



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 14, 2023

Mr. Charles V. McFeaters
President and Chief Nuclear Officer
PSEG Nuclear LLC - N09
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION AND SALEM NUCLEAR
GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT
NOS. 234, 347, AND 329 RE: REVISE TECHNICAL SPECIFICATIONS TO
DELETE METEOROLOGICAL TOWER LOCATION (EPID L-2023-LLA-0059)

Dear Mr. McFeaters:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment Nos. 234, 347, and 329, to Renewed Facility Operating License Nos. NPF-57, DPR-70, and DPR-75 for the Hope Creek Generating Station (Hope Creek), and Salem Nuclear Generating Station, Unit Nos. 1 and 2 (Salem), respectively, in response to your application dated April 21, 2023.

The amendments revise the Hope Creek and Salem technical specifications (TSs) to remove TS Section 5.5, Meteorological Tower Location. The proposed amendments remove the reference to the figures removed from the TS by the previously approved amendments.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's monthly *Federal Register* notice.

Sincerely,

/RA/

James S. Kim, Project Manager
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-354, 50-272, and 50-311

Enclosures:

1. Amendment No. 234 to NPF-57
2. Amendment No. 347 to DPR-70
3. Amendment No. 329 to DPR-75
4. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 234
Renewed License No. NPF-57

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, dated April 21, 2023, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 234, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Hipólito J. González, Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility Operating
License and Technical Specifications

Date of Issuance: August 14, 2023

ATTACHMENT TO LICENSE AMENDMENT NO. 234

HOPE CREEK GENERATING STATION

RENEWED FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove
3

Insert
3

Replace the following pages of Renewed Facility Operating License No. NPF-57 with the attached revised pages. The revised pages are identified by amendment number and contains a marginal line indicating the area of change.

Remove
xxiii
5-5

Insert
xxiii
5-5

reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility. Mechanical disassembly of the GE14i isotope test assemblies containing Cobalt-60 is not considered separation.
- (7) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Part 30, to intentionally produce, possess, receive, transfer, and use Cobalt-60.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at reactor core Power levels not in excess of 3902 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 234, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

INDEX

DESIGN FEATURES

<u>SECTION</u>	<u>PAGE</u>
<u>5.1 SITE LOCATION</u>	5-1
<u>5.2 CONTAINMENT</u>	
Configuration	5-1
Design Temperature and Pressure	5-1
Secondary Containment	5-1
<u>5.3 REACTOR CORE</u>	
Fuel Assemblies.....	5-4
Control Rod Assemblies	5-4
<u>5.4 REACTOR COOLANT SYSTEM</u>	
Design Pressure and Temperature	5-4
Volume	5-5
<u>5.5 DELETED</u>	5-5
<u>5.6 FUEL STORAGE</u>	
Criticality	5-5
Drainage.....	5-5
Capacity	5-5
<u>5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT</u>	5-5
Table 5.7.1 1 Component Cyclic or Transient Limits	5-6

DESIGN FEATURES

5.4 REACTOR COOLANT SYSTEM (continued)

VOLUME

5.4.2 The total water and steam volume of the reactor vessel and recirculation system is approximately 21,970 cubic feet at a nominal steam dome saturation temperature of 547°F.

5.5 DELETED

5.6 FUEL STORAGE

CRITICALITY

5.6.1 The spent fuel storage racks are designed and shall be maintained with:

- a. A k_{eff} equivalent to less than or equal to 0.95 when flooded with unborated water, including all calculational uncertainties and biases as described in Section 9.1.2 of the FSAR.
- b. A nominal 6.308 inch center-to-center distance between fuel assemblies placed in the storage racks.

5.6.1.2 The k_{eff} for new fuel for the first core loading stored dry in the spent fuel storage racks shall not exceed 0.98 when aqueous foam moderation is assumed.

DRAINAGE

5.6.2 The spent fuel storage pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 199' 4".

CAPACITY

5.6.3 The spent fuel storage pool shall be limited to a storage capacity of no more than 4006 fuel assemblies.

5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT

5.7.1 The components identified in Table 5.7.1-1 are designed and shall be maintained within the cyclic or transient limits of Table 5.7.1-1.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 347
Renewed License No. DPR-70

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and constellation Energy Generation, LLC (the licensees), dated April 21, 2023, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 347, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Hipólito J. González, Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed Facility Operating
License and Technical Specifications

Date of Issuance: August 14, 2023

ATTACHMENT TO LICENSE AMENDMENT NO. 347
SALEM NUCLEAR GENERATING STATION, UNIT NO. 1
RENEWED FACILITY OPERATING LICENSE NO. DPR-70
DOCKET NO. 50-272

Replace the following page of Renewed Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove
3

Insert
3

Replace the following pages of Renewed Facility Operating License No. DPR-70 with the attached revised pages as indicated. The revised pages are identified by amendment number and contains a marginal line indicating the area of change.

