

PSL L-23-1 NRC INITIAL WRITTEN EXAM - SRO

76.

Given the following:

- Unit 2 has experienced a Small Break LOCA
- 2-EOP-03, Loss of Coolant Accident (LOCA), is in progress
- All available charging pumps are RUNNING
- Containment Pressure is 0.75 psig and slowly RISING
- RCS Tcold is 525°F and slowly LOWERING
- Pressurizer Pressure indicates 1820 psia and slowly LOWERING
- Pressurizer Level indicates 7% and slowly LOWERING
- RCS cooldown is in progress

Subsequently:

- Annunciator R-8, SIAS Channel A/B Actuation Block Permissive, is in ALARM

Which ONE of the following completes the statements?

In accordance with 2-EOP-03, the US ____ (1) ____ direct the RCO to BLOCK the Safety Injection Actuation Signal (SIAS).

IF blocking SIAS is required, the RCO would utilize a ____ (2) ____ to block SIAS.

A.	(1)	will
	(2)	key switch
B.	(1)	will
	(2)	think pushbutton AND hand switch
C.	(1)	will NOT
	(2)	key switch
D.	(1)	will NOT
	(2)	think pushbutton AND hand switch

77.

Given the following conditions:

- Unit 2 tripped due to LOCA
- RCPs were secured due to a loss of CCW flow
- 2-EOP-03, LOCA is in progress at Step 19, COOLDOWN the RCS
- Containment Temperature is 205°F
- Containment Pressure is 3.8 psig
- Pressurizer Level indicates 0%
- Pressurizer Pressure is 1000 psia
- REP CET is 500°F
- S/G Pressures are 700 psia
- ECCS flow is MET per 2-EOP-99, Appendices / Figures / Tables / Data Sheets

Which ONE of the following completes the statement below?

The RCO will be directed to OPERATE the ____ (1) ____ to cooldown the RCS, NOT to exceed ____ (2) ____ in any one hour.

REFERENCE PROVIDED

A.	(1)	ADVs
	(2)	50°F
B.	(1)	ADVs
	(2)	100°F
C.	(1)	SBCS
	(2)	50°F
D.	(1)	SBCS
	(2)	100°F

78.

Given the following:

- Unit 1 is 100% power
- Component Cooling Water (CCW) Surge Tank levels are slowly RISING
- RR-26-56/57, CCW Radiation Recorder, trends are RISING
- Chemistry CCW sample indicates rising activity
- TIS-14-32A1, 1A1 RCP Seal Cooler CCW Outlet Temperature, is 205 °F and RISING
- 1-AOP-14.02, Component Cooling Water Excessive Activity, is in progress

Which ONE of the following completes the statements?

HCV-14-11A1, 1A1 RCP Seal Cooler HX Isolation Valve, ____ (1) ____ CLOSED.

Based on these indications, the US will ____ (2) ____.

A.	(1)	is
	(2)	remain in 1-AOP-14.02, Component Cooling Water Excessive Activity
B.	(1)	is
	(2)	transition to 1-AOP-01.09A1, Reactor Coolant Pump
C.	(1)	is NOT
	(2)	remain in 1-AOP-14.02, Component Cooling Water Excessive Activity
D.	(1)	is NOT
	(2)	transition to 1-AOP-01.09A1, Reactor Coolant Pump

79.

Given the following:

- Unit 1 is 100% power
- PIC-1100X, Pressurizer Pressure, indicates 2500 psia
- PIC-1100Y, Pressurizer Pressure, indicates 2150 psia and LOWERING
- 1-AOP-01.10, Pressurizer Pressure and Level, is in progress

Which ONE of the following completes the statements?

____(1)____ Pressurizer Spray Valve(s) is(are) FULL OPEN.

In accordance with ADM-25.04, Tech Spec Bases, the bases for the TMLP Reactor Trip setpoint is to ____ (2) ____.

Note: Departure from Nucleate Boiling Ratio is DNBR

A.	(1)	one
	(2)	prevent violation of the DNBR limit
B.	(1)	one
	(2)	ensure fuel centerline melting will NOT occur
C.	(1)	two
	(2)	prevent violation of the DNBR limit
D.	(1)	two
	(2)	ensure fuel centerline melting will NOT occur

80.

Given the following:

- Unit 1 is at 100% power

Subsequently:

- The 1A Main Feedwater TRIPS
- The Reactor Protection System (RPS) FAILS to AUTOMATICALLY TRIP the Reactor

Which ONE of the following completes the statements?

The Diverse Scram System (DSS) trips the reactor by OPENING the CEA Drive MG Set ____ (1) ____.

In accordance with UFSAR 7.6.1.4, the basis for ATWS worst case result in a(n) ____ (2) ____ event.

A.	(1)	supply breaker
	(2)	core melt
B.	(1)	supply breaker
	(2)	RCS over pressurization
C.	(1)	output contactor
	(2)	core melt
D.	(1)	output contactor
	(2)	RCS over pressurization

81.

