

Planning Basis for Radiological Emergency Preparedness

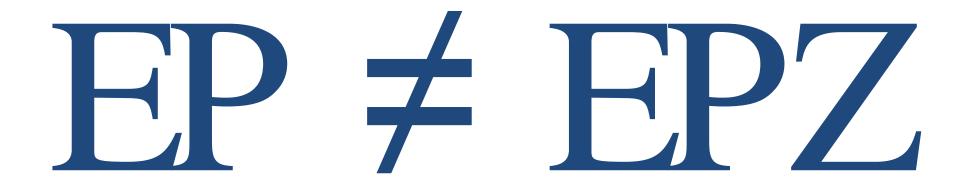
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Objective of Radiological EP

- The objective of emergency preparedness (EP) is to provide *dose savings* for a spectrum of accidents that could produce doses in excess of the Environment Protection Agency (EPA) protective action guides (PAG)
- Meeting NRC EP regulations provides reasonable assurance that *adequate protective measures can and will be taken* in the event of a radiological emergency
 - Reasonable assurance finding is made before a nuclear facility is licensed
 - Inspected over the lifetime of that facility







The NRC employs a graded approach to EP

- A graded approach is a process by which the safety requirements and criteria are set commensurate with several factors including magnitude of hazards involved, characteristics of a facility, the balance between radiological and nonradiological hazards.
- EP regulations employ a graded approach, which is a risk-informed process
 - Power reactors (low-power testing, power operations, decommissioning)
 - Research and test reactors
 - Fuel Fabrication Facilities
 - Independent Spent Fuel Storage Installations
 - Monitored Retrievable Storage



EP has a firmly established risk-informed basis

The consequences from a **spectrum of accidents**, tempered by probability considerations, should be considered to scope the planning efforts for:

- The **distance** to which planning for predetermined protective actions is warranted
- The **time** dependent characteristics of a potential release
- The type of radioactive *materials*

The planning basis document included a recommended 10 mile plume exposure path emergency planning zone (EPZ) and a 50 mile ingestion pathway zone for large light water reactors (PWR and BWR)



Planning Distance

The **<u>distance</u>** to which planning for predetermined protective actions is warranted

• The EPZ is a planning tool

"...it was the consensus of the Task Force that emergency plans could be based upon a generic distance out to which predetermined actions would provide dose savings..."

"...beyond the generic distance it was concluded that actions could be taken on an ad hoc basis..."

"The EPZ guidance does not change the requirements for emergency planning, it only sets bounds on the planning problem."



The EPZ is scalable

- EPZ size based on the consequences from a spectrum of accidents, tempered by probability considerations.
- NRC regulations provide for scalable EPZs
- Reactors have been approved for a 5 mile EPZ in the past
- Depending on facility type, the EPZ may be at the site-boundary or have no EPZ
- Considerable number of studies since the 1980s on sizing EPZs for passive and advanced reactor designs, many based on the NUREG-0396 methodology



EPZ simplifies decisions for a prompt response

Protective Action Guide (PAG)

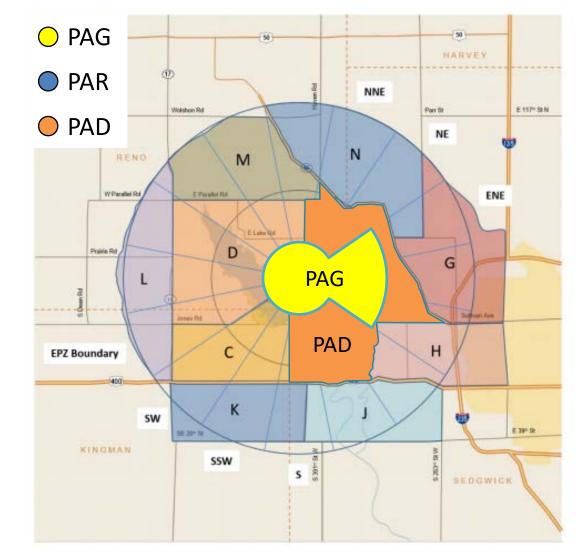
projected dose to an individual member of the public that warrants protective action

Protective Action Recommendation (PAR) recommended protective measure from

the nuclear power plant to offsite response organizations (OROs)

