MILLSTONE POWER STATION
UNIT 2
FINAL SAFETY ANALYSIS REPORT

Revision 41—06/29/23

Revision Summary

Revision 41—06/29/23

Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
41	06/29/23	MP2-UCR-2022-004	1.2.5, 1.2.12.d, 5.E, 11.1.3.1, 14.6.5.2.5.3, 14.6.5.2.5.4, Figures 8.1-1D and 14.6.5.2-15	Correct identified MPS 2 FSAR errors CR1194532, MP2-UCR-2015-020, MP2-UCR-2016-013, MP2-UCR-2016-016, MP2-UCR-2018-010, MP2-UCR-2018-015
41	06/292/23	MP2-UCR-2019-010	7.5.6.2.2.2.3, 7.5.6.2.2.2.4	[MP2 PIOPS Upgrade MP2-19-01091]
41	06/29/23	MP2-UCR-2023-002	3,5.2, 3.5.5, 14.0.7, 14.0.13, 14.1.5, 14.1.6, 14.3.3, 14.4.3, 14.4.8, 14.4.10, Table 14.1.5.2-6, Figure 1.0.7-(1,5,6)	Millstone Unit 2 Cycle 29 Updates to Millstone Unit 2 Final Safety Analysis Report EVAL-ENG-RSE-M2C29, Revision 0
41	06/29/23	MP2-UCR-2020-005	7.2.3.2.2	New Trip Units Applicable to All RPS Channels MP2-15-01097 Rev 7
41	06/29/23	MP2-UCR-2022-001	8.5.22, 8.5.4.1	MP2 Battery Charger Replacement (201B/DC2) MP2-21-01158
40.01	09/29/22	MP2-UCR-2022-003	2.5.4.2.1.b.1.B	Statement on PMW as Backup to AFW Discrepancy ETE-MP-2021-1131, Revision 1 CR1198357

Revision 40—06/30/22

Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
39.03	03/31/22	MP2-UCR-2021-007	2.5.4.2.1.A.12, 2.5.4.2.1.B.2.c, 2.5.4.2.1.B.3.c	Service Water Pump Motor Restoration Flood Water Level CR1171836
39.03	03/31/22	MP2-UCR-2022-002	Table 14.6.3-4, Table 14.6.3-5	Corrects transcription errors identified by the NRC CR1191354
39.02	01/27/22	MP2-UCR-2016-017	Tables 6.2-1, 6.4-1, 9.4-1, 9.5-2, and 11.1-1	Change to allow ANSI class ratings other than 600 or 800 to be used in various systems. CA 3033065
39.02	01/27/22	MP2-UCR-2018-013	7.5.7	MP2 Loose Parts Monitoring System Replacement MP2-17-00158-000
39.02	01/27/22	MP2-UCR-2020-007	Figure 8.1-1D	345kV Switch 15G-10T-4 Replacement MP2-19-01182
39.02	01/27/22	MP2-UCR-2020-009	8.5.22, 8.5.4.1	MP2 Battery Charger Replacement (201A/DC1) MP2-19-01174
39.02	01/27/22	MP2-UCR-2021-005	3.3.4, Table 14.1.5.2-6, Figures 14.0.7-1, 14.0.7-5, 14.0.7-6	Millstone Unit 2 Cycle 28 Updates to Millstone Unit 2 Final Safety Analysis Report EVAL-ENG-RSE-M2C28, Revision 0
39.02	01/27/22	MP2-UCR-2021-006	Table 5.2-11	MP2 10CFR50 Appendix J Scope Reduction Shutdown Cooling Penetration 10 ETE-MP-2021-1088
39.01	09/30/21	MP2-UCR-2018-017	7.8	FSAR change to update Communications System for replacement of Trunked Radio System. MPG-15-01007, CR499765

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
39.01	09/30/21	MP2-UCR-2021-002	Table 7.5-3	Update MP2 FSAR Accident Monitoring Instrument Table 7.5-3 ETE-MP-2020-1068

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
39	03/31/21	MP2-UCR-2020-011	7.11, 12.10	Addition of 50.69 Program P3 Technical Specification Amendment 337 Risk-Informed Categorization and Treatment of Structure, Systems, and Components of Nuclear Power Reactors
39	03/31/21	MP2-UCR-2021-001	14.6.1.4, 14.6.1.15, 14.6.1.6, 14.6.6, 14.6.7, Tables 14.6-1 and 14.6.1-2	Removal of Inadvertent Opening of a Power Operated Relief Valve in Mode 2 from the Millstone Unit 2 FSAR ETE-NAF-2020-0118
39	03/31/21	MP2-UCR-2021-003	1.10, 12.8	Incorporating Quality Assurace Program Description by Reference CA8102163
38.02	03/31/21	MP2-UCR-2016-011	8.1.2.2.c	MP2 FSAR Update To Document The Addition Of The Open Phase Detection Relay Schemes On The High Voltage Side Of The Generator Step-up And Rsst Transformers. MP2-15-01026, MP2-17-00156

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
38.02	03/31/21	MP2-UCR-2016-013	1.2.12.d, 11.1.3.1, 11.1.3.2, 11.1.5.1, Table 11.1-1, Figures 1.2-9, 9.12-1, 10.3-1, 11.1-4, 11.1-3, 11.2-5	Update the affected sections due to the removal of demineralizers T141A, T141B, T141C, and the installation of the Energy Solutions ALPS/AIM and SEDS systems. MP2-14-01169, MP2-15-01044, DC MP2-15-01045
38.02	03/31/21	MP2-UCR-2016-014	Table 11.1-1	Update Liquid and Aerated Waste Valve information to allow other stainless steel material grads and other ANSI ratings. MP2-14-01169
38.02	03/31/21	MP2-UCR-2018-015	Appendix 5.E.VI	Add NUPIPE II computer code to UFSAR Appendix 5E.VI MP2-17-00178
38.02	03/31/21	MP2-UCR-2019-001	14.1.5.3, 14.6.3.6.2, Table 14.1.5.3-1 , 14.1.5.3-2, 14.6.3-4 through 14.6.3-6	Updated Main Steam Line Break and Steam Generator Tube Rupture Dose Consequences. ETE-NAF-2019-0010, CR 1094790, CR 1103271
38.02	03/31/21	MP2-UCR-2019-007	9.8.2.2.3.5	Permanently Bypass RFM Aux. Hoist Load Cell Interlock, M2H12. MP2-19-01136
38.02	03/31/21	MP2-UCR-2020-001	Figure 8.1-1D	345kV Switch 15G-18T-8 Replacement. MP3-18-01094
38.02	03/31/21	MP2-UCR-2020-008	5.2.8.4.1, 5.2.8.4.2	Revision of "MP2 Tendon Surveillance Program Description". LBDCR 19-MP2-009