Remove
XVII
5-5

Insert
XVII
5-5

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at a steady state reactor core power level not in excess of 3459 megawatts (one hundred percent of rated core power).
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 347, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.
 - (3) Deleted Per Amendment 22, 11-20-79
 - (4) Less than Four Loop Operation

PSEG Nuclear LLC shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this renewed license) with less than four (4) reactor coolant loops in operation until safety analyses for less than four loop operation have been submitted by the licensees and approval for less than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this renewed license.
 - (5) PSEG Nuclear LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety

INDEX

DESIGN FEATURES

=====

<u>SECTION</u>	<u>PAGE</u>
<u>5.1 SITE LOCATION</u>	5-1
<u>5.2 CONTAINMENT</u>	
Configuration	5-1
Design Pressure and Temperature	5-4
<u>5.3 REACTOR CORE</u>	
Fuel Assemblies	5-4
Control Rod Assemblies	5-4
<u>5.4 REACTOR COOLANT SYSTEM</u>	
Design Pressure and Temperature	5-4
<u>5.5 DELETED</u>	5-5
<u>5.6 FUEL STORAGE</u>	
Criticality	5-5
Drainage	5-6a
Capacity	5-6a
<u>5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT</u>	5-6a

DESIGN FEATURES

- a. In accordance with the code requirements specified in Section 4.1 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 2485 psig, and
- c. For a temperature of 650°F, except for the pressurizer which is 680°F.

5.5 DELETED

5.6 FUEL STORAGE

CRITICALITY

- 5.6.1.1 The new fuel storage racks are designed and shall be maintained with:
- a. A maximum K_{eff} equivalent of 0.95 with the storage racks flooded with unborated water.
 - b. A nominal 21.0 inch center-to-center distance between fuel assemblies.
 - c. Unirradiated fuel assemblies with enrichments less than or equal to 4.25 weight percent (w/o) U-235 with no requirements for Integral Fuel Burnable Absorber (IFBA) pins.
 - d. Unirradiated fuel assemblies with enrichments (E) greater than 4.25 w/o U-235 and less than or equal to 5.0 w/o U-235 which contain a minimum number of Integral Fuel Burnable Absorber (IFBA) pins. This minimum number of IFBA pins shall have an equivalent reactivity hold-down which is greater than or equal to the reactivity hold down associated with N IFBA pins, at a nominal 2.35 mg B-10/linear inch loading (1.5X), determined by the equation below:

$$N = 42.67 (E - 4.25)$$



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 329
Renewed License No. DPR-75

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Constellation Energy Generation, LLC (the licensees), dated April 21, 2023, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 329, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Hipólito J. González, Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed Facility Operating
License and Technical Specifications

Date of Issuance: August 14, 2023

ATTACHMENT TO LICENSE AMENDMENT NO. 329
SALEM NUCLEAR GENERATING STATION, UNIT NO. 2
RENEWED FACILITY OPERATING LICENSE NO. DPR-75
DOCKET NO. 50-311

Replace the following page of Renewed Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove
3

Insert
3

Replace the following pages of Renewed Facility Operating License No. DPR-75 with the attached revised pages as indicated. The revised pages are identified by amendment number and contains a marginal line indicating the area of change.

Remove
XVII
5-5

Insert
XVII
5-5

- (3) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
 - (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;
 - (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at steady state reactor core power levels not in excess of 3459 megawatts (thermal).
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 329, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

INDEX

DESIGN FEATURES

=====

<u>SECTION</u>	<u>PAGE</u>
<u>5.1 SITE LOCATION</u>	5-1
<u>5.2 CONTAINMENT</u>	
Configuration	5-1
Design Pressure and Temperature	5-4
<u>5.3 REACTOR CORE</u>	
Fuel Assemblies	5-4
Control Rod Assemblies	5-4
<u>5.4 REACTOR COOLANT SYSTEM</u>	
Design Pressure and Temperature	5-4
<u>5.5 DELETED</u>	5-5
<u>5.6 FUEL STORAGE</u>	
Criticality	5-5
Drainage	5-5b
Capacity	5-5b
<u>5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT</u>	5-5b

