

RS-23-076

June 21, 2023

10 CFR 50.4(b)(6)  
10 CFR 50.71(e)  
10 CFR 54.37(b)U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20055

Dresden Nuclear Power Station, Units 1, 2, and 3  
Facility Operating License Nos. DRP-2, (Renewed) DPR-19 and DPR-25  
NRC Docket Nos. 50-010, 50-237, and 50-249

Subject: Dresden Updated Final Safety Analysis Report (UFSAR), Revision 15 and Fire Protection Report (FPR), Amendment 24

In accordance with the requirements of 10 CFR 50.71, "Maintenance of records, making of reports," Paragraph (e)(4), Constellation Energy Generation, LLC (CEG) submits Revision 15 to the Updated Final Safety Analysis Report (UFSAR) for Dresden Nuclear Power Station, Amendment 24 to the Fire Protection Report (FPR), summaries of evaluations conducted pursuant to 10 CFR 50.59, "Changes, tests, and experiments," 10 CFR 54.37(b), "Additional records and recordkeeping requirements," and NEI 99-04, "Guidelines for Managing NRC Commitment Changes."

The UFSAR is being submitted on Optical Storage Media (OSM) in its entirety, including documents incorporated by reference (Technical Specifications Bases, Technical Requirements Manual (TRM) and FPR). UFSAR pages changed as a result of this update are delineated with "Rev. 15 – June 2023" in the page header. FPR pages changed as a result of this update are delineated with "Amendment 24" in the page header.

One (1) OSM is included in this submission. The OSM labeled, "Constellation – Dresden Nuclear Power Station UFSAR Revision 15, June 2023" contains the following four components:

- 001 DRE UFSAR REV 15.pdf, 96.0 megabytes (MB)
- 002 DRE FPR AMEND 24.pdf, 189 MB
- 003 DRE TRM.pdf, 3.45 MB
- 004 DRE TECH SPEC BASES.pdf, 2.17 MB

A053  
A006  
NMSS01  
NRR  
NMSS

Attachment A provides a brief summary of the changes incorporated into UFSAR Revision 15.

Attachment B provides a brief summary of the changes incorporated into FPR Amendment 24.

Attachment C provides the summary report pursuant to 10 CFR 50.59(d)(2).

Attachment D provides a summary of changes to the TRM.

Attachment E provides a summary of regulatory commitment changes.

Attachment F provides the summary report pursuant to 10 CFR 54.37(b).

Attachment G contains the directory path, filename, and size of each individual file.

As required by 10 CFR 50.71(e)(2)(i), I, Director – Licensing, certify that to the best of my knowledge, the information contained in the enclosure and attachments to this letter accurately reflect information and analyses submitted to the NRC or prepared pursuant to NRC requirements, and changes made under the provisions of 10 CFR 50.59.

There are no commitments made in this document. Should you have any questions concerning this letter, please contact Amy Hambly at 630-657-2808:

Respectfully,

**Gullott, David**  
**M.**

Digitally signed by Gullott,  
David M.  
Date: 2023.06.21 11:14:55  
-05'00'

David M. Gullott  
Director - Licensing  
Constellation Energy Generation, LLC

Enclosure: OSM – Dresden Nuclear Power Station UFSAR, Revision 15, FPR Amendment 24

Attachments: Attachment A, "UFSAR Revision 15 Change Summary Report"  
Attachment B, "FPR Amendment 24 Change Summary Report"  
Attachment C, "10 CFR 50.59 Evaluation Summary Report"  
Attachment D, "Technical Requirements Manual Change Summary Report"  
Attachment E, "Summary of Regulatory Commitment Changes"  
Attachment F, "10 CFR 54.37(b) Aging Management Review Summary"  
Attachment G, "OSM Directory Structure"

cc: Regional Administrator – NRC Region III  
NRC Senior Resident Inspector – Dresden Nuclear Power Station  
NRC Project Manager, NRR – Dresden Nuclear Power Station

## Attachment A

### Dresden Nuclear Power Station

#### UFSAR Revision 15 Change Summary Report

##### 19-003 and 19-005 Revision to UFSAR Sections 10.3 and 11.2

Revised Sections 10.3.2, 11.2.2.1.3, 11.2.2.26, and 11.2.2.2.29 to reflect mechanically isolating the Maximum Recycle Reboilers per Engineering Change (EC) 627095 and EC 627096.