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
38.02	03/31/21	MP2-UCR-2020-010	7.8.3	Replacing ENRS (ARCOS) with DEENS (Everbridge) MP-20-04-00 - EP-AA-101-Attachment 2, 50.54(q), for change from Emergency Notification & Response System (ENRS) to Dominion Energy Emergency Notification System (DEENS)
38.01	09/30/20	MP2-UCR-2015-010	8.6.2.2, 5.6.4.1	Reflects replacement of inverters (INV 2, INV4, INV6). MP2-14-01158
38.01	09/30/20	MP2-UCR-2015-020	Figure 8.1-1D	Reflects revised discussion regarding piping weld examination method(s). LBDCR 14-MP2-019
38.01	09/30/20	MP2-UCR-2016-005	7.2.3.2.2	Delete the word "reed" in the first paragraph in MP2 FSAR section 7.2.3.2.2. MP2-15-01097
38.01	09/30/20	MP2-UCR-2020-004	6.7.2	Update Unit 2 FSAR to reflect changes for License Amendment No. 338. License Amendment No. 338

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
38	06/30/20	MP2-UCR-2019-030	8.3	Increasing the TRM required Diesel Fuel Oil Storage for T-148. ETE-MP-2020-1002

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
38	06/30/20	MP2-UCR-2020-002	3.3.1.1.2, 3.3.4, 3.5.2.2.1, 3.5.5, Figure 14.0.7-1, Figure 14.0.7-5, Figure 14.0.7-6, Table 14.1.5.2-6	Millstone Unit 2 Cycle 27 Updates to Millstone Unit 2 Final Safety Analysis Report. EVAL-ENG-RSE-M2C27
38	06/30/20	MP2-UCR-2020-003	5.2.6.1.1, 8.7.1.2, 8.7.2.2	Update the MP2 FSAR to reflect IEEE Standard 317-1976 as applicable to replacement medium voltage electrical containment penetration modules. MP2-19-01025
37.03	03/31/20	MP2-UCR-2019-006	9.2.4.1	Interm Retirement of MP2 Boric Acid Heat Tracing Circuits. ETE-MP-2019-1081
37.03	03/31/20	MP2-UCR-2018-010	1.2.5, 1.4.3.1, 4.3.2, Table 4.3-2, Table 10.2-1	Updating The FSAR to Accurately Reflect The New Unit 2 Main Steam Moisture Content. MP2-19-01066
37.03	03/31/20	MP2-UCR-2019-011	8.5.1.2	Changing of the Unit 2 Vital Battery Discharge Testing and Battery Charger Load Testing frequencies. ETE-MP-2019-1055
37.02	12/30/19	MP2-UCR-2017-017	Figure 14.0.7-5	Departure from Nucleate Boiling Limiting Condition for Operation Verification Results. ETE-NAF-2017-0169

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37.02	12/3019	MP2-UCR-2019-005	4.3.8.1.1, 4.3.8.3.1, 7.5.1.2.1.1.a, 7.5.1.2.1.1.c, 7.5.1.2.2, 7.5.5.1, 7.5.5.3, 7.5.5.4, 7.5.6.2.2.2.6, 7.5.6.3.2.1.8, 7.5.6.3.2.2.7, 7.6.2.2.2	Trend Recorder Description. MP2-17-00101, DCR M2-96064, M2-05003
37.01	09/30/19	MP2-UCR-2017-010	2.5.4.2.1	AOP 2560 - Actions Due to Intake Water Level Elevation Approaching 16'-6"
37.01	09/3019	MP2-UCR-2019-002	7.3.4.2.3	SAR section 7.3.4.2.3 not consistent with Specification 7604-M-480, Specification For Engineered Safety Actuation Systems for The Millstone Point Company Millstone Nuclear Power Station Unit 2 Waterford Connecticut. STI-M2-2018-001

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
36.2	12/31/18	MP2-UCR-2018-001	8.6.2.2	Unit 2 inverter replacement. MP2-14-01130
36.2	12/31/18	MP2-UCR-2018-005	7.4.7.2.2	Millstone Unit 2 Main Control Room Recorders Replacement on panel C-05. MP2-17-00171

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
36.2	12/31/18	MP2-UCR-2018-011	8.1.2.2	Generator Backup Audio-Tone Equipment Replacement. MP2-18-00102
36.2	12/31/18	MP2-UCR-2018-012	Table 11.1-1	Correction of Volume of Waste Gas Surge Tank in FSAR Table 11.1.-1. CR1078780, CR1091106
36.2	12/31/18	MP2-UCR-2018-014	14.0.7.2, 14.0.7-5, Figure 14.0.7-6, Table 14.1.5.2-6	Millstone Unit 2 Cycle 26 Updates to Millstone Unit 2 Final Safety Analysis Report. EVAL-ENG-RESE-M2C26
36.2	12/31/18	MP2-UCR-2018-016	Table 14.6.3-5	Specification of the Basis for the Steam Generator Partition Factor for Particulates. ETE-NAF-2018-0099
36.1	09/27/18	MP2-UCR-2015-017	8.4.2.1, 8.5.1.1, 8.5.2.1, 8.5.2.2, 8.5.3.1, 8.5.3.2	Reflects removal of 125 VDC lighting loads from Vital Station Batteries & installation of LED lighting. MP2-14-01185

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
36.1	09/27/18	MP2-UCR-2018-007	14.1.5.2.5.1.4, 14.1.5.2.5.1.6, 14.1.5.2.6.2.1, 14.1.5.2.6.2.3, Table 14.1.5.2-5, Table 14.1.5.2-6, Table 14.1.5.2-8, Figure 14.1.5.2.-9, Figure 14.1.5.2.-10, Figure 14.1.5.2.-11, Figure 14.1.5.2.-12, Figure 14.1.5.2.-13, Figure 14.1.5.2.-14, Figure 14.1.5.2.-15, Figure 14.1.5.2.-16	MPS2 FSAR Update for Post-Scram Main Steam Line Break with Decreased MTC and Revised AFW Flow Distribution. ETE-NAF-2018-0019
36.1	09/27/18	MP2-UCR-2018-009	14.6.5.2, Table 14.6.5.2-7	MPS3 FSAR Update to the Small Break Loss of Coolant Accident (SBLOCA) Results Due to Error Correction. ETE-NAF-2018-0060

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36	06/28/18	MP2-UCR-2017-005	10.4.5.5	Update FSAR Section 10.4.5.5 to read as follows: "The plant can carry 55 percent load with one-half of the feedwater heaters out of service and isolated from the system, with only one condensate pump in operation and with only one SGFP in service". LA3022214
36	06/28/18	MP2-UCR-2018-002	Table 1.3-1, Table 11.1-1	Correction of Volume of Waste Gas Test Tank in FSAR table 11.1-1 CR1078780 and CR1080672
36	06/28/18	MP2-UCR-2018-006	Table 4.6-2	Update to MPS2 Pressurizer RTNDT values determined through Drop Weight Testing. AREVA CR 2017-2164
35.3	03/29/18	MP2-UCR-2017-017	Table 14.0.7-5, Figure 14.0.7-5	Departure from Nucleate Boiling Limiting Condition for Operation Verification Results. ETE: ETE-NAF-2017-0169 CA3060853
35.3	03/29/18	MP2-UCR-2018-003	Table 9.11-2	Change Pressure Values on Table 9.11-2 for Instrument Air Receiver Tanks T-51E and T-51F. DC: MP2-17-00175
35.2	12/28/17	MP2-UCR-2017-013	9.2.3.3, 11.1.3.1	Modify U3 FSAR to correct item related to previous degasifier related FSAR change. FSARCR: 98-MP2-167
35.2	12/28/17	MP2-UCR-2017-014	9.2.2.1	Modify U2 FSAR to correct item related to VCT level controls related to a previous FSAR change. FSARCR: 97-MP2-66