Protective Action Decision (PAD)

measures taken in response to an actual or anticipated radiological release





Planning Time

The *time* dependent characteristics of a potential release

Table 2 - Guidance on Initiation and Duration of Release

Time from the initiating event	0.5 hours to one day
to start of atmospheric release	
Time period over which radioactive material may be continuously released	0.5 hours to several day
Time at which major portion of release may occur	0.5 hours to 1 day after start of release
Travel time for release to exposure point (time after release)	5 miles 0.5 to 2 ho 10 miles 1 to 4 hour



Effectiveness of protective actions related to timing

"The guidance cannot be very specific because of the wide range of time frames associated with the spectrum of accidents considered."

"Therefore, it will be necessary for planners to consider the possible different time periods between the initiating event and arrival of the plume and possible time periods of release in relationship to time needed to implement protective actions."



Timing considerations are conservatively bounded

TABLE V 2-1 SUMMARY OF ACCIDENTS INVOLVING CORE

	PROBABILITY		DURATION OF RELEASE	WARNING TIME FOR EVACUATION	ELEVATION OF RELEASE	CONTAINMENT ENERGY RELEASE	FRACTION OF CORE INVENTORY RELEASED							
RELEASE CATEGORY	Reactor-Yr	(Hr)	(Hr)	(Hr)	(Meters)	(10 ⁶ Btu/Hr)	Xe-Kr			Cs-Rb	Te-Sb	Ba-Sr		La ^(c)
PWR,1	9×10 ⁻⁷	2.5	0.5	1.0	25	520 ^(d)	0.9	6x10 ⁻³	0.7	0.4	0.4	0.05	0.4	3x10
PWR 2	8x10 ⁻⁶	2.5	0.5	1.0	0	170 .	0.9	7×10-3	0.7	0.5	0.3	0.06	0.02	4x10
PWR 3	4x10 ⁻⁶	5.0	1.5	2.0	0	6	0.8	6×10 ⁻³	0.2	0.2	0.3	0.02	0.03	3x10
PWR 4	5x10 ⁻⁷	2.0	3.0	2.0	0	1	0.6	2x10 ⁻³	0.09	0.04	0.03	5x10 ⁻³	3×10 ⁻³	4x10
PWR 5	7x10 ⁻⁷	2.0	4.0	1.0	0	0.3	0.3	2×10 ⁻³	0.03		5x10 ⁻³	1x10 ⁻³	6x10 ⁻⁴	7x10
PWR 6	6x10 ⁻⁶	12.0	10.0	1.0	O	N/A	0.3		8x10 ⁻⁴	8×10 ⁻⁴	1×10 ⁻³	9×10 ⁻⁵	7x10 ⁻⁵	1×10
PWR 7	4x10 ⁻⁵	10.0	10.0	1.0	0	N/R	6x10 ⁻³	2×10 ⁻⁵	2×10 ⁻⁵		2x10 ⁻⁵	1x10 ⁻⁶	1×10 ⁻⁶	2x10
PWR 8	4x10 ⁻⁵	0.5	0.5	N/A	0	N/A	2×10 ⁻³	5x10 ⁻⁶	1×10 ⁻⁴	5x10 ⁻⁴	1×10 ⁻⁶	1x10 ⁻⁸	o	0
PWR 9	4x10 ⁻⁴	0.5	0.5	N/A	٩	N/A ·	3×10 ⁻⁶	7x10 ⁻⁹	1×10 ⁻⁷	6x10 ⁻⁷	1x10 ⁻⁹	1x10 ⁻¹¹	0	0
BWR 1	1×10 ⁻⁶	2.0	2.0	1.5	25	130	1.0	7x10-3	0.40	0.40	0.70	0.05	0.5	5x10
BWR 2	6x10 ⁻⁶	30.0	3.0	2.0	0	30	1.0	7x10 ⁻³	0.90	0.50	0.30	0.10	0.03	4x10
BWR 3	2x10 ⁻⁵	30.0	3.0	2.0	25	20	1.0	7x10 ⁻³	0.10	0.10	0.30	0.01	0.02	3x10
BWR 4	2x10 ⁻⁶	5.0	2.0	2.0	25	N/A	0.6		8×10 ⁻⁴		4x10 ⁻³	6x10 ⁻⁴	6x10 ⁻⁴	1x10
BWR 5	1x10 ⁻⁴	3.5	5.0	N/A	150	N/A	5×10-4	2x10 ⁻⁹	6x10 ⁻¹¹	4x10 ⁻⁹	8×10 ⁻¹²	8x10 ⁻¹⁴	0	0