##### 19-011 Revision to UFSAR Sections 6.2 and 1.2

Revised Sections 6.2 and 12.2.2 to address Reactor Building roof replacement per EC 628992.

##### 20-008 Revision to UFSAR Section 9.1

Revised Section 9.1.3.2 to reflect installation of bypass switches for Fuel Pool cooling pump low suction pressure trips during certain plant evolutions per EC 631971 and EC 631972.

##### 21-001 Revision to UFSAR Section 6.2

Revised Table 6.2-9 primary containment isolation valve closing times related to Reactor Building Closed Cooling Water supply valves and Traversing Incore Probe ball valves.

##### 21-004 Revision to UFSAR Section 6.2

Revised Section 6.2.1.2.1.2 to reflect a Finite Element Analysis of the Unit 2 containment shell per EC 634241.

##### 21-005 Revision to UFSAR Section 8.3

Revised Table 8.3-1 to reflect new data for the replacement of the Unit 3 Main Power Transformer per EC 401945.

##### 21-006 and 21-007 Revision to UFSAR Section 4.6

Revised Section 4.6.3.3.2.1 to reflect change in filter rating of Unit 2 and Unit 3 Control Rod Drive system filter changes per EC 635376 and EC 635409

##### 21-008 Revision to UFSAR Section 2.2

Revised Sections 2.2.2.3 and 2.2.4 to incorporate Probabilistic Risk Assessment changes related to the change of using natural gas as the primary fuel for station heating boilers per EC 388091, EC 387722, EC 401837, and EC 404758.

##### 22-001 Revision to UFSAR Section 5.2

Revised Section 5.2.5 to reflect License Amendment related to TSTF-554-A, Revise Reactor Coolant Leakage Requirements, associated with NRC Safety Evaluation dated 01/13/2022.

##### 22-002 Revision to UFSAR Section 8.2

Revised Figure 8.2-1 to indicate the correct Transformer number shown on the diagram.

##### 22-003 Revision to UFSAR Section 6.2

Revised Section 6.2.6.1 to reflect updated ANSI/ANS standards and NEI technical report.

##### 22-004 Revision to UFSAR Section 2.2

Revised Sections 2.2.2.3 and 2.2.4, Table 2.2-6, and Figure 2.2-2 to reflect hazards associated with a natural gas pipeline to be installed near the station.

##### 22-008 Revision to UFSAR Section 8.2

Revised Figures 8.2-1 and 8.2-2 to reflect a reconfiguration of the offsite 345 kV transmission power sources and the addition of a new offsite substation.

## **Attachment B**

### **Dresden Nuclear Power Station**

#### **FPR Amendment 24 Change Summary Report**

##### **Revision Summary:**

Fire Protection Report Change Packages 21-002, 22-002 and 22-003 update Amendment 24 of the Fire Protection Report. The revisions associated with this update include:

##### **Volume 1 (Fire Hazards Analysis):**

1. Change 22-002: EC 637078 - Add epoxy-lined steel (SIPP) to the types of fire main yard loop piping in Section 2.4.1.2.
2. Change 22-003: IR 4525415 - Update Sections 2.4.3.3 and 4.11.1 to include CO2 Alterrex Systems in the exciter housings.