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35.1	09/28/17	MP2-UCR-2016-001	1.A, 8.1.2.2, 8.1.2.3	Eversource 15G Breaker Failure Relay Replacement. ETE-MP-2015-1186
35.1	09/28/17	MP2-UCR-2016-018	8.1.1.2, 8.1.2.1, 8.1.2.2, Fig 8.1-1B, Fig 8.1-1C, Fig 8.1-1D	Eversource Energy proposes to reconfigure the 345 kV Millstone - Haddam - Besesk 348 line. ETE-MP-2016-1135
35.1	09/28/17	MP2-UCR-2016-019	7.6.5.4.1	App R fire panel door lock removal. CR1045068
35.1	09/28/17	MP2-UCR-2016-020	7.5.5.3	Plant Process Computer Data Diode Installation. MP2-16-01035
35.1	09/28/17	MP2-UCR-2017-001	Figure 6.5-1, Table 6.5-2	Replacement of “C” CAR Fan M2F14C. MP2-17-01004

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35.1	09/28/17	MP2-UCR-2017-002	1.4.2, 3.1, 3.2.3, 3.3.1, 3.3.1.1, 3.3.1.1.1, 3.3.1.1.2, 3.3.1.3.1, 3.3.1.3.2, 3.3.4 Refs, Table 3.3-1, Figure 3.3-1A, Figure 3.3-1B, 3.4.2, 3.4.3.2.3, 3.5.1.1, 3.5.2.1.2, 3.5.2.1.3, 3.5.3.3.1, 3.5.3.3.4, Table 3.5-1, 9.5.2.1, 9.5.3.1, 9.5.4.1, 14.0.2, 14.0.7.1, 14.0.10, Table 14.0.2-2, Table 14.0.7-4, Table 14.0.7-5, Figure 14.0.7-1, Figure 14.0.7-2, Figure 14.0.7-5, Figure 14.0.7-6, 14.1.5.1.6.1, 14.1.5.1.6.1.3, 14.1.5.1.7, 14.1.5.2.5.1.4, 14.1.5.2.5.1.6, 14.1.5.2.6.1.1, 14.1.5.2.6.1.2, 14.1.5.2.6.1.3, Table 14.1.5.1-5, Table 14.1.5.1-6, able 14.1.5.1-7, Figure 14.1.5.1-1, Figure 14.1.5.1-2	This SAR Change Request updates the Millstone Unit 2 FSAR for the Cycle 25 core design and supporting safety analyses. ETE-NAF-2017-0001, EVAL-ENG-RSE-M2C25

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35.1	09/28/17	MP2-UCR-2017-002 (cont)	Figure 14.1.5.1-3, Figure 14.1.5.1-4, Figure 14.1.5.1-5, Figure 14.1.5.1-6, Table 14.1.5.2-4, Table 14.1.5.2-5, Table 14.1.5.2-6, Table 14.1.5.2-7, Figure 14.1.5.2-2, Figure 14.1.5.2-3, Figure 14.1.5.2-4, Figure 14.1.5.2-5, Figure 14.1.5.2-6, Figure 14.1.5.2-7, Figure 14.1.5.2-8, Figure 14.1.5.2-9, Table 14.2.4-3, Figure 14.2.4-1, Figure 14.2.4-2, Figure 14.2.4-3, Figure 14.2.4-4, Figure 14.2.4-5, 14.4.6.5, 14.4.6.6, 14.4.10 Refs, Table 14.4.6-2, Table 14.4.6-3, Table 14.4.6-4, 14.6.5, 14.6.5.1 through 14.6.5.8, 14.6.5.4 through 14.6.5.4.8 (new)	This SAR Change Request updates the Millstone Unit 2 FSAR for the Cycle 25 core design and supporting safety analyses. ETE-NAF-2017-0001, EVAL-ENG-RSE-M2C25

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Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
35.1	09/28/17	MP2-UCR-2017-002 (cont)	14.6.6 Refs, Table 14.6.5.1-3, Table 14.6.5.1-4, Table 14.6.5.1-5, Table 14.6.5.1-6, Table 14.6.5.1-7, Table 14.6.5.1-8, Figures 14.6.5.1-1 through Figure 14.6.5.1-17, Tables 14.6.5.4-1 through Table 14.6.5.4-7, Figures 14.6.5.4-1 through Figures 14.6.5.4-19	This SAR Change Request updates the Millstone Unit 2 FSAR for the Cycle 25 core design and supporting safety analyses. ETE-NAF-2017-0001, EVAL-ENG-RSE-M2C25
35.1	09/28/17	MP2-UCR-2017-006	4.3.8.5.11	Update to reflect that during normal operation, strainers for the B and C RCPs are operated with their strainer baskets removed and the A and D RCPs do not have strainers. MP2-17-01033
35.1	09/28/17	MP2-UCR-2017-007	7.3.1.2.8	Engineered Safety Feature Actuation UFSAR Response Time Change. ETE-MP-2015-1030
35.1	09/28/17	MP2-UCR-2017-008	4.3.2, 4.3.5	MP2 Plant Process Computer Replacement additional FSAR changes missed during the initial project. MP2-13-01067
35.1	09/28/17	MP2-UCR-2017-009	8.3.3.1	Implementation of Revised Combustible Gas (Hydrogen) Control Requirements in Containment additional FSAR changes missed during the initial project. M2-04009

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35.1	09/28/17	MP2-UCR-2017-011	9.10.2.1	Editorial correction to MP2 FSAR. Change last 2 sentences of 4th paragraph of 9.10.2.1 on page 9.10-2 "Pump Running and trouble signals for pump P-82 are available in the Unit 2" (Should be Unit 3) Control Room. DM3-01-0064-00
35.1	09/28/17	MP2-UCR-2017-012	14.8.2.1.2	Modify FSAR sections 10.4.5.5 and 14.8.2.1.2 to correct editorial errors. LA3022214, CR1047811

Revision 35—06/29/17

Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
35	06/29/17	Administrative	As identified in the 2017 NRC Submittal List of Changed Pages and submitted Summary of Change.	Major Revision issued for the purpose of 2017 NRC Submittal. Updated footer on all pages to reflect Revision 35.
35	06/29/17	MP2-UCR-2017-004	10.3.4.2, Table 10.3-1	LA3022214 - Modify FSAR Section 10.3.4.2 Tests and Inspection to read "See Section 5.2.7.4.1…" in lieu of 5.2.8.4.2.
35	06/29/17	MP2-UCR-2015-025	14.0.11, 14.6.1.1, 14.6.1.6	Confirmatory Order EA-13-188, AREVA analysis ANP-3406, Rev. 0 - Remove credit for charging from the long-term portion of the inadvertent opening of the PORV (IOPORV) event.