Given the following:

- Unit 2 is at 4% power
- SBCS is in service
- Main Feed is in service
- F-5, Instr Air Press High / Low, is in ALARM
- PI-18-9/16, Instr Air & Sta Air Pressure, is 70 psig and SLOWLY LOWERING
- The NPO reports an Instrument Air leak in the Turbine Building
- The crew implements 2-AOP-18.01, Instrument Air Malfunction

Subsequently:

- NR S/G Levels are 61% and slowly LOWERING

Which ONE of the following completes the statements?

In accordance with 2-AOP-18.01, Instrument Air Malfunction:

RCS Heat Removal will be provided by the ____ (1) ____.

To stabilize S/G levels the US will direct implementation of ____ (2) ____.

A.	(1)	ADVs
	(2)	2-AOP-09.02, Auxiliary Feedwater
B.	(1)	ADVs
	(2)	2-AOP-22.01, Rapid Downpower
C.	(1)	SBCS
	(2)	2-AOP-09.02, Auxiliary Feedwater
D.	(1)	SBCS
	(2)	2-AOP-22.01, Rapid Downpower

82.

Given the following:

07:00	Unit 2 is 100% power CEA 52 DROPS to the bottom of the core All OTHER CEAs are at 133 inches
07:05	2-AOP-66.01, Dropped or Misaligned CEA Abnormal Operations, is in progress Reactor Power is 97% and STABLE Tavg-Tref are MATCHED and STABLE
07:45	CEA 52 is determined to be functional and OPERABLE F_r^T from Plant Physics Curve Book Figure C.3. is 1.55 COLR Figure 3.1-1a, the allowable time to align CEA to its group is 63 minutes

Which ONE of the following completes the statement?

In accordance with 2-AOP-66.01, CEA 52 must be restored to a MINIMUM CEA position of ____ (1) ____ inches withdrawn by NO LATER THAN ____ (2) _____. or the US will direct a required downpower.

A.	(1)	118
	(2)	08:00
B.	(1)	118
	(2)	08:03
C.	(1)	126
	(2)	08:00
D.	(1)	126
	(2)	08:03

83.

Given the following:

- Unit 1 is shutting down for a refueling outage per 1-GOP-123, Turbine Shutdown - Full Load To Zero Load
- The RCO DEPRESSES the Manual Reactor Trip Pushbuttons
- The Core Mimic (Display 6801) light for CEA 2 indicates WHITE
- ALL other Core Mimic (Display 6801) CEA lights indicate AMBER

Subsequently:

- BRCO reports "The Reactor is tripped, assessing CEAs."
- The US receives NO further reports from the BRCO

Which ONE of the following completes the statements?

The US will communicate ____ (1) ____.

In accordance with Tech Spec Bases 3.1.1, Shutdown Margin is most limiting at the ____ (2) ____.

A.	(1)	"all CEAs are fully inserted"
	(2)	beginning of core life
B.	(1)	"all CEAs are fully inserted"
	(2)	end of core life
C.	(1)	"commence emergency boration"
	(2)	beginning of core life
D.	(1)	"commence emergency boration"
	(2)	end of core life

84.

Given the following:

00:00	Unit 1 is performing a cooldown using SBCS in accordance with 1-GOP-305, Reactor Plant Cooldown - Hot Standby to Cold Shutdown RCS Temperature is 450 °F and slowly LOWERING Pressurizer Pressure is 1000 psia and STABLE
00:05	Annunciator Q-16, CNTMT RAD HIGH CIS CHANNEL TRIP, is in ALARM ALL four Containment Isolation Radiation Monitors are in HIGH ALARM
00:21	A Containment Isolation flow path is discovered OPEN and was manually isolated

Which ONE of the following completes the statements?

At time 00:05, Containment Isolation ____ (1) ____.

At time 00:21, in accordance with the EAL - Hot Basis, the Emergency Plan Classification threshold for SU8, Failure to Isolate Containment time limit ____ (2) ____ been exceeded.

A.	(1)	will AUTOMATICALLY actuate
	(2)	has
B.	(1)	will AUTOMATICALLY actuate
	(2)	has NOT
C.	(1)	is BLOCKED and requires MANUAL actuation
	(2)	has
D.	(1)	is BLOCKED and requires MANUAL actuation
	(2)	has NOT

85.

Given the following:

- Unit 1 has experienced a Steam Generator Tube Leak on the 1B S/G
- The 1A Main Steam Safety Valve is STUCK OPEN
- Safety Injection (SIAS) has ACTUATED
- 1-EOP-15, Functional Recovery, is in progress

Which ONE of the following completes the statement?

In accordance with 1-EOP-15, the crew is required to address the ____ (1) ____ Safety Function FIRST, in order to ISOLATE the ____ (2) ____.

A.	(1)	HR-2, S/G with SIAS
	(2)	1A S/G
B.	(1)	HR-2, S/G with SIAS
	(2)	1B S/G
C.	(1)	CI-1, Containment Isolation
	(2)	1A S/G
D.	(1)	CI-1, Containment Isolation
	(2)	1B S/G

86.