##### **Volume 2 (Safe Shutdown Report):**

1. Change 21-002: IR 4435186 - Revise Figure 3.1-2 and Section 6.2.3.2 of SSA to Reflect CRD Crosstie Configuration.
2. Change 22-003: IR 4525764 - Update Table 7.3-1, Section I.5 to allow CRD pump amperage up to 39 amps during emergency use

##### **Volume 4 (Interim Measures/Exemption Requests):**

1. Change 22-002: EC 637078 - Add the 86-10 technical evaluation to the end of Volume 4 as Tab P, and update the table of contents (p.IV-2-ii).

## Attachment C

### Dresden Nuclear Power Station

#### 10 CFR 50.59 Evaluation Summary Report

Safety Evaluation Number 2021-04-001

Type of Safety Evaluation: Modification

Evaluation Reference Number: EC 629134, Revision 0

Title: Dresden EPG/SAG TSG (Appendix A / Appendix C) Calculations

#### Description:

This activity revises Dresden Operating Procedures (DEOPs) and Severe Accident Management Guidelines (SAMGs) based upon the generic Boiling Water Reactor Owners Group (BWROG) Emergency Procedures Guidelines/Severe Accident Guidelines (EPGs/SAGs) Revision 4, issued in June 2018. This activity also issues station calculation revisions associated with the BWROG EPG/SAG Revision 4, Appendix A and Appendix C.

The BWROG EPGs/SAGs provide generic symptomatic direction for BWR emergency response and severe accident mitigation. The guidelines were developed by the BWROG in response to NUREG-0737 Item I.C.1, "Guidance for the Evaluation and Development of Procedures for Transients and Accidents," and NEI 91-04, "Severe Accident Issue Closure Guidelines," Revision 1, Section 5.

A 10 CFR 50.59 Screening determined many changes being made during the proposed activity screened out, except the following:

- Evaluation of the change in manual operation of Automatic Depressurization System (ADS) and due to the cooldown rate in excess of Technical Specification (TS) cooldown rate being performed.
- Evaluation of the 1996 Anticipated Transient Without Scram (ATWS) Safety Evaluation Report (SER).
- Evaluation to address any adverse effect on ATWS strategies.

The 50.59 evaluation determined:

1. The changes to procedure developed from EPG/SAG Revision 4 provide direction for operator action as the results of transients or accidents, they do not cause any transient or accident, therefore, they cannot cause an increase in the frequency of occurrence of an accident evaluated in the UFSAR.
2. The ability to bypass interlocks and isolations is shown to not be required unless conditions have placed the unit outside of design basis conditions. Consequently, the change in these procedures do not result in more than a minimum increase in the frequency of occurrence of a malfunction of an SCC important to safety previously evaluated in the UFSAR.