DESIGN FEATURES

5.5 DELETED

5.6 FUEL STORAGE

CRITICALITY

- 5.6.1.1 The new fuel storage racks are designed and shall be maintained with:
- a. A maximum K_{eff} equivalent of equal to 0.95 with the storage racks flooded with unborated water.
 - b. A nominal 21.0 inch center-to-center distance between fuel assemblies.
 - c. Unirradiated fuel assemblies with enrichments less than or equal to 4.25 weight percent (w/o) U-235 with no requirements for Integral Fuel Burnable Absorber (IFBA) pins.
 - d. Unirradiated fuel assemblies with enrichments (E) greater than 4.25 w/o U-235 and less than or equal to 5.0 w/o U-235 which contain a minimum number of Integral Fuel Burnable Absorber (IFBA) pins. This minimum number of IFBA pins shall have an equivalent reactivity hold-down which is greater than or equal to the reactivity hold down associated with N IFBA pins, at a nominal 2.35 mg B-10/linear inch loading (1.5X), determined by the equation below:

$$N = 42.67 (E - 4.25)$$

- 5.6.1.2 The spent fuel storage racks are designed and shall be maintained with:
- a. A maximum K_{eff} equivalent of 0.95 with the storage racks filled with unborated water.
 - b. A nominal 10.5 inch center-to-center distance between fuel assemblies stored in Region 1 (flux trap type) racks.
 - c. A nominal 9.05 inch center-to-center distance between fuel assemblies stored in Region 2 (non-flux trap) racks.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 234, 347, AND 329 TO
RENEWED FACILITY OPERATING LICENSE NOS. NPF-57, DPR-70, AND DPR-75

PSEG NUCLEAR LLC

HOPE CREEK GENERATING STATION

AND

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-354, 50-272, AND 50-311

1.0 INTRODUCTION

By application dated April 21, 2023 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML23111A103), PSEG Nuclear LLC (PSEG, the licensee), submitted a license amendment request (LAR, application) to the U.S. Nuclear Regulatory Commission (NRC, Commission) to change technical specifications (TSs) for Salem Generating Station (Salem) Units 1 and 2, and Hope Creek Generating Station (Hope Creek).

The LAR proposed to remove Section 5.5, "Meteorological Tower Location," in its entirety from both the Salem and Hope Creek TSs.

1.1 Proposed Changes

The Salem and Hope Creek TS proposed deletions are identified below using strikethrough.

- Salem Unit 1 and Unit 2:

~~5.5 Meteorological Tower Location~~

~~5.5.1 The meteorological tower shall be located as shown in Figure 5.1-1.~~

- Hope Creek:

5.5 Meteorological Tower Location

~~5.5.1 The meteorological tower shall be located as shown in Figure 5.1.1-1.~~

The Index pages for both Salem and Hope Creek TS will also be revised to reflect the deletion of Section 5.5 as depicted in LAR Attachments 1, 2, and 3.

2.0 REGULATORY EVALUATION

Section 50.36, "Technical specifications," of Title 10 of the *Code of Federal Regulations* (10 CFR) contains the requirements for the content of TS. Pursuant to 10 CFR 50.36, TSs are required to include items in the following categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. Paragraph 50.36(c)(4) states, in part, that design features to be included in the TSs are those features of the facility such as materials of construction and geometric arrangements, which, if altered or modified, would have a significant effect on safety.

The NRC staff's guidance for the review of TSs is in NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR [Light Water Reactor] Edition" (SRP), Chapter 16.0, "Technical Specifications," Revision 3, dated March 2010 (ML100351425). As described therein, as part of the regulatory standardization effort, the NRC staff has prepared standard technical specifications (STS) for each of the LWR nuclear designs. Accordingly, for Westinghouse plant designs, the NRC staff's review includes consideration of whether the proposed changes are consistent with NUREG-1431¹, "Standard Technical Specifications – Westinghouse Plants." Additionally, for General Electric BWR/4 plant designs, the NRC staff's review includes consideration of whether the proposed changes are consistent with NUREG-1433², "Standard Technical Specifications – General Electric Plants (BWR/4)."

3.0 TECHNICAL EVALUATION

The NRC staff evaluated the licensee's application to determine whether the proposed changes described in sections 1.1 of this safety evaluation (SE) are consistent with the regulations and guidance described in section 2.0 of this SE.

In LAR section 1.0, "Summary Description," the licensee states the proposed changes to both Salem and Hope Creek are in alignment with NUREG-1431 and NUREG-1433, respectively. In accordance with SRP Chapter 16.0, the NRC staff determined that these STS are applicable because Salem is a Westinghouse plant design and Hope Creek is a General Electric (BWR/4) plant design.

¹ U.S. Nuclear Regulatory Commission, "Standard Technical Specifications, Westinghouse Plants," NUREG 1431, Volume 1, "Specifications," Revision 5, September 2021 (ADAMS Accession No. ML21259A155).

² U.S. Nuclear Regulatory Commission, "Standard Technical Specifications, General Electric Plants (BWR/4)" NUREG 1431, Volume 1, "Specifications," Revision 5, September 2021 (ADAMS Accession No. ML21272A357).