Given the following:

00:00	Unit 1 is at 100% power RCS Pressure is 2250 psia 1A1 Middle Seal Cavity Pressure is 2150 psig 1A1 Upper Seal Cavity Pressure is 1000 psig 1A1 Bleedoff Cavity Pressure is 110 psig 1-AOP-01.09A1, 2A1 Reactor Coolant Pump, is in progress
00:30	The Reactor is tripped due to RCP degrading conditions, in accordance with 1-AOP-01.09A1

Which ONE of the following completes the statements?

At time 00:00, the 2A1 RCP ____ (1) ____ is(are) failed.

In accordance with LI-AA-102-1001, Regulatory Reporting, a report to the NRC is required NO LATER THAN ____ (2) ____.

REFERENCE PROVIDED

A.	(1)	LOWER seal ONLY
	(2)	04:30
B.	(1)	LOWER seal ONLY
	(2)	08:30
C.	(1)	LOWER and MIDDLE seals
	(2)	04:30
D.	(1)	LOWER and MIDDLE seals
	(2)	08:30

87.

Given the following:

- Unit 1 is 100% power

Subsequently:

- Pressurizer pressure is LOWERING
- 1-AOP-01.10, Pressurizer Pressure and Level, is in progress
- Pressurizer Spray Valve position indications:

Spray Valve	Valve Position Indication
PCV-1100E	RED
PCV-1100F	GREEN

- The reactor automatically tripped due to the abnormal condition
- The Shift Manager determined the event to be reportable to the NRC

Which ONE of the following completes the statements?

In accordance with 1-AOP-01.10, the US will direct the RCO to place the ____ (1) ____.

In accordance with EPIP-08, Off-Site Notifications And Protective Action Recommendations, the report to the NRC will be made via the ____ (2) ____ phone.

A.	(1)	Pressurizer Spray Valve Selector Switch to 1100F
	(2)	Hot Ring Down (HRD)
B.	(1)	Pressurizer Spray Valve Selector Switch to 1100F
	(2)	Emergency Notification System (ENS)
C.	(1)	HIC-1100, PZR Pressure Spray Cntl Vlv, in MANUAL and adjust output
	(2)	Hot Ring Down (HRD)
D.	(1)	HIC-1100, PZR Pressure Spray Cntl Vlv, in MANUAL and adjust output
	(2)	Emergency Notification System (ENS)

88.

Given the following:

- Unit 1 is performing an RCS Cooldown per 1-GOP-305, Reactor Plant Cooldown - Hot Standby to Cold Shutdown
- The crew is preparing to secure the 1A1 Reactor Coolant Pump (RCP)

Subsequently:

- Prior to bypassing the RPS RCS Low Flow Trip, the Board RCO SECURES the 1A1 RCP
- Annunciator L-8, Reactor Coolant Flow Low Channel Trip is in ALARM

Which ONE of the following completes the statements?

In accordance with LI-AA-102-1001, Regulatory Reporting, this would be classified as a ____ (1) ____ Actuation.

In accordance with 1-ADM-09.26, Administrative Control of Valves, Locks and Switches, hanging of a Deviation tag on the RPS Zero Power Mode Bypass key switch ____ (2) ____ required.

A.	(1)	Valid
	(2)	is
B.	(1)	Valid
	(2)	is NOT
C.	(1)	Invalid
	(2)	is
D.	(1)	Invalid
	(2)	is NOT

89.

Given the following:

- Unit 1 is 100% power
- The previous shift noted the 1A S/G level swings between 65 – 67% were occurring

Subsequently:

- The BRCO reports the following occurred over the last 60 minutes:
 - o FCV-9011, 1A HPFRV, flow oscillations occurred every 8 minutes
 - o 1A S/G Level changed between 65-67% Narrow Range
 - o 1A S/G DCS Feedwater Flow perturbations from 6,670 to 6,770 klb/hr
- 1A S/G Level has returned to 65% and STABLE
- 1-AOP-09.01, Feedwater Control System Abnormal Operations, is in progress

Which ONE of the following completes the statements?

In accordance with 1-AOP-09.01 Attachment 1, Criteria to remove an Feed Regulating Valve from Service, ____ (1) ____ been MET.

On a Reactor and Turbine Trip, the 1A 100% Bypass ____ (2) ____ AUTOMATICALLY close.

REFERENCE PROVIDED

A.	(1)	has
	(2)	will
B.	(1)	has
	(2)	will NOT
C.	(1)	has NOT
	(2)	will
D.	(1)	has NOT
	(2)	will NOT

90.

Given the following:

- Unit 1 is 100% power
- The 1A Emergency Diesel Generator is INOPERABLE

Subsequently:

- A Loss of Offsite power occurs
- Annunciator A-6, 1B Emerg D/G Breaker Failure, is in ALARM
- Annunciator A-36, 1B Emerg D/G Local Alarm DC Pwr Fail, is in ALARM
- 1-EOP-10, Station Blackout is in progress

Which ONE of the following completes the statements?

In accordance with 1-EOP-10, the SNPO would be FIRST directed to perform 1-EOP-99, Appendices/Figures/Tables/Datasheets ____ (1) ____.