NUREG-075/014 (WASH-1400), "Reactor Safety Study: An Assessment of Accident Risks in U.S. Commercial Nuclear Power Plants," October 1975



<u>Time</u> basis informs functional requirements

- ...nuclear power reactor licensees shall establish and maintain the capability to assess, classify, and declare an emergency condition within 15 minutes after the availability of indications to plant operators that an emergency action level has been exceeded...
- A licensee shall have the **capability** to notify responsible State and local governmental agencies within **15 minutes** after declaring an emergency
- The design objective of the prompt public alert and notification system shall be to have the **capability** to essentially complete the initial alerting and initiate notification of the public within the plume exposure pathway EPZ within about **15 minutes**



Our understanding of accidents has evolved...

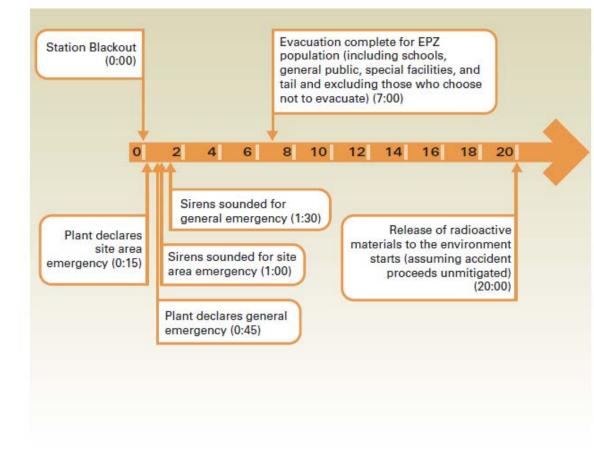
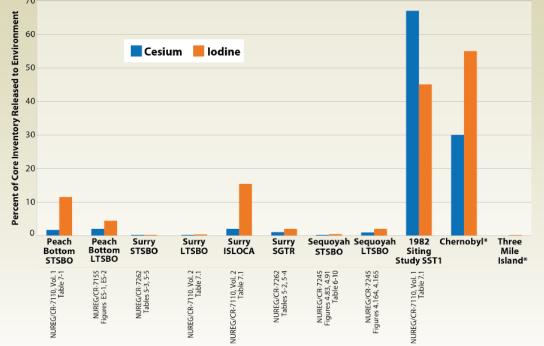


Figure 4.1 Percentages of Cesium and Iodine Released to the Environment for SOARCA Unmitigated Scenarios, 1982 Siting Study (SST1), and Historical Accidents

The SOARCA unmitigated release of Cesium-137 and lodine-131, for each of the modelled scenarios, are much smaller than estimated in the earlier 1982 Siting Study Source Term 1 (SST1) case. Some of these releases develop over a period of time and are also much smaller than those from the Chernobyl accident.



* Chernobyl release data is estimated at 20-40 percent for cesium-137 and 50-60 percent for iodine-131. Three Mile Island released an extremely small quantity of iodine-131 (~ 15 curies) and zero cesium-137. Fukushima releases are estimated to be approximately one-tenth of releases from Chernobyl [IAEA Report GC(59)/14].

NUREG/BR-0359, Revision 3, "Modeling Potential Reactor Accident Consequences—State-of-the-Art Reactor Consequence Analyses: Using decades of research and experience to model accident progression, mitigation, emergency response, and health effects," October 2020