Revision 35—06/29/17 (Continued)

Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
34.3	03/30/17	MP2-UCR-2016-008	9.5.1.1, 9.5.2.1, 9.5.3.1	The NRC requested additional information be provided in the FSAR to describe the procedural controls in place to maintain the Millstone 2 Spent Fuel Pool temperature below 150°F. LAR 15-571
34.2	12/22/16	MP2-UCR-2016-016	14.6.5.2, 14.6.5.2.5, 14.6.5.2.5.1, 14.6.5.2.5.2, 14.6.5.2.5.3, 14.6.5.2.5.4, 14.6.5.2.5.5, 14.6.5.2.5.8, 14.6.5.2.5.9, 14.6.5.2.6, 14.6.5.2.7, 14.6.5.2.8, Figures 14.6.5.2-1, 14.6.5.2-2, 14.6.5.2-3, 14.6.5.2-4, 14.6.5.2-5, 14.6.5.2-6, 14.6.5.2-7, 14.6.5.2-8, 14.6.5.2-9, 14.6.5.2-10, 14.6.5.2-11, 14.6.5.2-12, 14.6.5.2-13, 14.6.5.2-14, 14.6.5.2-15, 14.6.5.2-16, 14.6.5.2-17, 14.6.5.2-18, 14.6.5.2-19, 14.6.5.2-20, 14.6.5.2-21, 14.6.5.2-22, Tables 14.6.5.2-3, 14.6.5.2-4, 14.6.5.2-5, 14.6.5.2-6, 14.6.5.2-7	This SAR change request updates the Small Break Loss of Coolant Accident (SBLOCA) analysis presented in FSAR Section 14.6.5.2.
34.2	12/22/16	MP2-UCR-2016-012	15.2.1.21	MP3 HVK Chiller Service Water Thermal Performance Testing Elimination.
34.2	12/22/16	MP2-UCR-2016-010	15.2.1.2	Cancellation of Millstone 2 Boraflex Monitoring Program.

Revision 35—06/29/17 (Continued)

Revision	Revision Release Date	Change Activity	Document Elements Affected (Sections, Tables, Figures)	Summary Description of Changes Made under the provisions of 10 CFR 50.59 except where indicated in brackets.
34.2	12/22/16	MP2-UCR-2016-009	11.E.1	RM-8123 and RM-8262 are described incorrectly in the FSAR. This change removes the reference to a gamma geiger muller detector chamber type and replace it with the correct beta scintillator type detector.
34.2	12/22/16	MP2-UCR-2016-003	9.3.2.3	Update Subsection 9.3.2.3 to reflect changes made by DCN DM2-00-0166-07 that eliminated the packing leak-off lines on M22-SI-306.
34.2	12/22/16	MP2-UCR-2015-007	1.2.10.4, 1.A, 9.8.2.1.1, 9.8.2.1.2, 9.8.2.2.2, Figure 9.8-7, Figure 9.8-8A, Figure 9.8-8B, Figure 9.8-8C, 14.7.4.1	Reflects changes associated with LBDCR 12-MP2-008.
34.2	12/22/16	MP2-UCR-2013-017	Table 6.7-1, Table 9.9-11	Reflects correction of technical value in the respective tables.
34.1	9/29/16	MP2-UCR-2016-007	Chapter 1 section 1.A; Chapter 8 List of Figures; Chapter 8 Section 8.1.1.4, 8.1.2.1; 8.1.2.2; 8.1.2.3 and Figure 8.1-1A; 8.1-1B	Unit 2 SLOD Removal (LBDCR 15-MP2-006)
34.1	9/29/16	MP2-UCR-2016-006	Section 14.8.2.2.4	Millstone 2 NRC License Amendment 326, “Millstone Power Station Unit 2 - Issuance of Amendment to Revise Technical Specifications for Containment Leak Rate Testing (CAC No. MF5838),” March 31, 2016 (ADAMS Accession Number ML16068A312).
34.1	9/29/16	MP2-UCR-2016-004	Table 11.2-1	Correct typo identified by CR1017686.

LIST OF ABBREVIATIONS

Abbreviation	
ADH	Aerated Drain Header
AEAS	Auxiliary Exhaust Actuation Signal
AES	Auxiliary Exhaust System
ATWS	Anticipated Transient Without Scram
BOL	Beginning of Life
BWR	Boiling Water Reactor
CEA	Control Element Assembly
CEDM	Control Element Drive Mechanism
CEDS	Control Element Drive System
CFR	Code of Federal Regulations
CIAS	Containment Isolation Actuation Signal
CRFS	Control Room Filtration System
CSAS	Containment Spray Actuation Signal
CSS	Containment Spray System
CVCS	Chemical and Volume Control System
CVH	Containment Vent Header
DBA	Design Basis Accident
DBE	Design Basis Earthquake
DNB	Departure from Nucleate Boiling
DNBR	Departure from Nucleate Boiling Ratio
DRL	Division of Reactor Licensing
EB	Enclosure Building
EBFR	Enclosure Building Filtration Region
EBFS	Enclosure Building Filtration System

List of Abbreviations (Continued)

Abbreviation	
ECCS	Emergency Core Cooling System
EOL	End of Life
ESCS	Engineered Safety Features Control System
F	Degrees Fahrenheit
FPS	Fire Protection System
FT	Fuel Transfer
FW	Feed Water
HMP	High Mechanical Performance
HPSI	High Pressure Safety Injection
HTP	High Thermal Performance
IA	Instrument Air
ID	Inside Diameter
LOCA	Loss of Coolant Accident
LPSI	Low Pressure Safety Injection
MPC	Maximum Permissible Concentration
MSL	Mean Sea Level
MSS	Main Steam System
MTU	Metric Tons Uranium
MWD	Megawatt Days
NDT	Non-Destructive Testing
NDTT	Nil Ductility Transition Temperature
NS	Nitrogen Supply
NSSS	Nuclear Steam Supply System
OBE	Operating Basis Earthquake

List of Abbreviations (Continued)

Abbreviation	
OD	Outside Diameter
PA	Purge Air
PDH	Primary Drain Header
PDT	Primary Drain Tank
PPM	Parts Per Million
PVMP	Preoperational Vibration Monitoring Program
PWR	Pressurized Water Reactor
RBCCW	Reactor Building Closed Cooling Water
RBCCWS	Reactor Building Closed Cooling Water System
RCS	Reactor Coolant System
RWS	Radioactive Waste System
RWST	Refueling Water Storage Tank
SA	Station Air
SGBS	Steam Generator Blowdown System
SIAS	Safety Injection Actuation Signal
SIS	Safety Injection System
SRAS	Sump Recirculation Actuation Signal
SRST	Spent Resin Storage Tank
SS	Sampling System
TBCCWS	Turbine Building Closed Cooling Water System
WGH	Waste Gas Header
WREC	Westinghouse Rector Evaluation Center
WTR	Westinghouse Test Reactor