Reactor Coolant System Heat Removal would be performed by using the Atmospheric Dump Valves ____ (2) ____.

A.	(1)	Appendix C, Diesel Generator Local Start for the 1B EDG
	(2)	operated from the control room
B.	(1)	Appendix C, Diesel Generator Local Start for the 1B EDG
	(2)	local operation per Appendix U of 1-EOP-99
C.	(1)	Appendix F, Alternate Method of Cross-tying AC Power from Unit to Unit Using 2A4 or 2B4 Switchgear
	(2)	operated from the control room
D.	(1)	Appendix F, Alternate Method of Cross-tying AC Power from Unit to Unit Using 2A4 or 2B4 Switchgear
	(2)	local operation per Appendix U of 1-EOP-99

91.

Given the following:

- Unit 2 has experienced a LOCA
- 2-EOP-03, LOCA is in progress
- Hydrogen Purge is in progress in accordance with 2-EOP-99, Appendix N, Hydrogen Purge System Operation
- Annunciator X-1, Cont Cntmt/H₂ Purge Adsorber Temp High, is in ALARM
- H₂ Purge Flow Exhaust indicates 50 cfm
- Containment temperature is 120 °F

Which ONE of the following completes the statements?

In accordance with 2-EOP-99, Appendix N, the crew is required to ____ (1) ____.

____ (2) ____ is the cause for this alarm.

A.	(1)	secure the hydrogen purge
	(2)	Restricted air flow through the filter train
B.	(1)	secure the hydrogen purge
	(2)	High Containment temperature
C.	(1)	throttle open FCV-25-28, Continuous Containment / Hydrogen Purge Control Valve Bypass to raise flow
	(2)	Restricted air flow through the filter train
D.	(1)	throttle open FCV-25-28, Continuous Containment / Hydrogen Purge Control Valve Bypass to raise flow
	(2)	High Containment temperature

92.

Given the following:

- Unit 1 is 35% power and STABLE
- The 1B1 Circulating Water Pump is out of service
- Annunciator E-2, 1A1 Circ Water Pump Ovrld/Trip, is in ALARM
- Annunciator D-13, Turbine Vacuum Low, is in ALARM
- RCO reports the 1A1 Circ Water Pump has tripped
- Condenser Back Pressure rose to 7 inHgA
- 1-AOP-21.01, Circulating Water System, is in progress
- The crew commenced a rapid downpower per 1-AOP-22.01, Rapid Downpower

Subsequently:

- The BRCO reports Condenser Back Pressure is 6.2 inHgA and LP Inlet (OPC) Pressure is 80 psig

Which ONE of the following completes the statements?

In accordance with 1-AOP-21.01, the rapid downpower ____ (1) ____ be stopped. The bases for exiting the Restricted Operating Region is due to the impact on the working life of the ____ (2) ____ Pressure Turbine last blade row.

REFERENCE PROVIDED

A.	(1)	can
	(2)	Low
B.	(1)	can
	(2)	High
C.	(1)	can NOT
	(2)	Low
D.	(1)	can NOT
	(2)	High

93.

Given the following:

- Unit 1 is 100% power
- SNPO reports the in-service Gas Decay Tank (GDT) pressure has dropped from 80 psig to 59 psig in 36 hours
- 1-AOP-06.04, Uncontrolled Release of Radioactive Gas, is in progress
- A waste gas leak search has been initiated per 1-NOP-06.26, Waste Gas System Leakage Search

Subsequently:

- Eight (8) hours into the shift, the Waste Gas leak was located and ISOLATED in the Reactor Auxiliary Building

Which ONE of the following completes the statements?

In accordance with 1-AOP-06.04, the Control Room Ventilation ____ (1) ____ required to be placed in Recirc Mode per 1-AOP-25.02.

In accordance with C-200, Offsite Dose Calculation Manual, this ____ (2) ____ considered an UNPLANNED release.

A.	(1)	is
	(2)	is
B.	(1)	is
	(2)	is NOT
C.	(1)	is NOT
	(2)	is
D.	(1)	is NOT
	(2)	is NOT

94.

Given the following:

- Unit 2 is implementing 2-EOP-15, Functional Recovery
- The crew is performing Safety Function Status Checks

Which ONE of the following completes the statements?

The instrumentation that is to be selected FIRST to assess the status of Safety Functions is instrumentation ____ (1) ____.

In accordance with the Unit 2 UFSAR, Section 7.5.2.9, Post Accident Monitoring Instrumentation, the basis for utilizing this instrumentation is ____ (2) ____.

A.	(1)	identified by White Bezel around the face of the instrument
	(2)	qualified for harsh environment per Reg Guide (RG) 1.97
B.	(1)	identified by White Bezel around the face of the instrument
	(2)	these instruments are solely used for UFSAR Chapter 15, Accident Analysis
C.	(1)	utilized by Engineered Safety Features Actuation System (ESFAS) system
	(2)	qualified for harsh environment per Reg Guide (RG) 1.97
D.	(1)	utilized by Engineered Safety Features Actuation System (ESFAS) system
	(2)	these instruments are solely used for UFSAR Chapter 15, Accident Analysis

95.

