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2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10 of the Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.
- 3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION
- a. ISSUED TO (Name and Address)
 U.S. Department of Energy Division of Naval Reactors Washington, DC 20585

b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION Deactivated S5W Reactor Compartment Safety Analysis Report for packaging, dated July 1981, as supplemented.

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

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5.

(a) Packaging

- (1) Model Nos.: S5W Reactor Compartment and SSN 688 Class Reactor Compartment Disposal Package.
- (2) Description

The package consists of a deactivated and defueled S5W or SSN 688 Class reactor compartment which has been separated from the remainder of the submarine hull and prepared for shipment by sealing all openings and attaching handling fixtures. For each package model, the reactor compartment itself is between two containment bulkheads that are added to the package before shipping. The ship's hull and the containment bulkheads define the package containment boundaries. The containment bulkheads are either installed at the ends of the package or recessed. The strength of all package boundary closures is at least equivalent to the strength of the bulkheads. The deactivated reactor plant remains in place within the reactor compartment during shipment. The plant is defueled and drained except for small inaccessible pockets of liquid, primarily water. Potentially radioactively contaminated components and piping from other locations in the ship may be placed within the package and secured.

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5. (a) Packaging (Continued)

(2) Description (Continued)

The S5W Reactor Compartment package is between 35 and 45 feet long and approximately cylindrical with a maximum diameter of approximately 33 feet. The containment bulkheads are made of HS steel. The bulkheads may be installed at the ends of the package or may be recessed. The forward containment bulkhead may include existing ship structures which have been sealed to form a watertight bulkhead. The hull is constructed of HY-80 steel. The maximum weight of the S5W package is 2,160,000 pounds for the 598 and 585 classes and is 2,262,400 pounds for all other classes.

The SSN 688 Class Reactor Compartment package is approximately 46 feet long and approximately cylindrical with a maximum diameter of approximately 33 feet. The containment bulkheads are made of HS steel. The bulkheads may be installed at the ends of the package or may be recessed. The hull is constructed of HY-80 steel. The maximum weight of the package is 3,360,000 pounds.

(3) Drawings

The package is constructed in accordance with the drawings, figures, and sketches included in the application, as supplemented (see the "References" section of this certificate).

(b) Contents

Activated structural components associated with the S5W or SSN 688 Class reactor vessel complex, plant piping, ion exchanger resin, purification filter media (SSN 688 Class only), residual liquid and other miscellaneous components and materials contaminated with radioactive corrosion products (crud).

6. Residual liquids contained within plant systems must be removed prior to transport to the maximum extent practical, in accordance with established procedures, methods, and controls, as described in submittal dated April 5, 1996, or in the Safety Analysis Report for the individual submarine class reactor compartment packages.

Model Nos.	Maximum Volume of Residual Liquid (gallons)
S5W Reactor Compartment Disposal Package	≤ 660
SSN 688 Reactor Compartment Disposal Package	≤ 1,200

Table 1. Maximum Volume of Residual Liquid per Model No.

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7. For packages with recessed containment bulkheads, the aft containment bulkheads and stiffeners, horizontal divider plate, and any structure between the pressure hull and the outer non-pressure hull must be recessed at least 7 inches from the aft end of the S5W package. The forward containment bulkhead and stiffeners, existing tank stiffeners, deck structure, and horizontal girders must be recessed at least 15 inches from the forward end of the S5W package.

For SSN 688 Class packages with recessed containment bulkheads, both the aft and forward containment bulkheads, stiffeners and horizontal girders must be recessed at least 15 inches from the end of the package.

- 8. The Lowest Service Temperature (LST) must be determined for each package. The package shall not be shipped if the normal daily minimum temperature expected during the shipment of the package, as determined on the basis of weather forecasts, is less than the LST.
- 9. Ion exchanger resin with up to 3.1 curies (1.1 x 10¹¹ becquerels) of Co-60 may be shipped in the S5W package. Shipment of the S5W packages shall not occur before 180 days after final reactor shutdown.
- For SSN 688 Class packages, the Co-60 activity of the ion exchanger resin (A_{IX}) and activity of the 10. purification filter media (A_{PF}) must meet the following conditions (Reference Table 5-3 of the application):

* 14

when only the purification filter is solidified, (a)

 A_{PF} * 27.2 + A_{IX} * 283.8 ≤ 995 mrem/hr (Curies) A_{PF} * 7.35 x 10⁻¹⁰ + A_{IX} * 7.67 x 10⁻⁹ ≤ 995 mrem/hr (becquerels)