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14.0.7–2	FIGURE 14.0.7–2	Thermal Margin/Low Pressure Trip Function A1	N/A	
14.0.7–3	FIGURE 14.0.7–3	Thermal Margin/Low Pressure Trip Function QR1	N/A	
14.0.7–4	FIGURE 14.0.7–4	Not Used	N/A	
14.0.7–5	FIGURE 14.0.7–5	Verification of the Departure from Nucleate Boiling Limiting Condition for Operation	N/A	
14.0.7–6	FIGURE 14.0.7–6	Verification of Local Power Density Limiting Condition for Operation	N/A	
14.0.7–7	FIGURE 14.0.7–7	Linear Heat Rate Limiting Condition of Operation Used in Local Power Density Limiting Condition of Operation Verification	N/A	
14.1.3–1	FIGURE 14.1.3–1	Normalized Power and Heat Flux for the Increase in Steam Flow Event	N/A	
14.1.3–2	FIGURE 14.1.3–2	Reactivity Feedback for the Increase in Steam Flow Event	N/A	
14.1.3–3	FIGURE 14.1.3–3	Reactor Coolant Temperatures for Increase in Steam Flow Event	N/A	
14.1.3–4	FIGURE 14.1.3–4	Core Inlet Mass Flow Rate for the Increase in Steam Flow Event	N/A	
14.1.3–5	FIGURE 14.1.3–5	Pressurizer Pressure for the Increase in Steam Flow Event	N/A	
14.1.3–6	FIGURE 14.1.3–6	Steam Generator Pressures for the Increase in Steam Flow Event	N/A	

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14.1.3–8	FIGURE 14.1.3–8	Main Feedwater Flow for the Increase in Steam Flow Event	N/A	
14.1.3–9	FIGURE 14.1.3–9	Main Feedwater Temperature for the Increase in Steam Flow Event	N/A	
14.1.5.1–1	FIGURE 14.1.5.1–1	Normalized Core Power (0.20 ft^2 Asymmetric Break Inside Containment)	N/A	
14.1.5.1–2	FIGURE 14.1.5.1–2	Core Inlet Temperatures (0.20 ft^2 Asymmetric Break Inside Containment)	N/A	
14.1.5.1–3	FIGURE 14.1.5.1–3	Reactivity Feedback (0.20 ft^2 Asymmetric Break Inside Containment)	N/A	
14.1.5.1–4	FIGURE 14.1.5.1–4	Pressurizer Pressure (0.20 ft^2 Asymmetric Break Inside Containment)	N/A	
14.1.5.1–5	FIGURE 14.1.5.1–5	Steam Generator Pressures (0.20 ft^2 Asymmetric Break Inside Containment)	N/A	
14.1.5.1–6	FIGURE 14.1.5.1–6	Steam Mass Flow Rates (0.20 ft^2 Asymmetric Break Inside Containment)	N/A	
14.1.5.1–7	FIGURE 14.1.5.1–7	Normalized Power and Heat Flux (Asymmetric 3.51 ft^2 Break Inside Containment with Loss of Offsite Power)	N/A	
14.1.5.1–8	FIGURE 14.1.5.1–8	Reactor Coolant Temperatures (Asymmetric 3.51 ft^2 Break Inside Containment with Loss of Offsite Power)	N/A	
14.1.5.1–9	FIGURE 14.1.5.1–9	Normalized Reactor Coolant System Flow Rate (Asymmetric 3.51 ft^2 Break Inside Containment with Loss of Offsite Power)	N/A	
14.1.5.1–10	FIGURE 14.1.5.1–10	Pressurizer Pressure (Asymmetric 3.51 ft^2 Break Inside Containment with Loss of Offsite Power)	N/A	

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14.1.5.2–1	FIGURE 14.1.5.2–1	One Pump High Pressure Safety Injection System Delivery vs. Primary Pressure (Post-Scram Steam Line Break)	N/A	
14.1.5.2–2	FIGURE 14.1.5.2–2	Steam Generator Break Flow (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–3	FIGURE 14.1.5.2–3	Steam Generators' Secondary Pressures (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–4	FIGURE 14.1.5.2–4	Core Inlet Temperatures (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–5	FIGURE 14.1.5.2–5	Pressurizer Pressure (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–6	FIGURE 14.1.5.2–6	Pressurizer Level (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–7	FIGURE 14.1.5.2–7	Steam Generators' Secondary Mass (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–8	FIGURE 14.1.5.2–8	Reactivity Components (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–9	FIGURE 14.1.5.2–9	Reactor Power (HZP Post-Scram Steam Line Outside Containment Break with Offsite Power Available)	N/A	
14.1.5.2–10	FIGURE 14.1.5.2–10	Steam Generator Break Flow (HZP Post-Scram Steam Line Outside Containment Break with Loss of Offsite Power)	N/A	
14.1.5.2–11	FIGURE 14.1.5.2–11	Steam Generators' Secondary Pressures (HZP Post-Scram Steam Line Outside Containment Break with Loss of Offsite Power)	N/A	
14.1.5.2–12	FIGURE 14.1.5.2–12	Core Inlet Temperatures (HZP Post-Scram Steam Line Outside Containment Break with Loss of Offsite Power)	N/A	