Given the following:

- Unit 1 is performing a FULL Core Offload

Which ONE of the following completes the statements?

In accordance with 0-NOP-67.05, Refueling Operation, the Refueling Supervisor ____ (1) ____ direct the CORE OFFLOAD activities from the FUEL HANDLING BUILDING.

In accordance with Ops policy 202, Fuel Handling Supervisor Responsibilities, a Non-licensed Fuel Handling Supervisor ____ (2) ____ directly supervise fuel handling activities associated with movement of irradiated fuel assemblies in the FUEL HANDLING BUILDING.

A.	(1)	can
	(2)	can
B.	(1)	can
	(2)	can NOT
C.	(1)	can NOT
	(2)	can
D.	(1)	can NOT
	(2)	can NOT

96.

Given the following:

- 2 Departments, Electrical and I&C Maintenance are scheduled to work on the 1A1 Debris Filter System CONCURRENTLY utilizing a generic troubleshooting guide
- The work is expected to take 2 shifts
- They are requesting to use DIRECT CONTROL in accordance with OP-AA-101-1000, Clearance and Tagging
- The Electrical Disconnect switch is within line of sight of the work site
- BOTH the Electrical and I&C Maintenance Departments have verified the disconnect will completely isolate the energy source

Which ONE of the completes the statements?

In accordance with OP-AA-101-1000:

The ____ (1) ____ authorizes use of DIRECT CONTROL.

Regarding the evolution described above, DIRECT CONTROL ____ (2) ____ be authorized.

A.	(1)	Operations Manager
	(2)	can
B.	(1)	Operations Manager
	(2)	can NOT
C.	(1)	Operations Shift Supervision
	(2)	can
D.	(1)	Operations Shift Supervision
	(2)	can NOT

97.

Given the following:

- Unit 1 is 100% power
- 1A High Pressure Safety Injection Pump (HPSI) is INOPERABLE

Subsequently:

- At 0100 on 8/17/23, the 1B HPSI pump was declared INOPERABLE due to an oil leak
- Tech Spec 3.0.3 Limiting Condition for Operation was entered

Which ONE of the following completes the statement?

In accordance with Tech Spec 3.0.3, the LATEST Unit 1 is required to be in Cold Shutdown is by _____ on 8/18/23.

A.	0100
B.	0200
C.	1300
D.	1400

98.

Given the following:

- Unit 2 is 100% power
- “B” Main Steam Line (MSL) radiation has a loss of communication with the Radiation Monitoring Control System
- There is a suspected tube leak in the 2B S/G
- The RM-23P is being placed in service at the “B” MSL RM-80 Microprocessor in accordance with 2-NOP-26.01, Radiation Monitors

Which ONE of the following completes the statements?

The “B” MSL radiation levels can be read locally at the RM-23P ____ (1) ____ .

In accordance with ADM-11.16, Transient Procedure Use and Adherence, the “B” MSL Radiation Monitor is declared ____ (2) ____ .

A.	(1)	ONLY
	(2)	INOPERABLE
B.	(1)	ONLY
	(2)	OPERABLE but Degraded
C.	(1)	and RM-23 at PACB-2
	(2)	INOPERABLE
D.	(1)	and RM-23 at PACB-2
	(2)	OPERABLE but Degraded

99.

Given the following:

- An event requiring activation of the Emergency Response Organization (ERO) is in progress
- NO Radioactive releases have occurred

Which ONE of the following completes the statements?

The MINIMUM Emergency Action Level declaration that a Site Evacuation of non-essential personnel is mandatory is a(n) ____ (1) ____ .

Extra qualified NON-LICENSED Operators report to the ____ (2) ____ .

A.	(1)	ALERT
	(2)	affected Control Room
B.	(1)	ALERT
	(2)	Operations Support Center (OSC)
C.	(1)	Site Area Emergency
	(2)	affected Control Room
D.	(1)	Site Area Emergency
	(2)	Operations Support Center (OSC)

100.

Given the following:

- Unit 1 has declared a SITE AREA EMERGENCY
- The Technical Support Center (TSC) is OPERATIONAL
- The Emergency Offsite Facility (EOF) is OPERATIONAL

Which ONE of the following completes the statements?

In accordance with EPIP-02, Duties and Responsibilities of the Emergency Coordinator:

At this time, ____ (1) ____ CANNOT be delegated by the Emergency Coordinator.