Release Characteristics

The type of radioactive *materials*

Table 3 RADIONUCLIDES WITH SIGNIFICANT CONTRIBUTION TO DOMINANT EXPOSURE MODES

adionuclides with ontribution to Thy	vroid Exposure	Radionuclides with Contribution to Who	le Body Exposure	Radionuclides with Significant Contribution to Lung Exposure* (Lung only controlling when thyroid dose is reduced by iod blocking or there is a long del prior to releases).				
Radionuclide	Half Life (days)	Radionuclide	Half Life (days)	Radionuclide	Half Life (days)			
I-131	8.05	I-131	8.05	I-131	8.05			
I-132	0.0858	Te-132	3.25	I-132	0.0858			
I-133	0.875	Xe-133	5.28	I-133	0.875			
I-134	0.0366	I-133	0.875	I-134	0.0366			
I-135	.028	Xe-135	0.384	1-135	.028			
Te-132	3.25	I-135	.028	Cs-134	750 😫			
Kr-88	0.117	Cs-134	750	Kr-88	0.117 '			
		Kr-88	0.117	Cs-137	11,000			
		Cs-137	11,000	Ru-106	365			
				Te-132	3.25			
	·			Ce-144	284			



WASH-1400 informed early understanding

	PROBABILITY		DURATION	WARNING TIME FOR EVACUATION (Hr)	ELEVATION OF	CONTAINMENT ENERGY RELEASE	FRACTION OF CORE INVENTORY RELEASED								
RELEASE CATEGORY	per Reactor-Yr	(Hr)	RELEASE (Hr)		RELEASE (Meters)	(10 ⁶ Btu/Hr)	Xe-Kr	Org. I	I	Cs-Rb	Te-Sb	Ba-Sr	Ru ^(b)	La ^(c)	
PWR,1	9×10 ⁻⁷	2.5	0.5	1.0	25	520 ^(d)	0.9	6x10 ⁻³	0.7	0.4	0.4	0.05	0.4	3x10-1	
PWR 2	8x10 ⁻⁶	2.5	0.5	1.0	0	170	0.9	7x10 ⁻³	0.7	0.5	0.3	0.06	0.02	4x10	
PWR 3	4x10 ⁻⁶	5.0	1.5	2.0	0	6	0.8	6×10 ⁻³	0.2	0.2	0.3	0.02	0.03	3x10	
PWR 4	5x10 ⁻⁷	2.0	3.0	2.0	0	1	0.6	2x10 ⁻³	0.09	0.04	0.03	5x10 ⁻³	3×10 ⁻³	4x10	
PWR 5	7x10 ⁻⁷	2.0	4.0	1.0	٥	0.3	0.3	2×10 ⁻³	0.03	9x10 ⁻³	5x10 ⁻³	1x10 ⁻³	6x10 ⁻⁴	7x10	
PWR 6	6x10 ⁻⁶	12.0	10.0	1.0	O	N/A	0.3	2x10 ⁻³	8x10 ⁻⁴	8×10 ⁻⁴	1x10 ⁻³	9x10 ⁻⁵	7x10-5	1x10	
PWR 7	4x10 ⁻⁵	10.0	10.0	1.0	0	N/R	6x10 ⁻³		2x10 ⁻⁵		2x10 ⁻⁵	1x10 ⁻⁶	1×10 ⁻⁶	2x10	
PWR 8	4x10 ⁻⁵	0.5	0.5	N/A	٥	N/A		5x10 ⁻⁶	1x10 ⁻⁴	5x10 ⁻⁴	1×10 ⁻⁶	1x10 ⁻⁸	0	0	
PWR 9	4x10 ⁻⁴	0.5	0.5	N/A	٩	N/X ·	3×10 ⁻⁶	7x10 ⁻⁹	1×10 ⁻⁷	6x10 ⁻⁷	1×10 ⁻⁹	1x10 ⁻¹¹	0	0	
BWR 1	1×10 ⁻⁶	2.0	2.0	1.5	25	130	1.0	7x10 ⁻³	0.40	0.40	0.70	0.05	0.5	5x10	
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BWR 5	1x10 ⁻⁴	3.5	5.0	N/A	150	N/A	5x10 ⁻⁴	2x10 ⁻⁹	6x10 ⁻¹¹	4x10 ⁻⁹	8x10 ⁻¹²	8x10 ⁻¹⁴	0	0	