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14.1.5.2–14	FIGURE 14.1.5.2–14	Pressurizer Level (HZP Post-Scram Steam Line Outside Containment Break with Loss of Offsite Power)	N/A	
14.1.5.2–15	FIGURE 14.1.5.2–15	Reactivity Components (HZP Post-Scram Steam Line Outside Containment Break with Loss of Offsite Power)	N/A	
14.1.5.2–16	FIGURE 14.1.5.2–16	Reactor Power (HZP Post-Scram Steam Line Outside Containment Break with Loss of Offsite Power)	N/A	
14.2.1–1	FIGURE 14.2.1–1	Reactor Power Level for Loss of External Load (Primary Overpressurization Case)	N/A	
14.2.1–2	FIGURE 14.2.1–2	Core Average Heat Flux for Loss of External Load (Primary Overpressurization Case)	N/A	
14.2.1–3	FIGURE 14.2.1–3	Reactor Coolant System Temperatures for Loss of External Load (Primary Overpressurization Case)	N/A	
14.2.1–4	FIGURE 14.2.1–4	Primary System Pressures for Loss of External Load (Primary Overpressurization Case)	N/A	
14.2.1–5	FIGURE 14.2.1–5	Total Reactivity for Loss of External Load (Primary Overpressurization Case)	N/A	
14.2.1–6	FIGURE 14.2.1–6	Reactor Power Level for Loss of External Load (Secondary Overpressurization Case)	N/A	
14.2.1–7	FIGURE 14.2.1–7	Core Average Hot Flux for Loss of External Load (Secondary Overpressurization Case)	N/A	
14.2.1–8	FIGURE 14.2.1–8	Reactor Coolant System Temperatures for Loss of External Load (Secondary Overpressurization Case)	N/A	
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14.2.1–12	FIGURE 14.2.1–12	Reactor Power Level for Loss of External Load (MDNBR Case)	N/A	
14.2.1–13	FIGURE 14.2.1–13	Normalized Heat Flux for Loss of External Load (MDNBR Case)	N/A	
14.2.1–14	FIGURE 14.2.1–14	Reactor Coolant System Temperature for Loss of External Load (MDNBR Case)	N/A	
14.2.1–15	FIGURE 14.2.1–15	Pressurizer Pressure for Loss of External Load (MDNBR Case)	N/A	
14.2.1–16	FIGURE 14.2.1–16	Total Reactivity for Loss of External Load (MDNBR Case)	N/A	
14.2.1–17	FIGURE 14.2.1–17	Maximum Secondary System Pressure for Loss of External Load (MDNBR Case)	N/A	
14.2.4–1	FIGURE 14.2.4–1	Reactor Power Level for MSIV Closure (Lower Steam Flow Case)	N/A	
14.2.4–2	FIGURE 14.2.4–2	Reactor Coolant System Temperatures for MSIV Closure (Lower Steam Flow Case)	N/A	
14.2.4–3	FIGURE 14.2.4–3	Pressurizer Pressure for MSIV Closure (Lower Steam Flow Case)	N/A	
14.2.4–4	FIGURE 14.2.4–4	Isolated Steam Generator Pressure at Bottom of Boiler Region for MSIV Closure (Lower Steam Flow Case))	N/A	
14.2.4–5	FIGURE 14.2.4–5	Open MSIV Steam Generator Steam Dome Pressure for MSIV Closure (Lower Steam Flow Case))	N/A	
14.2.7–1	FIGURE 14.2.7–1	Reactor Coolant System Loop Temperatures for Minimum Steam Generator Inventory Case: Offsite Power Available, “B” Motor-Driven AFW Pump Fails to Start	N/A	

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14.2.7–3	FIGURE 14.2.7–3	Pressurizer Level for Minimum Steam Generator Inventory Case: Offsite Power Available, “B” Motor-Driven AFW Pump Fails to Start	N/A	
14.2.7–4	FIGURE 14.2.7–4	Steam Generator for Liquid Mass Inventory for Minimum Steam Generator Inventory Case: Offsite to Power Available, “B” Motor-Driven AFW Pump Fails to Start	N/A	
14.2.7–5	FIGURE 14.2.7–5	Steam Generator Collapsed Liquid Level for Minimum Steam Generator Inventory Case: Offsite to Power Available, “B” Motor-Driven AFW Pump Fails to Start	N/A	
14.2.7–6	FIGURE 14.2.7–6	Reactor Coolant System Loop Temperatures for Maximum Pressurizer Level Case: Loss of Offsite Power, One Motor-Driven AFW Pump Fails to Start	N/A	
14.2.7–7	FIGURE 14.2.7–7	Steam Generator Dome Pressure for Maximum Pressurizer Level Case: Loss of Offsite Power, One Motor-Driven AFW Pump Fails to Start	N/A	
14.2.7–8	FIGURE 14.2.7–8	Pressurizer Level for Maximum Pressurizer Level Case: Loss of Offsite Power, One Motor-Driven AFW Pump Fails to Start	N/A	
14.2.7–9	FIGURE 14.2.7–9	Steam Generator Liquid Mass Inventory for Maximum Pressurizer Level Case: Loss of Offsite Power, One Motor-Driven AFW Pump Fails to Start	N/A	
14.2.7–10	FIGURE 14.2.7–10	Steam Generator Collapsed Liquid Level for Maximum Pressurizer Level Case: Loss of Offsite Power, One Motor-Driven AFW Pump Fails to Start	N/A	
14.3.1–1	FIGURE 14.3.1–1	Reactor Power Level for Loss of Forced Reactor Coolant Flow	N/A	

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14.3.1–2	FIGURE 14.3.1–2	Core Average Heat Flux for Loss of Forced Reactor Coolant Flow	N/A	
14.3.1–3	FIGURE 14.3.1–3	Reactor Coolant System Temperature for Loss of Forced Reactor Coolant Flow	N/A	
14.3.1–4	FIGURE 14.3.1–4	Pressurizer Pressure for Loss of Forced Reactor Coolant Flow	N/A	
14.3.1–5	FIGURE 14.3.1–5	Reactivities for Loss of Forced Reactor Coolant Flow	N/A	
14.3.1–6	FIGURE 14.3.1–6	Primary Coolant Flow Rate for Loss of Forced Reactor Coolant Flow	N/A	
14.3.1–7	FIGURE 14.3.1–7	Secondary Pressure for Loss of Forced Reactor Coolant Flow	N/A	
14.3-3–1	FIGURE 14.3-3–1	Reactor Power Level for Reactor Coolant Pump Rotor Seizure	N/A	
14.3-3–2	FIGURE 14.3-3–2	Core Average Heat Flux for Reactor Coolant Pump Rotor Seizure	N/A	
14.3-3–3	FIGURE 14.3-3–3	Reactor Coolant System Temperatures for Reactor Coolant Pump Rotor Seizure	N/A	
14.3-3–4	FIGURE 14.3-3–4	Pressurizer Pressure for Reactor Coolant Pump Rotor Seizure	N/A	
14.3-3–5	FIGURE 14.3-3–5	Reactivities for Reactor Coolant Pump Rotor Seizure	N/A	
14.3-3–6	FIGURE 14.3-3–6	Primary Coolant Flow Rate for Reactor Coolant Pump Rotor Seizure	N/A	
14.3-3–7	FIGURE 14.3-3–7	Secondary Pressure for Reactor Coolant Pump Rotor Seizure	N/A	
14.4.1–1	FIGURE 14.4.1–1	Reactor Power Level for Low Power Bank Withdrawal	N/A	
14.4.1–2	FIGURE 14.4.1–2	Core Average Heat Flux for Low Power Bank Withdrawal	N/A	
14.4.1–3	FIGURE 14.4.1–3	Reactor Coolant Temperatures for Low Power Bank Withdrawal	N/A	
14.4.1–4	FIGURE 14.4.1–4	Pressurizer Pressure for Low Power Bank Withdrawal	N/A	
14.4.1–5	FIGURE 14.4.1–5	Reactivities for Low Power Bank Withdrawal	N/A	

