



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

July 13, 2023

Mr. David P. Rhoades  
Senior Vice President  
Constellation Energy Generation, LLC  
President and Chief Nuclear Officer  
Constellation Nuclear  
4300 Winfield Road  
Warrenville, IL 60555

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2 –  
CORRECTION TO THE RENEWED FACILITY OPERATING LICENSE PAGES  
IN AMENDMENT NOS. 340 AND 318 (EPID L-2020-LLA-0256)

Dear Mr. Rhoades:

The U.S. Nuclear Regulatory Commission (NRC) staff has identified two typographical errors in Renewed Facility Operating License (RFOL) No. DPR-53, Unit 1, and RFOL DPR-69 for Calvert Cliffs Nuclear Power Plant (Calvert Cliffs), Unit 2. The NRC staff is issuing this letter to correct the errors per the guidance in the NRC memorandum dated January 16, 1997 (Agencywide Documents Access and Management System Accession (ADAMS) No. ML103260096), which is derived from the Staff Requirements Memorandum (ML003754054) for SECY-96-238 (ML20134M324, non-public).

By letter dated December 14, 2021 (ML21299A005), the NRC staff issued Amendment Nos. 340 to RFOL DPR-53 for Calvert Cliffs, Unit 1, and Amendment 318 for RFOL DPR-69, Unit 2, in response to license amendment request dated November 24, 2020 (ML20329A334), as supplemented by letter dated July 26, 2021 (ML21208A007), from Constellation Energy Generation, LLC.

Two typographical errors on technical specification (TS) page 5.5-16 were introduced. The first error occurred on top of the page when parts c and d were deleted. The second error occurred at the bottom of the page when the following text was shifted from the next page:

The peak calculated containment internal pressure for the design basis loss-of-coolant accident,  $P_a$ , is 49.7 psig. The containment design pressure is 50 psig.

To correct these errors, the NRC staff is re-issuing RFOL DPR-53 and DPR-69, TS page 5.5-16, adding back in parts c and d at the top of the page and removing the text from the bottom of the page.

The enclosure to this letter has a copy of the revised and corrected RFOL DPR-53 and DPR -69 page 5.5-16.

If you have any questions, please contact me at 301-415-0489 or by email to [Sujata.Goetz@nrc.gov](mailto:Sujata.Goetz@nrc.gov).

Sincerely,

*/RA/*

Sujata Goetz, Project Manager  
Plant Licensing Branch I  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

Enclosure:  
Corrected RFOL No. DPR-53  
and DPR-69 page 5.5-16

cc: Listserv

**ENCLOSURE**

CORRECTED RENEWED FACILITY OPERATING LICENSE NO. DPR-53  
PAGE 5.5-16

## 5.5 Programs and Manuals

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- c. Provisions to ensure that an inoperable supported system's Completion Time is not inappropriately extended as a result of multiple support system inoperabilities; and
- d. Other appropriate limitations and remedial or compensatory actions.

A loss of safety function exists when, assuming no concurrent single failure, no concurrent loss of offsite power or no concurrent loss of onsite diesel generator(s), a safety function assumed in the accident analysis cannot be performed. For the purpose of this program, a loss of safety function may exist when a support system is inoperable, and:

- a. A required system redundant to system(s) supported by the inoperable support system is also inoperable; or
- b. A required system redundant to system(s) in turn supported by the inoperable supported system is also inoperable; or
- c. A required system redundant to support system(s) for the supported systems (a) and (b) above is also inoperable.

The SFDP identifies where a loss of safety function exists. If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered. When a loss of safety function is caused by the inoperability of a single Technical Specification support system, the appropriate Conditions and Required Actions to enter are those of the support system.

### 5.5.16 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage testing of the containment as required by 10 CFR 50.54(o) and 10 CFR Part 50, Appendix J, Option B. This program shall be in accordance with the guidelines contained in Nuclear Energy Institute (NEI) 94-01, "Industry Guideline for Implementing Performance Based Option of 10 CFR Part 50, Appendix J," Revision 3-A, dated July 2012, and the conditions and limitations specified in NEI 94-01, Revision 2-A dated October 2008.

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JULY 13, 2023

**DISTRIBUTION:**

PUBLIC

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- RidsNrrPMCalvert Cliffs Resource
- RidsNrrLAKEntz Resource
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**ADAMS Accession No.: ML23186A037**

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