In accordance with the Commission's Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors (58 FR 39132; July 22, 1993), improved STS (e.g., NUREG-1431 and NUREG-1433) have been developed and are maintained for each nuclear steam supply system owners' group. The Commission encourages licensees to use the improved STS as the basis for plant-specific TSs. Although the Salem Units 1 and 2, and Hope Creek TSs are not based on the guidance in NUREG-1431 and NUREG-1433, respectively, the Commission policy also recognizes selective incorporation of improved STS requirements may occur by noting that "licensees may adopt portions of the improved STS without fully implementing all STS improvements."

As described in the LAR, the licensee proposed to delete TS Section 5.5 in its entirety. In conjunction with deleting TS Section 5.5, the licensee also proposed corresponding changes to the TS Index. TS 5.5 is located in the Design Features section of the Salem and Hope Creek TSs.

In LAR Section 3.0, "Technical Evaluation," the licensee states that "[t]he location of the meteorological tower is already defined in station-controlled documents such as plant drawings, the UFSAR [Updated Final Safety Analysis] and the ODCM [Offsite Dose Calculation Manual] for both stations." In the LAR, the licensee considers having these items in the TSs in addition to licensee-controlled station documents to be duplicative. In addition, in the application the licensee states that "[d]eletion of Section 5.5 from the TS does not affect the substance of any TS requirement. Deleting this TS for both stations is considered to be administrative in nature, does not affect the design or operation of any plant Structures, Systems or Components and will also align the Salem and Hope Creek TS with NUREG-1431 and NUREG-1433 respectively relative to unnecessary information within the TS."

The NRC staff reviewed the information provided in the application regarding the proposed deletion of TS 5.5 for both Salem and Hope Creek. Based on the information provided in the application and summarized above, the NRC staff determined that the proposed deletion of TS 5.5 will not impact any safety limits, limiting safety system settings, limiting conditions of operation or surveillance requirements as described in paragraphs (c)(1), (c)(2), and (c)(3) of 10 CFR 50.36. In addition, for each station, the NRC staff noted that the licensee stated that the location of the meteorological tower is contained in licensee-controlled documents such as the UFSAR, and the ODCM. Changes to the UFSAR are governed by the requirements of 10 CFR 50.59, "Changes, Tests, and Experiments," and changes to the ODCM are subject to the administrative control provisions of 10 CFR 50.36(c)(5) through the licensee's plant-specific TS 6.14 ODCM requirements. Furthermore, the NRC staff determined that the proposed changes align with guidance provided in NUREG-1431 and NUREG-1433 Design Features sections, which does not include information related to meteorological tower location.

Based on the above, the NRC staff concludes that the proposed removal of the design features information from the Salem and Hope Creek TSs that describes the location of the meteorological tower will have no significant impact on safety and, therefore, does not meet the criteria of 10 CFR 50.36(c)(4) for items to be included in TSs. Therefore, based on the preceding, the NRC staff finds the proposed changes are acceptable.

The application also contained changes to the TS Index that reflect the proposed changes to delete TS Section 5.5. The NRC staff finds the proposed changes acceptable because they are editorial clarifications and do not substantively change TS requirements.

3.3 Conclusion

Based on the above, the NRC staff determined that the proposed TS changes to Salem and Hope Creek are acceptable because the proposed TSs are consistent with applicable STS guidance and comply with the requirements of 10 CFR 50.36.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments on July 31, 2023. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change the format of the license or permit or otherwise make editorial, corrective or other minor revisions. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, as published in the *Federal Register* on June 13, 2023 (88 FR 38551), and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Ashley

Date: August 14, 2023

SUBJECT: HOPE CREEK GENERATING STATION AND SALEM NUCLEAR
GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT
NOS. 234, 347, AND 329 RE: REVISE TECHNICAL SPECIFICATIONS TO
DELETE METEOROLOGICAL TOWER LOCATION (EPID L-2023-LLA-0059)
DATED AUGUST 14, 2023

DISTRIBUTION:

Public
PM File Copy
RidsACRS_MailCTR Resource
RidsNrrDorLpl1 Resource
RidsRgn1MailCenter Resource

RidsNrrDssStsb Resource
RidsNrrLAKEntz Resource
RidsNrrPMSalem Resource
RidsNrrPMHopeCreek Resource
CAshley, NRR

ADAMS Accession No.: ML23192A821**NRR-058**

OFFICE	NRR/DORL/LPL1/PM	NRR/DORL/LPL1/LA	NRR/DSS/STSB/BC
NAME	JKim	KEntz	VCusumano
DATE	7/17/2023	7/13/2023	6/16/2023
OFFICE	OGC – NLO	NRR/DORL/LPL1/BC	NRR/DORL/LPL1/PM
NAME	MCarpentier	HGonzález	JKim
DATE	7/27/2023	8/14/2023	8/14/2023

OFFICIAL RECORD COPY