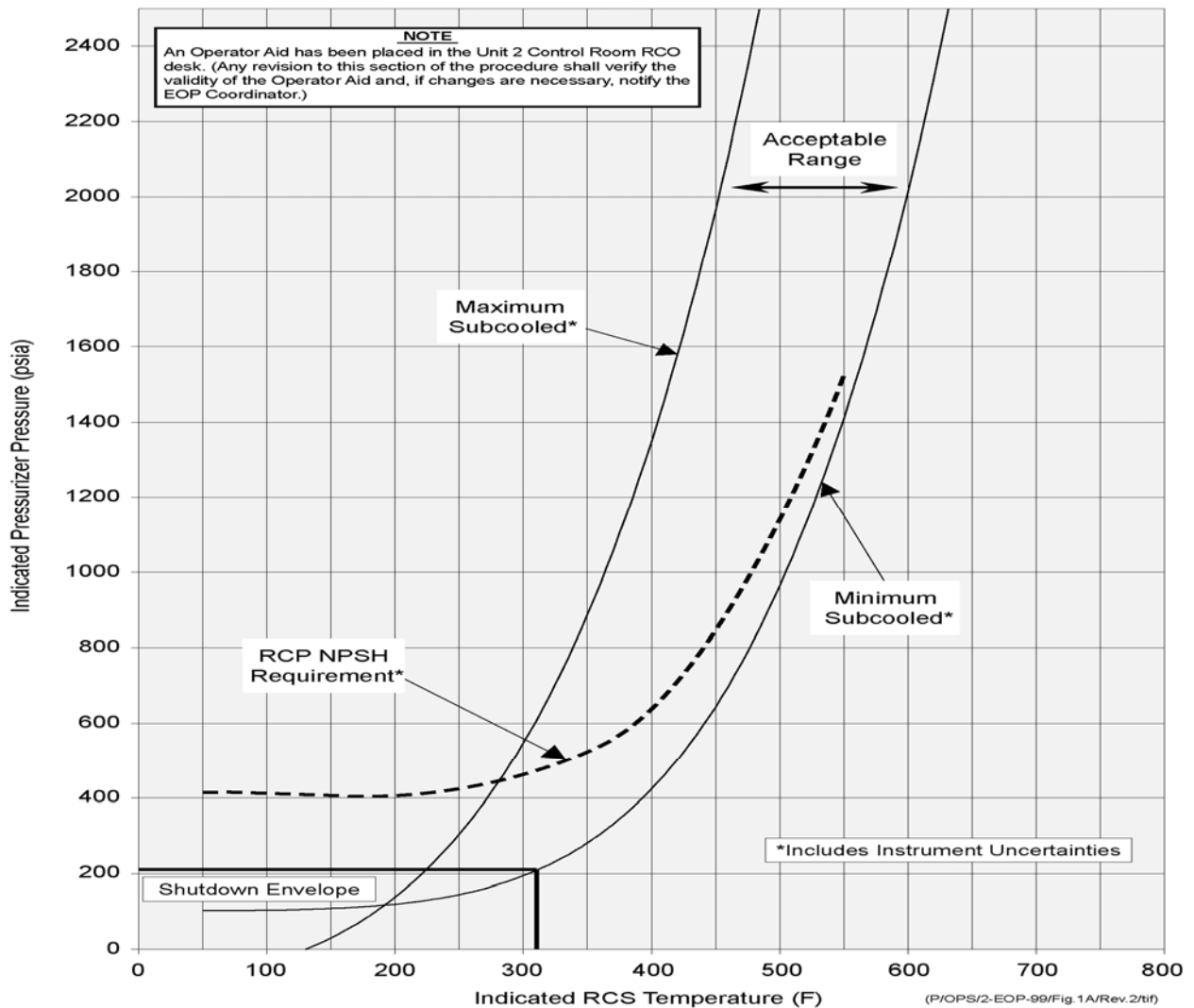
PRIOR to EC turnover to the TSC, if the Shift Manager becomes incapacitated during the event, the ____ (2) ____ is the NEXT designated position to assume the duties of the Emergency Coordinator.

A.	(1)	Emergency Classifications
	(2)	Unit 2 Unit Supervisor
B.	(1)	Emergency Classifications
	(2)	Shift Technical Advisor
C.	(1)	Protective Action Recommendations
	(2)	Unit 2 Unit Supervisor
D.	(1)	Protective Action Recommendations
	(2)	Shift Technical Advisor

FIGURE 1A
RCS Pressure Temperature
 (Containment Temperature Less Than or Equal to 200°F)
 (Page 1 of 1)

CAUTION

The RCP NPSH curve assumes one pump is operating in each loop. RCP instrumentation should be monitored for seal and pump performance per Table 13, RCP Operating Limits.