(b) when only the ion exchanger is solidified,

> $A_{IX} * 53.9 + A_{PF} * 299.6 \le 995$ mrem/hr (Curies) $A_{IX} * 1.46 \times 10^{-9} + A_{PF} * 8.10 \times 10^{-9} \le 995$ mrem/hr (becquerels) Nn ***

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- 10. For SSN 688 Class packages, the Co-60 activity of the ion exchanger resin (A_{IX}) and activity of the purification filter media (A_{PF}) must meet the following conditions (reference table 5-3 of the application) (continue):
 - (c) when both the purification filter and ion exchanger are solidified;

 $A_{IX} * 53.9 + A_{PF} * 27.2 \le 995$ mrem/hr (Curies) $A_{PF} * 1.46 \times 10^{-9} + A_{IX} * 7.35 \times 10^{-10} \le 995$ mrem/hr (becquerels)

(d) when neither the ion exchanger or purification filter are solidified;

 $A_{PF} \le 3.3$ curies of Co-60 (1.22 x 10¹¹ becquerels), $A_{IX} \le 3.5$ curies of Co-60 (1.30 x 10¹¹ becquerels), and $A_{IX} + A_{PF} \le 4.5$ curies of Co-60 (1.67 x 10¹¹ becquerels)

If the activity exceeds any of the above values, supplemental shielding must be added. See condition Nos. 12 and 13 for the supplemental shielding requirements.

These activity limits are based on limiting the total radiation levels to 995 mrem/hr under hypothetical accident conditions. If contaminated filters are shipped with the package, the sum of the radiation levels from each filter at 1 meter, shielded by the hull, must be subtracted from this 995 mrem/hr, and the A_{PF} and A_{IX} activity limits must be reduced accordingly for SSN 688 Class packages.

- 11. The following conditions apply to shipments of the SSN 688 Class packages:
 - (a) shipments shall not occur before 365 days after final reactor shutdown; and
 - (b) submarines SSN 701 and SSN 711 reactor compartments are not authorized for shipment in the SSN 688 Class packages.
- 12. Prior to shipment, radiation surveys of the exterior of the unshielded package must be taken. Additional shielding must be provided on the exterior of the package by steel plates securely welded to the package surface so as to remain in place under the hypothetical accident conditions in 10 CFR Part 71 if either of the following conditions exists:
 - (a) Radiation levels on the exterior of the package obtained via surveys exceed 200 mrem/hr on-contact or 10 mrem/hr at 2 meters.
 - (b) When the purification filter and ion exchanger are not solidified, and $A_{PF} > 3.3$ curies of Co-60, $A_{IX} > 3.5$ curies of Co-60, or $A_{IX} + A_{PF} > 4.5$ curies of Co-60. (SSN 688 Class only)

Radiation surveys must be re-performed after adding supplemental shielding. Final radiation levels must not exceed 200 mrem/hr on-contact and 10 mrem/hr at 2 meters.

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13. When condition 12(a) exists, 0.5-inch thick steel plates must extend from one inch above the bottom of the shielded tunnel on the starboard side to 87° beyond the keel on the port side and extend 110 inches forward and aft of the centerline of the Pressure Vessel. Additional 1.25" steel plate must extend 22° beyond the keel on the port and starboard side and 77 inches forward and aft of the centerline of the Pressure Vessel.

When condition 12(b) exists, 1-inch thick steel plates must either:

- (a) Extend from one inch above the bottom of the shielded tunnel on the starboard side to 64° beyond the keel on the port side and extend from frame 73 to frame 76. In this case, A_{PF} cannot exceed 7.8 curies of Co-60 (2.89 x 10¹¹ becquerels), A_{IX} cannot exceed 8.2 curies of Co-60 (3.03 x 10¹¹ becquerels), and the total of A_{PF} and A_{IX} cannot exceed 8.5 curies of Co-60 for D1G-2 cores or 9.0 curies of Co-60 for D2W cores (3.14 x 10¹¹ and 3.33 x 10¹¹ becquerels, respectively); or
- (b) Extend from one inch above the bottom of the shielded tunnel on the starboard side to 37° beyond the keel on the port side and extend from frame 73 to frame 76. In this case, A_{PF} cannot exceed 7.8 curies of Co-60 (2.89 x 10¹¹ becquerels), A_{IX} cannot exceed 3.5 curies of Co-60 (1.30 x 10¹¹ becquerels), and the total of A_{PF} and A_{IX} cannot exceed 8.5 curies of Co-60 for D1G-2 cores or 9.0 curies of Co-60 for D2W cores (3.14 x 10¹¹ and 3.33 x 10¹¹ becquerels, respectively).