## Attachment C

### Dresden Nuclear Power Station

#### 10 CFR 50.59 Evaluation Summary Report

3. The changes to procedures developed from EPG/SAG Rev. 4 do not result in more than a minimum increase in the consequences of an accident previously evaluated because manual actuation of ADS prior to the automatic actuation assumed in the safety analysis for the limiting Small Break Loss of Coolant Accident (SBLOCA) occurs at a higher Reactor Pressure Vessel (RPV) water level (between Top of Active Fuel (TAF) and Minimum Steam Cooling RPV Water Level (MSCRWL)) than does the analysis.
4. The changes to procedures developed from EPG/SAG Rev. 4 do not result in more than a minimum increase in the consequences of a malfunction of an SCC important to safety previously evaluated because manual actuation of ADS instead of the automatic action, while substituting a manual action for an automatic actuation, causes only a minimal increase in the consequences of ADS not actuating. The operators are trained that this action is of high importance. Procedures are written to be followed. The actual change from EPG/SAG Rev. 3 to Rev. 4 ensures the operators are implementing manual ADS sooner than previously (EPG/SAG Revision 3).
5. The procedure changes from implementing EPG/SAG Rev. 4 are used to respond to accidents but do not create possibility for accidents of different types because the actions are only implemented in response to symptoms from the as-found conditions.
6. The procedure changes from implementing EPG/SAG Rev. 4 are used to respond to accidents but do not create the possibility of a malfunction of an SCC important to safety with different results. The results of operator actions should be to mitigate an event. The results of consequence do not change- i.e., impact to health and safety of the public is the ultimate result. These procedures are in place to help operators mitigate any such events.
7. The procedure changes from implementing EPG/SAG Rev. 4 do not result in a design basis limit for a fission product barrier being exceeded or altered. The procedures developed from EPG/SAG Rev. 4 seek to limit PCT to 1500°F or less with allowance to rise to 1800°F under certain conditions. The UFSAR seeks to limit peak cladding temperature (PCT) to <2200°F. The actions specified seek to maintain fission product barriers (RPV, Primary Containment, Secondary Containment) and minimize offsite dose rates (Radioactivity Release Guideline).
8. The proposed activity revises the DEOPs which provide procedural guidance to operators for emergency conditions. The procedures developed from EPG/SAG Revision 4 do not affect the methods of evaluation described in the UFSAR in establishing the design bases or the safety analyses. The Appendix C calculations for the Action limits and curves are not described in the UFSAR. Calculations supporting the DEOPs are also revised by this activity. These calculation revisions do not involve revising or replacing an UFSAR described evaluation methodology that is used in establishing the design bases or used in the safety analyses.
9. The proposed activity are changes to the DEOPs and comply with the requirements of TS 5.4.1.b. The purpose of DEOPs is to return the plant to tech spec conditions when an event takes the plant outside of Technical Specifications. Further, the revised DEOPs continue to use Technical Specification and Operating License values as written and do not create a situation where they would require revision. Therefore, no change to Technical Specifications is required.

Based upon the results of this evaluation, implementing the activity per governing procedures may be performed without prior NRC approval.

## Attachment C

### Dresden Nuclear Power Station

#### 10 CFR 50.59 Evaluation Summary Report

Safety Evaluation Number 2023-01-001

Type of Safety Evaluation: Modification

Evaluation Reference Number: EC 395525, Revision 8 and EC 397957, Revision 7

Title: Replace HPCI Signal Converter and Flow Indicating Controller

#### Description:

This activity replaces the High-Pressure Coolant Injection (HPCI) Signal Converter panel with a functionally equivalent design. The activity also installed electrical components to facilitate use of either the existing or the new signal converter. The additional fuses and switches were not part of the original design. These new connections required further evaluation and all other changes of the activity were screened out.

#### The 50.59 Evaluation determined:

The adverse changes noted above (related to the installation of the power switch and fuse) were reviewed in the Evaluation and found to be acceptable based on the quality assurance program utilized as part of the work package development, the quality control program which inspected the work, and the governance which assure that such new control switches are adequately controlled and that mispositioning is prevented. Further, extensive testing of the new circuit including checking of connections and post-installation testing to assure adequate performance that meets the design requirements is also conducted.

1. The activity does not introduce the possibility of a change in the frequency of occurrence of any previously evaluated accident because the HPCI system is not an initiator of any accidents nor does this change have any direct effect on the scenarios leading to those accidents as this is an electrical change limited to the HPCI flow control logic. These changes result in no increase in the frequency of occurrence of accidents previously evaluated.
2. As the new components continue to uphold the same functional requirements prescribed by codes and standards as the rest of the HPCI Signal Converter equipment, the additional components do not result in more than a minimal increase in a power failure or a malfunction.
3. The radiological consequence of accidents which credit the HPCI system operation are not altered as result of the addition of the switch and fuse. The radiological consequence of an accident in which a the HPCI system is not available is not altered because the addition of the switch and fuse is bounded by the existing failure modes of HPCI. The new control switch and fuse is physically independent from the backup system, Automatic Depressurization System (ADS). There is no increase in the consequences of an accident.
4. The activity does not introduce the possibility of a change in the consequences of a malfunction because a failure of the new fuse and new control switch is bounded within the failure of the overall HPCI system.