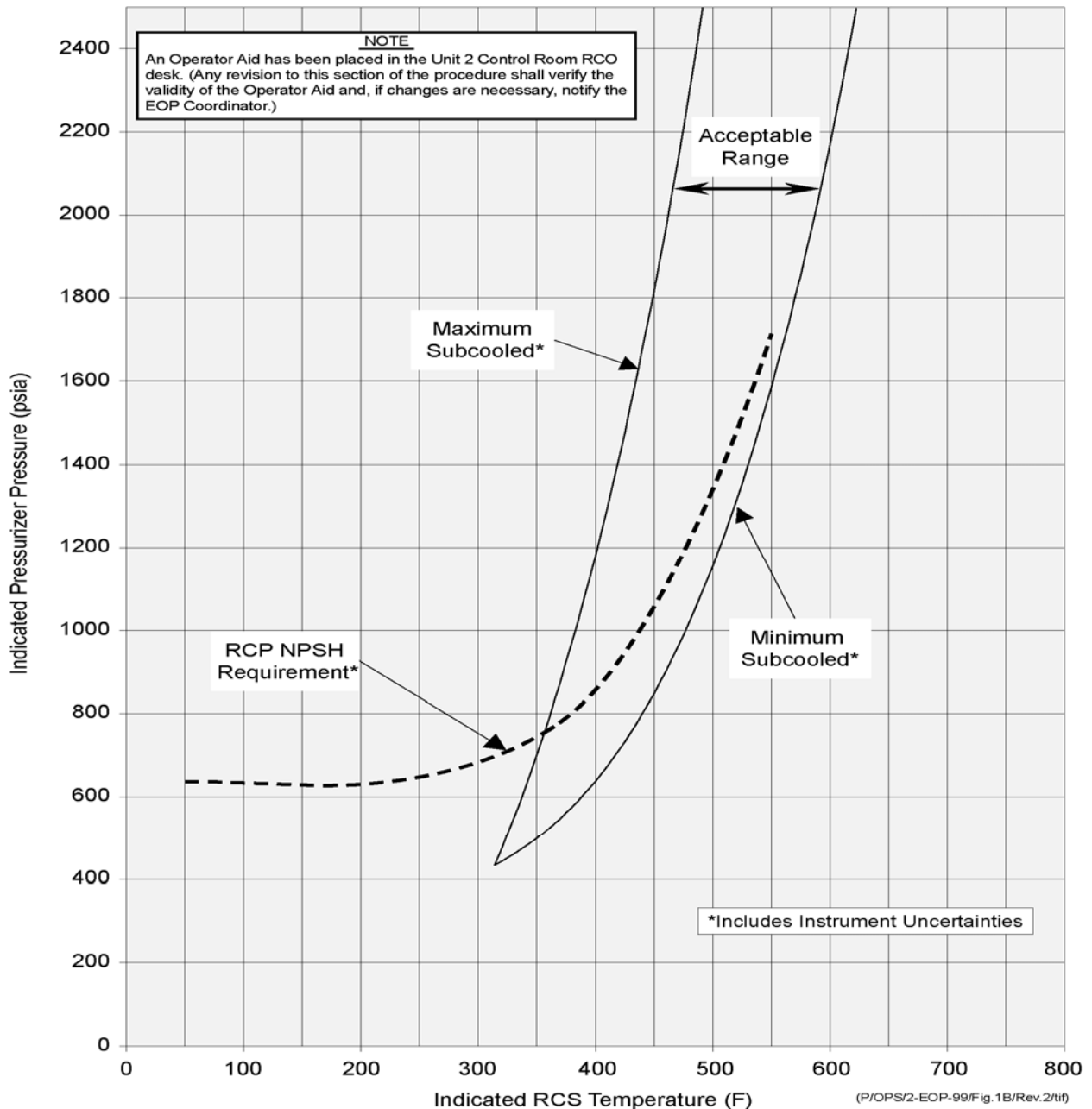
RCS Pressure Range	Required QSPDS Subcooled Margin Reading (Rep CET)
2250 psia to 1000 psia	40 to 180°F
1000 psia to 500 psia	50 to 170°F
Less than 500 psia	80 to 160°F

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FIGURE 1B
RCS Pressure Temperature
(Containment Temperature Greater Than 200°F)
 (Page 1 of 1)

CAUTION

The RCP NPSH curve assumes one pump is operating in each loop. RCP instrumentation should be monitored for seal and pump performance per Table 13, RCP Operating Limits.



REVISION NO.: 17	PROCEDURE TITLE: FEEDWATER CONTROL SYSTEM ABNORMAL OPERATIONS	PAGE: 21 of 53
PROCEDURE NO.: 1-AOP-09.01	ST. LUCIE UNIT 1	

ATTACHMENT 1
General Information
(Page 3 of 4)