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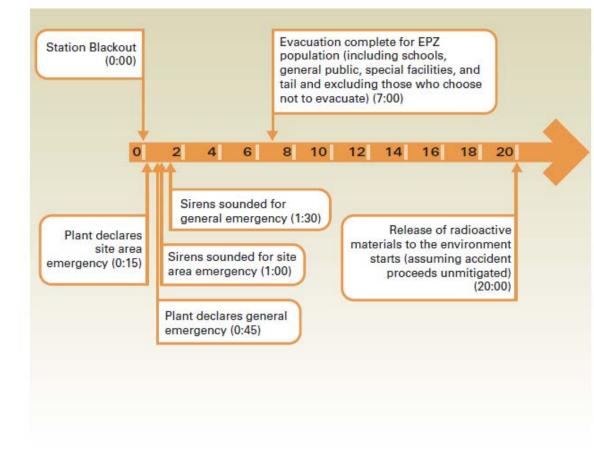
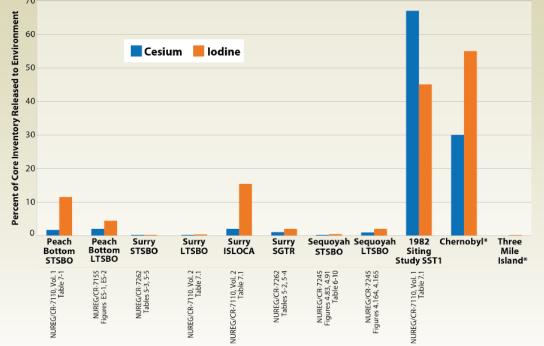


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NUREG/BR-0359, Revision 3, "Modeling Potential Reactor Accident Consequences—State-of-the-Art Reactor Consequence Analyses: Using decades of research and experience to model accident progression, mitigation, emergency response, and health effects," October 2020



Planning basis balances protection and resources

The consequences from a spectrum of accidents, **tempered by probability considerations**, should be considered to scope the planning efforts for:

- The distance to which planning for predetermined protective actions is warranted
- The time dependent characteristics of a potential release
- The type of radioactive materials

The planning basis document included a recommended 10 mile plume exposure path emergency planning zone (EPZ) and a 50 mile ingestion pathway zone for large light water reactors (PWR and BWR)



The likelihood of events considered is very low

	PROBABILITY	TIME OP	DURATION OF	TIME FOR	OF	CONTAINMENT ENERGY RELEASE	FRACTION OF CORE INVENTORY RELEASED							
release Category	per Reactor-Yr	RELEASE R	(Hr)	EVACUATION (Hr)	RELEASE (Meters)	(10 ⁶ Btu/Hr)	Xe-Kr	Org. 1	I	Cs-Rb	Te-Sb	Ba-Sr		La ^(c)
PWR 1	9×10 ⁻⁷	2.5	0.5	1.0	25	520 ^(d)	0.9	6x10 ⁻³	0.7	0.4	0.4	0.05	0.4	3x10
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PWR 5	7x10 ⁻⁷	2.0	4.0	1.0	٥	0.3	0.3	2×10 ⁻³			5x10 ⁻³	1×10 ⁻³	6x10 ⁻⁴	7×10
PWR 6	6x10 ⁻⁶	12.0	10.0	1.0	O	N/A	0.3		8x10 ⁻⁴	8×10 ⁻⁴	1×10 ⁻³	9×10 ⁻⁵	7x10-5	
PWR 7	4x10 ⁻⁵	10.0	10.0	1.0	o	N/R	6x10 ⁻³	2x10 ⁻⁵	2×10 ⁻⁵	1×10 ⁻⁵	2x10 ⁻⁵	1x10 ⁻⁶	1×10 ⁻⁶	2×10
PWR 8	4x10 ⁻⁵	0.5	0.5	N/A	٥	N/A	2×10 ⁻³	5x10 ⁻⁶	1×10 ⁻⁴		1×10 ⁻⁶	1x10 ⁻⁸	0	0
PWR 9	4x10 ⁻⁴	0.5	0.5	N/A	٩	N/A ·	3x10 ⁻⁶	7x10 ⁻⁹	1×10 ⁻⁷	6x10 ⁻⁷	1x10 ⁻⁹	1x10 ⁻¹¹	0	0
BWR 1	1×10 ⁻⁶	2.0	2.0	1.5	25	130	1.0	7x10-3	0.40	0.40	0.70	0.05	0.5	5x10
BWR 2	6x10 ⁻⁶	30.0	3.0	2.0	0	30	1.0	7×10 ⁻³	0.90	0.50	0.30	0.10	0.03	4x10
BWR 3	2x10 ⁻⁵	30.0	3.0	2.0	25	20	1.0	7x10 ⁻³	0.10	0.10	0.30	0.01	0.02	-
BWR 4	2x10 ⁻⁶	5.0	2.0	2.0	25	N/A	0.6	7x10 ⁻⁴	8×10 ⁻⁴	5×10 ⁻³	4x10 ⁻³	6x10 ⁻⁴	6x10 ⁻⁴	1x10
BWR 5	1x10 ⁻⁴	3.5	5.0	N/A	150	N/A	5×10 ⁻⁴	2×10 ⁻⁹	6x10 ⁻¹¹	4×10 ⁻⁹	8×10 ⁻¹²	8x10 ⁻¹⁴	0	0