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14.4.2–1	FIGURE 14.4.2–1	Reactor Core Power for an Uncontrolled Bank Withdrawal at Power	N/A	
14.4.2–2	FIGURE 14.4.2–2	Core Average Heat Flux for an Uncontrolled Bank Withdrawal at Power	N/A	
14.4.2–3	FIGURE 14.4.2–3	Reactor Coolant System Temperatures for an Uncontrolled Bank Withdrawal at Power	N/A	
14.4.2–4	FIGURE 14.4.2–4	Pressurizer Pressure for an Uncontrolled Bank Withdrawal at Power	N/A	
14.4.2–5	FIGURE 14.4.2–5	Reactivities for an Uncontrolled Bank Withdrawal at Power	N/A	
14.4.2–6	FIGURE 14.4.2–6	Secondary Pressure for an Uncontrolled Bank Withdrawal at Power	N/A	
14.4.3.1–1	FIGURE 14.4.3.1–1	Reactor Power Level for the Limiting Dropped Control Rod/Bank Case	N/A	
14.4.3.1–2	FIGURE 14.4.3.1–2	Reactor Coolant System Temperatures for the Limiting Dropped Control Rod/Bank Case	N/A	
14.4.3.1–3	FIGURE 14.4.3.1–3	Pressurizer Pressure for the Limiting Dropped Control Rod/Bank Case	N/A	
14.4.3.1–4	FIGURE 14.4.3.1–4	Secondary Pressure for the Limiting Dropped Control Rod/Bank Case	N/A	
14.4.8–1	FIGURE 14.4.8–1	Core Power for a CEA Ejection (Minimum Departure for Nucleate Boiling Ratio Case)	N/A	
14.4.8–2	FIGURE 14.4.8–2	Core Average Heat Flux for a CEA Ejection (Minimum Departure for Nucleate Boiling Ratio Case)	N/A	
14.4.8–3	FIGURE 14.4.8–3	Reactor Coolant System Temperatures for a CEA Ejection (Minimum Departure for Nucleate Boiling Ratio Case)	N/A	

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14.4.8–5	FIGURE 14.4.8–5	Reactivities for a CEA Ejection (Minimum Departure for Nucleate Boiling Ratio Case)	N/A	
14.4.8–6	FIGURE 14.4.8–6	Secondary Pressure for a CEA Ejection (Minimum Departure for Nucleate Boiling Ratio Case)	N/A	
14.4.8–7	FIGURE 14.4.8–7	Core Power for a CEA Ejection (Overpressure)	N/A	
14.4.8–8	FIGURE 14.4.8–8	Core Average Heat Flux for a CEA Ejection (Overpressure)	N/A	
14.4.8–9	FIGURE 14.4.8–9	Primary System Temperatures for a CEA Ejection (Overpressure)	N/A	
14.4.8–10	FIGURE 14.4.8–10	Pressurizer Pressure for a CEA Ejection (Overpressure)	N/A	
14.4.8–11	FIGURE 14.4.8–11	Reactivities for a CEA Ejection (Overpressure)	N/A	
14.4.8–12	FIGURE 14.4.8–12	Secondary Pressure for a CEA Ejection (Overpressure)	N/A	
14.6.1–1	FIGURE 14.6.1–1	Reactor Power Level for an Inadvertent Opening of a Pressurized Water Reactor Pressurizer Pressure Relief Valve (Rated Power)	N/A	
14.6.1–2	FIGURE 14.6.1–2	Core Average Heat Flux for an Inadvertent Opening of a Pressurized Water Reactor Pressurizer Pressure Relief Valve (Rated Power)	N/A	
14.6.1–3	FIGURE 14.6.1–3	Reactor Coolant System Temperatures for an Inadvertent Opening of a Pressurized Water Reactor Pressurizer Pressure Relief Valve (Rated Power)	N/A	
14.6.1–4	FIGURE 14.6.1–4	Pressurizer Pressure for an Inadvertent Opening of a Pressurized Water Reactor Pressurizer Pressure Relief Valve (Rated Power)	N/A	
14.6.1–5	FIGURE 14.6.1–5	Reactivities for an Inadvertent Opening of a Pressurized Water Reactor Pressurizer Pressure Relief Valve (Rated Power)	N/A	

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14.6.3–1	FIGURE 14.6.3–1	Steam Generator Tube Rupture with the Loss of Offsite Power RCS Temperature Versus Time	N/A	
14.6.3–2	FIGURE 14.6.3–2	Steam Generator Tube Rupture with the Loss of Offsite Power Pressurizer Level Versus Time	N/A	
14.6.3–3	FIGURE 14.6.3–3	Steam Generator Tube Rupture with the Loss of Offsite Power Pressurizer Pressure Versus Time	N/A	
14.6.3–4	FIGURE 14.6.3–4	Steam Generator Tube Rupture with the Loss of Offsite Power Steam Generator Pressure Versus Time	N/A	
14.6.3–5	FIGURE 14.6.3–5	Steam Generator Tube Rupture with the Loss of Offsite Power Total Break Flow Rate Versus Time	N/A	
14.6.3–6	FIGURE 14.6.3–6	Steam Generator Tube Rupture with the Loss of Offsite Power Flashed Break Flow Rate Versus Time	N/A	
14.6.3–7	FIGURE 14.6.3–7	Steam Generator Tube Rupture with the Loss of Offsite Power Atmospheric Dump Valve Flow Rate per Steam Generator Versus Time	N/A	
14.6.3–8	FIGURE 14.6.3–8	Steam Generator Tube Rupture with the Loss of Offsite Power Main Steam Safety Valve Flow Rates per Steam Generator Versus Time	N/A	
14.6.3–9	FIGURE 14.6.3–9	Steam Generator Tube Rupture with the Loss of Offsite Power Auxiliary Feedwater Flow Versus Time	N/A	
14.6.5.1–1	FIGURE 14.6.5.1–1	Scatter Plot Operational Parameters	N/A	
14.6.5.1–2	FIGURE 14.6.5.1–2	PCT Versus PCT Time Scatter Plot	N/A	
14.6.5.1–3	FIGURE 14.6.5.1–3	PCT Versus PCT Time Scatter Plot	N/A	
14.6.5.1–4	FIGURE 14.6.5.1–4	Maximum Local Oxidation Versus PCT Scatter Plot	N/A	

