vertical Concrete Cask (body) and the Vertical Concrete Cask Lid, in terms of applicability of the American Concrete Institute (ACI) Specifications ACI-349 and ACI-318.

The U.S. Nuclear Regulatory Commission (NRC) staff completed an acceptance review of your submittal and have identified no administrative omissions or deficiencies that would prevent us from proceeding with a detailed review. The actions included in your submittal for Amendment No. 9, and Revision to Amendment Nos. 6, 7, and 8 have been assigned Cost Activity Code/Enterprise Project Identification Numbers 001028 / L-2023-LLA-0149 and 001028/ L-2023-LLA-0148, respectively. Please reference these numbers and Docket No. 72-1025 in any future correspondence associated with this request.

The enclosure includes an observation on information that the NRC staff could potentially request later as a request for additional information (RAI) during the detailed technical review. Observations are not the result of a detailed technical review and may be resolved during the staff's detailed review. The staff included the observation to allow you to start earlier on this item containing the potential to be asked later as an RAI. A response to the observation is not required for the NRC staff to begin its detailed technical review.

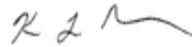
Please note that the NRC staff's technical review of your submittal may identify issues that could require that you provide additional information. Based on our preliminary evaluation and projection of current review schedules, we anticipate completing our review and submitting the draft CoC and preliminary safety evaluation report to Rulemaking in September 2024. This schedule includes the issuance, if needed, of an RAI in March 2024, and receiving your response to the RAI 30 days after its issuance. Rulemaking is estimated to take approximately 26 weeks, and if there are no significant adverse comments received during the comment

period, the CoC amendment and revisions are estimated to become effective in March 2025. Please note that these dates could change depending on the findings of our technical review, urgent assignments, or other factors, such as whether your responses to the RAI are or not adequate. The NRC staff estimates that completing the review of your submittal will require approximately 240 staff hours. We will promptly communicate significant schedule and/or cost changes.

In accordance with Title 10 of the *Code of Federal Regulations* (10 CFR) Section 2.390, "Public inspections, exemptions, requests for withholding," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room (PDR) or from the Publicly Available Records component of the NRC's ADAMS. ADAMS is accessible from the NRC website at <http://www.nrc.gov/reading-rm/adams.html>. The PDR is open by appointment. To make an appointment to visit the PDR, please send an email to PDR.Resource@nrc.gov or call 1-800-397-4209 or 301-415-4737, between 8:00 a.m. and 4:00 p.m. eastern time (ET), Monday through Friday, except Federal holidays.

If you have any questions regarding this communication, please contact me at (301) 415-7116, or via email to Kristina.Banovac@nrc.gov.

Sincerely,



Signed by Banovac, Kristina
on 01/10/24

Kristina L. Banovac, Project Manager
Storage and Transportation Licensing Branch
Division of Fuel Management
Office of Nuclear Material Safety
and Safeguards

Docket No.: 72-1025
EPID Nos.: L-2023-LLA-0149
L-2023-LLA-0148

Enclosure:
Observation

SUBJECT: ACCEPTANCE OF NAC INTERNATIONAL REQUEST FOR AMENDMENT NO. 9, AND REVISION TO AMENDMENT NOS. 6, 7, AND 8, OF THE CERTIFICATE OF COMPLIANCE NO. 1025 FOR THE NAC MULTI-PURPOSE CANISTER SYSTEM - ENTERPRISE PROJECT IDENTIFICATION NUMBERS L-2023-LLA-0149 AND L-2023-LLA-0148

DOCUMENT DATE: January 10, 2024

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**Observation
NAC International
Model No. NAC Multi-Purpose Canister System
Docket No. 72-1025
Certificate of Compliance No. 1025
Amendment No. 9, and Revision to Amendment Nos. 6, 7, And 8**

This observation identifies information that the U.S. Nuclear Regulatory Commission (NRC) staff could potentially request later as a request for additional information (RAI) during the detailed technical review. The observation does not require a response for the NRC staff to begin its detailed technical review of the application. However, NAC International (the applicant) may respond to the observation before the NRC staff formally makes any RAI.

Structural Evaluation

Clarify if the minimum required concrete density for the vertical storage cask lid, for the multi-purpose canister (MPC) storage system for the La Crosse Boiling Water Reactor (MPC-LACBWR), to meet shielding requirements is 145 pounds per cubic foot (pcf).

Per the final safety analysis report (FSAR), Section 1.A.2.1.2, the top of the MPC-LACBWR vertical storage cask is closed by a lid with integral radiation shield. The radiation shield is approximately 8-inch-thick concrete encased in a carbon steel shell extending into the cask cavity from the bottom surface of the 1.5-inch-thick carbon steel lid. The specification summary for the encased concrete lid per FSAR Table 1.A.2-6 and the proposed Technical Specification Appendix B, Section 3.3, requires standard weight concrete density to be 140 pcf (minimum). However, FSAR Table 5.A.3-8, "MPC-LACBWR Structural and Shield Material Regional Densities," for the shielding analysis shows concrete density to be 2.3233 gram/cubic centimeter, which is 145 pcf. Therefore, a clarification is required for the minimum required concrete density to meet shielding requirements.

This information is necessary to ensure compliance with the requirements in Title 10 of the *Code of Federal Regulations* Section 72.236(d).