TABLE V 2-1 SUMMARY OF ACCIDENTS INVOLVING CORE

NUREG-075/014 (WASH-1400), "Reactor Safety Study: An Assessment of Accident Risks in U.S. Commercial Nuclear Power Plants," October 1975



...down to 1 chance in 10 lifetimes of the universe

TABLE V 3-4 PWR LARGE LOCA ACCIDENT SEQUENCES vs. RELEASE CATEGORIES

1

AB-a

ACDGI-a

AHFI-a

ACHF-a

ACDI-a

ACDG-a

AGI-a AFI-a

ACG-a

ACGI-a

ACF-a

ACDF-a ACEI-a ACEI-a

ACEGI-a

3 x 10⁻¹⁰

AE (8)

2 x 10⁻¹⁰

5 x 10⁻⁸

1 x 10⁻¹¹ 7 x 10⁻⁹ 1 x 10⁻⁹

ACEF-a ACE-a AHF-a

Core melt | No core melt Release Categories 7 2 9 3 Dominant Large LOCA Accident Sequences With Point Estimates ACD-8 1x10-11 AD-8 4x10-9 AB-Y 1x10-10 AD-a AB-c 1x109 AD−ε A-B 2 x10-7 A 1x10⁻⁴ 2x10⁻⁶ 2x10⁻⁸ 1x10-11 AH-β 3x10⁻⁹ ADF- 8-10 AH-ε 1x10-6 AHF-Y_11 2x10 AH-a AF-a 1x10-10 1x10⁻⁸ AHF-ε 1x10-10 AF-8 ACD-a 5x10-11 AB−§ 1x10⁻⁸ 4x 10⁻¹¹ AG-8 AG-a 9x10-11 9x10-9 Other Large LOCA Accident Sequences AHG-a ACDGI-8 AHI- B ACHGI- e AHG-6 AI-8 AI ADF-8 AHGI-a ADF-a ADFI-a ADG- 8 ACDI-8 ACDG-8 AHG- B AHGI-B ADI-B AHFI-6 ACHF-6 AHFI- c AHGI- 6 AC-B ACI-B AC ADFI- c AHGI- c ACDF- E ACH-c ACHP-Y ACH-6 ACDF-Y ACH-a ADGI- 8 ACDGI-E ACHI- # ACHI-a ACHI-a ACHG-a ACHGI-a AGI-6 AFI-6 ACEP-Y ACE- B ACHI-B ACHF- c ACHG- d AHFI-8 ACEI- B ACEG- B ACHG-8 AE- B AEF- E ACHG- E AEFI-E ADFI-B ACHF-B ACHGI- c ACEF-E ACEGI-B AEI-B ACDI- e ACEGI-E ACDF-6 AEG-6 ACDG- & AHF-6 AHFI-Y AEF-B AEFI-B ACEF-B ACG-6 ACGI-8 ACF-6 AEGI-B ACDG- E ADG- 6 ADG1-6 AHI-a AHG- E ADGI-a ADI- E ACEF-B AEF-ô AEFI-ô ACEF-ô AB-B AHF-B ADI-a ADG-a ADG- E ACD-E AE-a AEI-a AEF-a ADGI-C 3-IHA AE-E AEFI-a AEI-E AEG-a ACE-C ACEI-C ACEG-e ACEG-6 ACEGI-ACHGI-5 AEG-S AECI-8 AEG-C AEGI-C