When both conditions 12(a) and 12(b) exist, 1-inch thick steel plates must extend from one inch above the bottom of the shielded tunnel on the starboard side to 87° beyond the keel on the port side and extend 110 inches forward and aft of the centerline of the Pressure Vessel. Additional 0.75 inch steel plates must extend 22° beyond the keel on the port and starboard side and extend 77 inches forward and aft of the centerline of the Pressure Vessel. Additional 0.75 inch steel plates must extend 22° beyond the keel on the port and starboard side and extend 77 inches forward and aft of the centerline of the Pressure Vessel. In this case, A_{PF} cannot exceed 7.8 curies of Co-60 (2.89 x 10¹¹ becquerels), A_{IX} cannot exceed 8.2 curies of Co-60 (3.03 x 10¹¹ becquerels), and the total of A_{PF} and A_{IX} cannot exceed 8.5 curies of Co-60 for D1G-2 cores or 9.0 curies of Co-60 for D2W cores (3.14 x 10¹¹ and 3.33 x 10¹¹ becquerels, respectively).

The activity limits for the ion exchanger and purification filter are based on limiting the total radiation levels to 995 mrem/hr under hypothetical accident conditions. If contaminated filters are shipped with the package, the sum of the radiation levels from each filter at 1 meter, shielded by the hull, must be subtracted from this 995 mrem/hr, and the A_{PF} and A_{IX} activity limits must be reduced accordingly for SSN 688 Class packages.

- 14. Ensure the radiation surveys are completed as described in enclosures 3, 4, and 5 of the supplement dated December 2, 2011.
- 15. In addition to the requirements of subpart G of 10 CFR Part 71:
 - (a) Each package must be prepared for shipment and operated in accordance with the procedures described in chapter 7.0, "Package Operations," of the application, as supplemented.
 - (b) Each package must be tested and maintained in accordance with the procedures described in chapter 8.0, "Acceptance Tests and Maintenance Program," of the application, as supplemented.

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- 16. The hydrogen concentration within the package must be less than 5 percent by volume during the shipment period, as demonstrated by test or analysis.
- 17. Expiration date: September 30, 2028.

REFERENCES

Deactivated S5W Reactor Compartment Safety Analysis Report for Packaging, WAPD-REO(C)-250, dated July 1981.

Supplements: Naval Reactors Memoranda Nos. Z#C90-14416 dated March 29, 1990, and supplement dated July 6, 1990; Z#C90-14456 dated August 30, 1990; Z#C92-14438 dated August 3, 1992; Z#C93-00069 dated October 14, 1993; Z#C95-00113 dated March 16, 1995; Z#96-14430 dated April 5, 1996; Z#96-14434 dated April 10, 1996; Z#C95-00191 dated December 14, 1995; Z#96-14457 dated June 20, 1996; Z#C96-14520 dated November 22, 1996; Z#C96-14549 dated December 19, 1996; Z#C97-14698 dated October 31, 1997; Z#C98-00021 dated February 27, 1998; Z#C02-03057 dated March 15, 2002; Z#C07-02023 dated September 19, 2007; Z#08-03540 dated September 11, 2008; Z#C07-04862 dated January 3, 2008; Z#C09-02922 dated July 1, 2009; and NR (Angerhofer) email dated July 30, 2008; S#C11-01953 dated April 29, 2011; S#C11-02849 dated June 30, 2011; S#C11-04480 dated October 27, 2011; S#C11-05155 dated December 2, 2011; Z#13-02768 dated June 21, 2013; Z#18-01456 dated April 17, 2018; and Z#C18-02148 dated June 4, 2018; Z#C19-03241 dated August 5, 2019; Z#C20-00660 dated January 30, 2020; and Email dated March 19, 2020 from M. Kuprenas (Agencywide Documents Access and Management System Accession No. ML20126F329); and Z#23-01853 dated May 8, 2023.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Yon Y. Conduine Signed by Diaz-Sanabria, Yoira on 07/26/23

Yoira Diaz-Sanabria, Chief Storage and Transportation Licensing Branch Division of Fuel Management Office of Nuclear Material Safety and Safeguards

Date: July 26, 2023