## Attachment C

### Dresden Nuclear Power Station

#### 10 CFR 50.59 Evaluation Summary Report

5. Failure of the switch and/or fuse (loss of power to the control circuit) is bounded by existing HPCI failure modes. Failure of the control switch and/or fuse (loss of power to the HPCI control circuit) will not impact ADS or any other accident mitigation system due to the electrical independence. There is no possibility of an accident of a different type than is already analyzed in the UFSAR.

6. Failure of the control switch and/or fuse (loss of power to the control system) does not introduce a failure mode that is not bounded by those described in the HPCI System in UFSAR Section 7.3.1.3. Installation of the fuse and the switch do not change how HPCI responds to an accident. There is no possibility for a malfunction of an SSE important to safety with a different result than any previously evaluated in the UFSAR.

7. The activity does not result in a change that would cause any system parameter to change and does not result in a Design Basis Limit for a Fission Product Barrier (DBLFPB) as described in the UFSAR being exceeded or altered.

8. The activity does not involve a method of evaluation. The activity does not result in a departure from a method of evaluation described in the UFSAR used in establishing the design bases or in the Safety Analyses.

9. The proposed activity does not impact any Technical Specification, Technical Specification Bases, or Technical Requirements Manual requirements for any of the systems impacted by this activity. There is no change required to the operating license.

Based upon the results of this evaluation, implementing the activity per governing procedures may be performed without prior NRC approval.



## Attachment D

### Dresden Nuclear Power Station

#### Technical Requirements Manual Change Summary Report

##### 20-008 Revision to TRM Section 3.5.a and Bases

Revised TRM to incorporate use of Technical Specification LCO 3.0.10 for TRM LCO 3.5.a, Required Action B.1 Completion Time. Reference NRC Safety Evaluation dated July 10, 2020.

##### 20-010 Revision to TRM Section 3.7.l and Bases

Revised TRM to reflect fire hose removal from fire hose stations and replaced by high-rise packs which are staged on the fire brigade carts. Reference Engineering Change 633230.

##### 21-001 Revision to TRM Sections 3.7.k and 3.7.n

Revised TRM to add actions linking functionality of fire barriers to the functionality of gaseous suppression systems.

##### 21-002 Revision to TRM Section 3.7.n

Revised TRM Required Actions numbering and enhanced the OR/AND conjunctions to ensure the impact review of gaseous suppression systems prior to subsequent compensatory actions on nonfunctional fire barrier(s).

##### 21-003 Revision to Appendix D and Appendix G

Revised TRM Appendices by adding Operations review signature for Technical Specification Bases and TRM Changes.

##### 21-009 Revision to TRM Appendix E

Updated Appendix E to include Dresden 2 COLR Revision 19 for D2C28.

##### 22-002 Revision to TRM Appendix G

Revised TRM by removing the requirement of providing TRM changes to other Constellation stations since TRMs and TRM change packages are accessible to all sites within the fleet.

##### 22-003 Revision to TRM Section 3.3.a

Revised TRM to incorporate changing the frequency of TRM Surveillance Requirement 3.3.a.2 for Functions 1.a, 1.b, and 1.c.

##### 22-004 Revision to TRM Appendix F

Updated Appendix F to include Dresden 3 COLR Revision 18 for D3C28.

##### Revisions to TRM Appendix I as part of the Surveillance Frequency Control Program (SFCP)

22-001: TS SR 3.3.5.2.2 is revised based on License Amendments 275/268 (Unit 2/3), NRC Safety Evaluation dated December 7, 2021.

## Attachment E

### Dresden Nuclear Power Station

#### Summary of Regulatory Commitment Changes

In accordance with the guidance provided in Nuclear Energy Institute (NEI) 99-04, Revision 0, "Guidelines for Managing NRC Commitment Changes," dated July 1999, Dresden Nuclear Power Station is submitting a summary of commitments revised that can be changed with notification to the NRC submitted along the FSAR updates as required by 10 CFR 50.71(e).