TABLE V 3-4 PWR LARGE LOCA ACCIDENT SEQUENCES vs. RELEASE CATEGORIES

	Core melt No core melt								
[1	Release Cate	egories			
1	1	2	3	4	5	6	7	8	9
[Dominant L	arge LOCA	Accident Sec	quences Wit	Point Esti	imates	
	AB-a 1x10-11	AB-Y 1x10-10	AD-a 2x10 ⁻⁸	ACD-8 1x10-11	AD-8 4x10-9	AB-c 1x109	AD-e 2x10-6	A-B 2 x10-7	A 1x10 ⁻⁴
	AF-a 1x10-10	AHF-Y_11 2x10	AH-a 1x10 ⁻⁸		AH-β 3x10-9	ADF- 8-10 2x10-10	AH-ε 1x10 ⁻⁶	-	
1	ACD-a 5x10-11	AB-\$ 4x 10 ⁻¹¹	AF-8 1x10 ⁻⁸			AHF-ε 1x10-10			
	AG-a 9x10-11		AG-8 9x10-9						
				Other La	rge LOCA Act	cident Segu	encės		
ł		T	l						
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ΑΣ <mark>(a)</mark>	3 x 10 ⁻¹⁰	2 x 10 ⁻¹⁰	5 x 10 ⁻⁸	1 x 10 ⁻¹¹	7 x 10 ⁻⁹	1 x 10 ⁻⁹	AEG-e AEGI-e 3 x 10 ⁻⁶	² x 10 ⁻⁷	1 x 10 ⁻⁴

NUREG-075/014 (WASH-1400), "Reactor Safety Study: An Assessment of Accident Risks in U.S. Commercial Nuclear Power Plants," October 1975

2 x 10⁻⁷

3 x 10

1 x 10⁻⁴



"Worst case" is addressed in the planning

"Regulation does not require dedication of resources to handle every possible accident that can be imagined. The concept of the regulation is that there should be core planning with sufficient planning flexibility to develop reasonable response to those very serious low probability accidents which could affect the public."



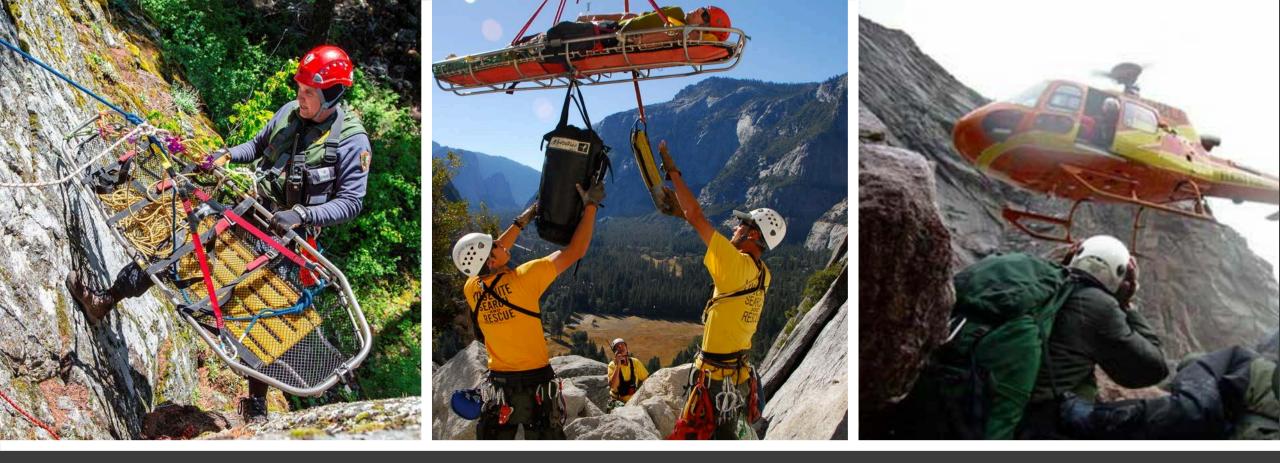
Capabilities are available at Federal level

"The Task Force believes that it *is not appropriate* to develop specific plans for the most severe and most improbable Class 9 events."

"The Task Force, however, does believe that consideration should be given to the *characteristics* of Class 9 events in *judging* whether emergency plans based primarily on smaller accidents can be expanded to cope with larger events."