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14.6.5.1–6	FIGURE 14.6.5.1–6	Peak Cladding Temperature (Independent of Elevation) for the Demonstration Case	N/A	
14.6.5.1–7	FIGURE 14.6.5.1–7	Break Flow for the Demonstration Case	N/A	
14.6.5.1–8	FIGURE 14.6.5.1–8	Core Inlet Mass Flux for the Demonstration Case	N/A	
14.6.5.1–9	FIGURE 14.6.5.1–9	Core Outlet Mass Flux for the Demonstration Case	N/A	
14.6.5.1–10	FIGURE 14.6.5.1–10	Void Fraction at RCS Pumps for the Demonstration Case	N/A	
14.6.5.1–11	FIGURE 14.6.5.1–11	ECCS Flows (Includes SIT, HPSI and LPSI) for the Demonstration Case	N/A	
14.6.5.1–12	FIGURE 14.6.5.1–12	Upper Plenum Pressure for the Demonstration Case	N/A	
14.6.5.1–13	FIGURE 14.6.5.1–13	Collapsed Liquid Level in the Downcomer for the Demonstration Case	N/A	
14.6.5.1–14	FIGURE 14.6.5.1–14	Collapsed Liquid Level in the Lower Plenum for the Demonstration Case	N/A	
14.6.5.1–15	FIGURE 14.6.5.1–15	Collapsed Liquid Level in the Core for the Demonstration Case	N/A	
14.6.5.1–16	FIGURE 14.6.5.1–16	Containment and Loop Pressures for the Demonstration Case	N/A	
14.6.5.1–17	FIGURE 14.6.5.1–17	Pressure Differences between Upper Plenum and Downcomer for the Demonstration Case	N/A	
14.6.5.2–1	FIGURE 14.6.5.2–1	Peak Cladding Temperature Versus Break Size (SBLOCA Break Spectrum)	N/A	
14.6.5.2–2	FIGURE 14.6.5.2–2	Reactor Power - 3.78-Inch Break	N/A	
14.6.5.2–3	FIGURE 14.6.5.2–3	Primary and Secondary System Pressures - 3.78-Inch Break	N/A	
14.6.5.2–4	FIGURE 14.6.5.2–4	Break Mass Flow Rate - 3.78-Inch Break	N/A	
14.6.5.2–5	FIGURE 14.6.5.2–5	Break Vapor Void Fraction - 3.78-Inch Break	N/A	

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14.6.5.2–7	FIGURE 14.6.5.2–7	Total Core Inlet Mass Flow Rate - 3.78-Inch Break	N/A	
14.6.5.2–8	FIGURE 14.6.5.2–8	Downcomer Collapsed Liquid Level - 3.78-Inch Break	N/A	
14.6.5.2–9	FIGURE 14.6.5.2–9	Inner and Outer Core Collapsed Liquid Level - 3.78-Inch Break	N/A	
14.6.5.2–10	FIGURE 14.6.5.2–10	Reactor Vessel Mass - 3.78-Inch Break	N/A	
14.6.5.2–11	FIGURE 14.6.5.2–11	RCS Loop Mass Flow Rates - 3.78-Inch Break	N/A	
14.6.5.2–12	FIGURE 14.6.5.2–12	Steam Generator Main Feedwater Mass Flow Rates - 3.78-Inch Break	N/A	
14.6.5.2–13	FIGURE 14.6.5.2–13	Steam Generator Auxiliary Feedwater Mass Flow Rates - 3.78-Inch Break	N/A	
14.6.5.2–14	FIGURE 14.6.5.2–14	Steam Generator Total Mass - 3.78-Inch Break	N/A	
14.6.5.2–15	FIGURE 14.6.5.2–15	Steam Generator Narrow Range Level % - 3.78-Inch Break	N/A	
14.6.5.2–16	FIGURE 14.6.5.2–16	High Pressure Safety Injection Mass Flow Rates - 3.78-Inch Break	N/A	
14.6.5.2–17	FIGURE 14.6.5.2–17	Low Pressure Safety Injection Mass Flow Rates - 3.78-Inch Break	N/A	
14.6.5.2–18	FIGURE 14.6.5.2–18	Safety Injection Tank Mass Flow Rates - 3.78-Inch Break	N/A	
14.6.5.2–19	FIGURE 14.6.5.2–19	Integrated Break Flow And ECCS Flow - 3.78-Inch Break	N/A	
14.6.5.2–20	FIGURE 14.6.5.2–20	Hot Assembly Collapsed Liquid Level - 3.78-Inch Break	N/A	
14.6.5.2–21	FIGURE 14.6.5.2–21	Hot Assembly Mixture Level - 3.78-Inch Break	N/A	
14.6.5.2–22	FIGURE 14.6.5.2–22	Peak Cladding Temperature At Pct Location (11.02 Ft) - 3.78-Inch Break	N/A	
14.6.5.2–23	FIGURE 14.6.5.2–23	(DELETED by FSARCR 00-MP2-023)	N/A	
14.6.5.2–24	FIGURE 14.6.5.2–24	(DELETED by FSARCR 00-MP2-023)	N/A	

LIST OF EFFECTIVE FIGURES (CONTINUED)

Figure Number	Figure Label	Title	Drawing Number	Sheet
14.6.5.3–1	FIGURE 14.6.5.3–1	Long Term Cooling Plan	N/A	
14.6.5.3–2	FIGURE 14.6.5.3–2	Reactor Coolant System Refill Time vs. Break Area	N/A	
14.6.5.3–3	FIGURE 14.6.5.3–3	Core Flush by Hot Side Injection for a Double-Ended Guillotine Cold Leg Break	N/A	
14.6.5.3–4	FIGURE 14.6.5.3–4	Inner Vessel Boric Acid Concentration vs. Time for a Double-Ended Guillotine Cold Leg Break	N/A	
14.8.2–1	FIGURE 14.8.2–1	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Containment Pressure vs. Time	N/A	
14.8.2–2	FIGURE 14.8.2–2	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Containment Temperature vs. Time	N/A	
14.8.2–3	FIGURE 14.8.2–3	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Mass Flow Rate vs. Time	N/A	
14.8.2–4	FIGURE 14.8.2–4	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Energy Release Rate vs. Time	N/A	
14.8.2–5	FIGURE 14.8.2–5	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Integrated Mass Flow vs. Time	N/A	
14.8.2–6	FIGURE 14.8.2–6	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Integrated Energy Release vs. Time	N/A	
14.8.2–7	FIGURE 14.8.2–7	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Affected Steam Generator Pressure vs. Time	N/A	

LIST OF EFFECTIVE FIGURES (CONTINUED)

Figure Number	Figure Label	Title	Drawing Number	Sheet
14.8.2–8	FIGURE 14.8.2–8	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Unaffected Steam Generator Pressure vs. Time	N/A	
14.8.2–9	FIGURE 14.8.2–9	Main Steam Line Break Analysis - 102% Power with Loss of Offsite Power and Failure of Vital Bus Cabinet VA-10 or VA-20 - Affected Steam Generator Liquid Mass vs. Time	N/A	
14.8.3–1	FIGURE 14.8.3–1	Deleted by FSARCR 04-MP2-018	N/A	
14.8.3–2	FIGURE 14.8.3–2	Deleted by FSARCR 04-MP2-018	N/A	
14.8.3–3	FIGURE 14.8.3–3	Deleted by FSARCR 04-MP2-018	N/A	
14.8.3–4	FIGURE 14.8.3–4	Deleted by FSARCR 04-MP2-018	N/A	
14.8.3–5	FIGURE 14.8.3–5	Deleted by FSARCR 04-MP2-018	N/A	
14.8.3–6	FIGURE 14.8.3–6	Deleted by FSARCR 04-MP2-018	N/A	

a. Incorporated by reference this engineering controlled drawing is, coincidentally, an MPS-2 FSAR figure. Refer to the controlled plant drawing for the latest revision.