No commitment changes were processed requiring NRC notification during the period of June 1, 2021, through May 31, 2023.

There were three late performances of regulatory commitments.

Commitment Number: RCMT 101522-45-18

#### Source/Original Documents:

1. License Renewal, Dresden Units 2 and 3 License Renewal Application, Appendix A, dated 01/03/2003 (ML030090655)
2. Additional Information for the Review of the License Renewal Applications, dated 10/03/2003 (ML032820273)
3. Additional Information for the Review of the License Renewal Applications, dated 01/26/2004 (ML040340487)
4. NUREG-1796, SER License Renewal of Dresden Units 2 and 3, dated 10/31/2004 (ML043060582 and ML043060584)

#### Comments:

During 2nd quarter 2022, an oil sample was unable to be obtained from the Unit 3 High Pressure Coolant Injection (HPCI) turbine oil system due to the condition documented in Issue Report (IR) 04506202 and IR 4507313. This was a missed Regulatory Commitment of performing quarterly HPCI turbine lube oil reservoir sample.

The oil sample was completed during the 3rd quarter 2022 HPCI surveillance.

**Attachment E**

**Dresden Nuclear Power Station**

**Summary of Regulatory Commitment Changes**

Commitment Number: RCMT 101522-52-02 and RCMT 101522-45-39

Source/Original Document:

1. License Renewal, Dresden Units 2 and 3 License Renewal Application, Appendix A, dated 01/03/2003 (ML030090655)
2. License Renewal, Dresden Units 2 and 3 License Renewal Application, Appendix B-F, dated 01/03/2003 (ML030090341)
3. NUREG-1796, SER License Renewal of Dresden Units 2 and 3, dated 10/31/2004 (ML043060582 and ML043060584)

Comments:

During the 2nd half of 2022, the FatiguePro monitoring report was unable to be performed due to the condition documented in IR 4550786. This was a missed Regulatory Commitment of performing the semi-annual FatiguePro monitoring report.

The software issue resolution is in progress and the FatiguePro monitoring report is scheduled to be completed during 2nd quarter 2023 (WO 5289511).

Commitment Number: RCMT 101522-45-28

Source/Original Document:

1. License Renewal, Dresden Units 2 and 3 License Renewal Application, Appendix A, dated 01/03/2003 (ML030090655)
2. Additional Information for the Review of the License Renewal Applications, dated 10/03/2003 (ML032820273)
3. Additional Information for the Review of the License Renewal Applications, dated 01/26/2004 (ML040340487)
4. NUREG-1796, SER License Renewal of Dresden Units 2 and 3, dated 10/31/2004 (ML043060582 and ML043060584)

Comments:

During 1st quarter 2023, an oil sample was unable to be obtained from the Unit 2 HPCI booster bearing housing due to the condition documented in IR 4559886. This was a missed Regulatory Commitment of performing the two-year HPCI booster bearing housing oil sample.

HPCI booster bearing housing oil sample is scheduled to be completed at the next available opportunity, during the 2nd quarter 2023 HPCI surveillance (WO 5346172).

## **Attachment F**

### **Dresden Nuclear Power Station**

#### **10CFR 54.37(b) Aging Management Review Summary**

In accordance with 10 CFR 54.37(b) and the guidance specified in Regulatory Issue Summary 2007-16, Revision 1, "Implementation of the Requirements of the 10 CFR 54.37(b) for Holders of Renewed Licenses," the UFSAR update required by 10 CFR 50.71 must include any Structures, Systems or Components (SSCs) newly identified that would have been subject to an aging management review or evaluation of time-limited aging analyses in accordance with 10 CFR 54.21. This UFSAR update must describe how the effects of aging will be managed such that the intended function(s) in 10 CFR 54.4(b) will be effectively maintained during the period of extended operation.