"The planning basis recommended by the Task Force therefore includes some of the key characteristics of very large releases to assure that *site specific capabilities could be effectively augmented* with general emergency preparedness (response) resources of the Federal government should the need arise."

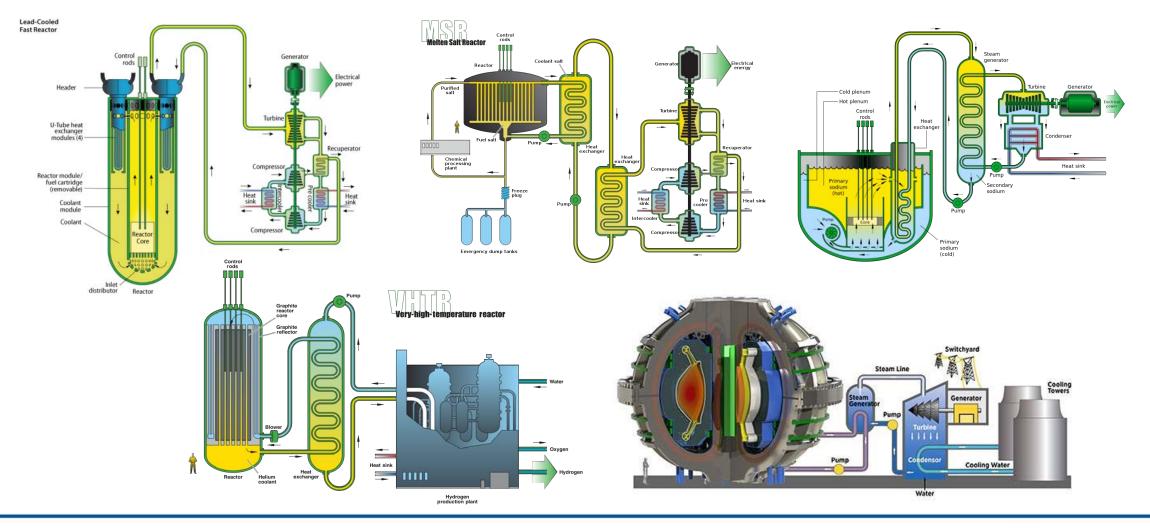




Response capabilities are coordinated across levels

Licensee State Federal

Different facilities require different capabilities

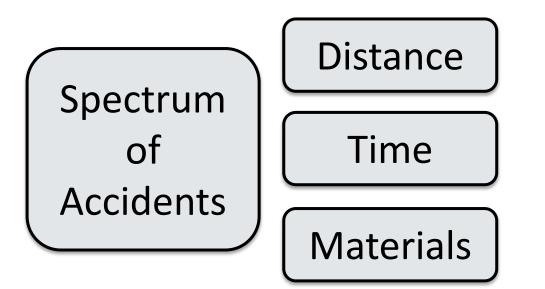




The planning basis informs EP planning functions

Ensure capabilities exist to detect, classify, notify, assess, mitigate, and effectively respond to an emergency

Planning Basis



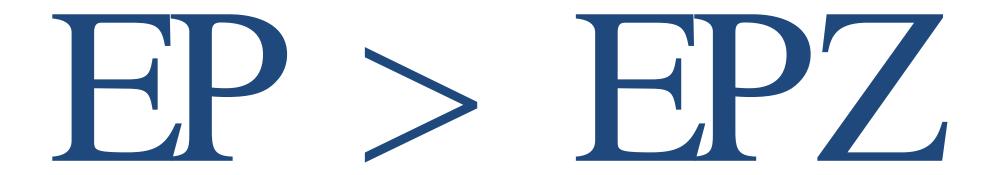
Emergency Planning Needs and Functions

EPZ size, exposure pathways, protective action strategies

Timeliness of classification and notification, protective action strategies, mitigation

Detection and assessment capabilities, radiological protection, mitigation







Key Takeaways Emergency Preparedness...

- EP ensures protective actions can and will be taken
- The EP planning basis is valid and useful
- EP regulations are risk-informed and commensurate to the potential hazards presented by the class of facility
 - Informed by characteristics of a spectrum of accidents
 - Evidence-based
 - Contain built-in conservatisms
 - Emphasize capabilities