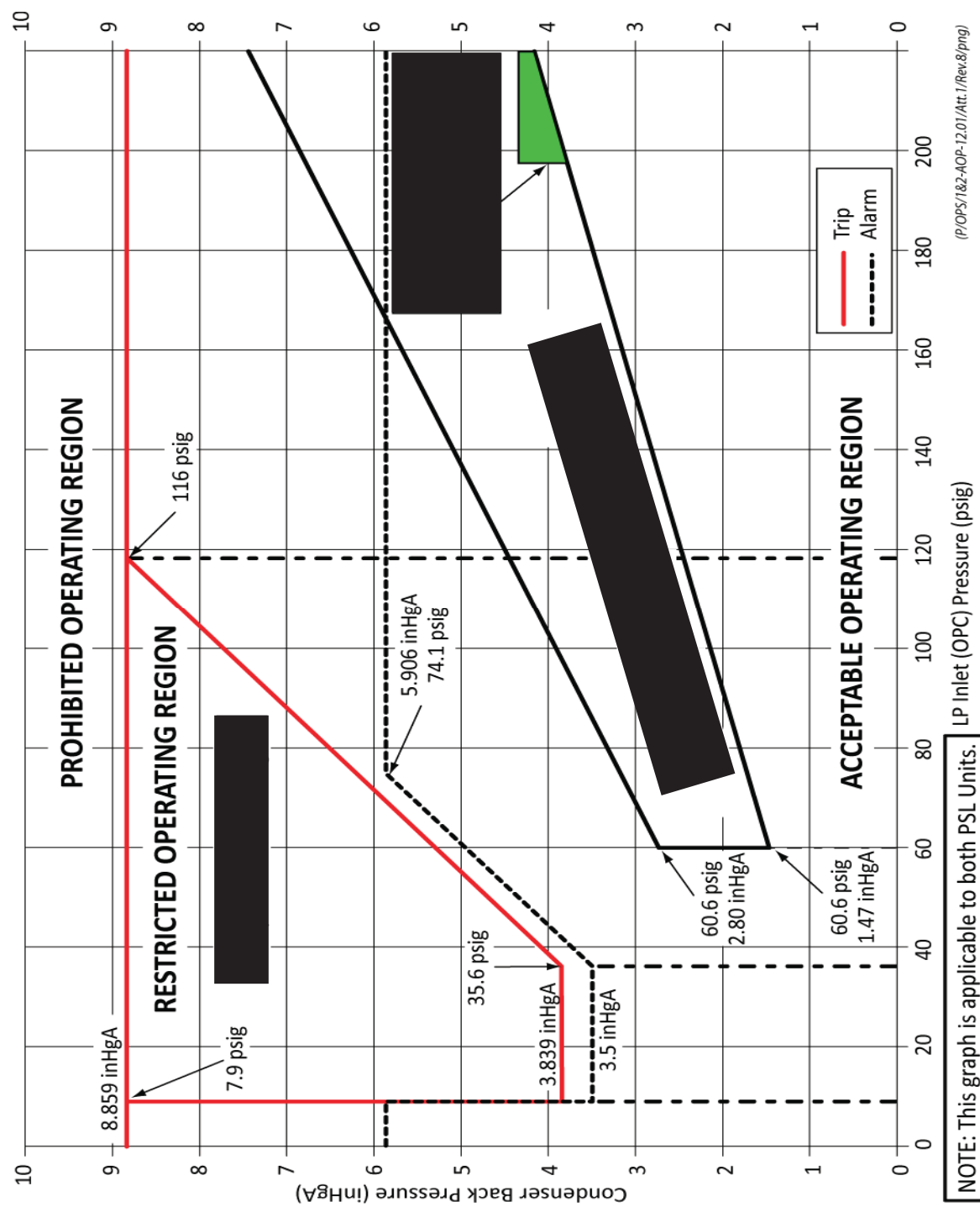
6.0 CRITERIA TO REMOVE A FEED REGULATING VALVE FROM SERVICE AS A RESULT OF DEGRADED PERFORMANCE

If any of the following conditions exist:

- Sustained, or degrading FRV performance indicated by S/G level swings greater than +/- 0.5% (>10 swings/hr exceeding criteria)
- DCS feedwater flow swings > +/- 1% (>30 swings/hr exceeding criteria).
- S/G level alarm **NOT** caused by plant transient.

then the associated FRV (FCV-9011/9021) shall be removed from service by transitioning S/G water level control to the 100% bypass valve and LCV-9005/9006 per Attachment 2, Transferring Level Control from HPFRV 1A to Bypass Valves or Attachment 3, Transferring Level Control from HPFRV 1B to Bypass Valves.

ATTACHMENT 6
Condenser Pressure Limitations
 (Page 1 of 3)



NOTE: This graph is applicable to both PSL Units.

(P/OPS/1&2-AOP-12.01/Att. 1/Rev.8.png)

PSL L-23-1 NRC WRITTEN EXAM ANSWER KEY

<u>Q</u>	<u>ANSWER</u>	<u>Q</u>	<u>ANSWER</u>	<u>Q</u>	<u>ANSWER</u>	<u>Q</u>	<u>ANSWER</u>
1	A	26	C	51	D	76	C
2	A	27	A	52	D	77	B
3	C	28	A	53	D	78	B
4	C	29	A	54	C	79	C
5	B	30	B	55	D	80	D
6	B	31	B	56	D	81	A
7	D	32	B	57	B	82	C
8	D	33	D	58	B	83	D
9	B	34	D	59	B	84	A
10	C	35	A	60	B	85	C
11	A	36	B	61	C	86	A
12	B	37	B	62	C	87	B
13	A	38	B	63	B	88	B
14	B	39	D	64	A	89	C
15	A	40	D	65	B	90	B
16	A	41	B	66	A	91	C
17	C	42	B	67	B	92	C
18	C	43	A	68	B	93	C
19	B	44	A	69	C	94	A
20	A	45	A	70	A	95	C
21	B	46	A	71	C	96	D
22	D	47	B	72	B	97	D
23	B	48	D	73	D	98	A
24	A	49	C	74	C	99	D
25	C	50	D	75	A	100	A