The 10 CFR 54.37(b) review of changes to the plant Current Licensing Basis covered the period of April 1, 2021 to April 1, 2023. The review included:

- Engineering Changes that were installed or completed since the last UFSAR update,
- UFSAR pending change descriptions and revised text, and
- NRC Interim Staff Guidelines (ISGs) related to license renewal.

No "newly identified" SSCs were identified that require aging management or evaluation of Time Limiting Aging Analyses (TLAA's) in accordance with the License Renewal Rule. Therefore, there are no associated updates required to the Dresden UFSAR described Aging Management Programs.

**Attachment G**

**Dresden Nuclear Power Station**

**OSM Directory Structure**

<b>Directory Path</b>	<b>File Name</b>	<b>Size</b>
D:\001 DRE UFSAR REV 15	000 Cover and List of Affected Pages.pdf	156 KB
D:\001 DRE UFSAR REV 15	001 CHAP 01 Introduction.pdf	426 KB
D:\001 DRE UFSAR REV 15	002 CHAP 02 Site Characteristics.pdf	1457 KB
D:\001 DRE UFSAR REV 15	003 CHAP 03 Design of SCE.pdf	8912 KB
D:\001 DRE UFSAR REV 15	004 CHAP 04 Reactor.pdf	3923 KB
D:\001 DRE UFSAR REV 15	005 CHAP 05 Reactor Coolant Sys.pdf	2937 KB
D:\001 DRE UFSAR REV 15	006 CHAP 06 Engineered Safety Features.pdf	9581 KB
D:\001 DRE UFSAR REV 15	007 CHAP 07 Instrumentation and Controls.pdf	6912 KB
D:\001 DRE UFSAR REV 15	008 CHAP 08 Electrical Power.pdf	4035 KB
D:\001 DRE UFSAR REV 15	009 CHAP 09 Auxiliary Systems.pdf	5134 KB
D:\001 DRE UFSAR REV 15	010 CHAP 10 Steam and Power Conv Sys.pdf	396 KB
D:\001 DRE UFSAR REV 15	011 CHAP 11 Radioactive Waste Mgmt.pdf	8075 KB
D:\001 DRE UFSAR REV 15	012 CHAP 12 Radiation Protection.pdf	41375 KB
D:\001 DRE UFSAR REV 15	013 CHAP 13 Conduct of Operations.pdf	203 KB
D:\001 DRE UFSAR REV 15	014 CHAP 14 Initial Test Program.pdf	596 KB
D:\001 DRE UFSAR REV 15	015 CHAP 15 Accident and Transient Analysis.pdf	3880 KB
D:\001 DRE UFSAR REV 15	016 CHAP 16 Technical Specifications.pdf	44 KB
D:\001 DRE UFSAR REV 15	017 CHAP 17 Quality Assurance.pdf	53 KB
D:\001 DRE UFSAR REV 15	018 APPENDIX A UFSAR Supplement.pdf	301 KB
D:\002 DRE FPR AMEND 24	000 FPR List of Affected Pages	116 KB
D:\002 DRE FPR AMEND 24	001 FPR VOL 1 Updated Fire Hazards Analysis.pdf	4841 KB
D:\002 DRE FPR AMEND 24	002 FPR VOL 2 Safe Shutdown Analysis.pdf	7675 KB
D:\002 DRE FPR AMEND 24	003 FPR VOL 3 Part 1 of 2.pdf	33371 KB
D:\002 DRE FPR AMEND 24	004 FPR VOL 3 Part 2 of 2.pdf	37109 KB
D:\002 DRE FPR AMEND 24	005 FPR VOL 4.pdf	87326 KB
D:\002 DRE FPR AMEND 24	006 FPR VOL 5.pdf	23917 KB
D:\003 DRE TRM	001 DRE TRM.pdf	3539 KB
D:\004 DRE TECH SPEC BASES	001 DRE Tech Spec Bases.pdf	2227 KB