



South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

January 26, 2023  
NOC-AE-23003939  
10 CFR 50.90  
STI: 35424982

ATTN: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

South Texas Project  
Units 1 & 2  
Docket Nos. STN 50-498, STN 50-499  
Supplement to Application to Revise Technical Specifications to Adopt  
TSTF-554, "Revise Reactor Coolant Leakage Requirements" (EPID: L-2021-LLR-0075)

References:

1. Letter from K. Harshaw to Document Control Desk; "South Texas Project Units 1 & 2 Docket Nos. STN 50-498, STN 50-499 Application to Revise Technical Specifications to Adopt TSTF-554, "Revise Reactor Coolant Leakage Requirements;" August 9, 2022; NOC-AE-22003908; ML22221A212.
2. Letter from K. Harshaw to Document Control Desk; "South Texas Project Units 1 & 2 Docket Nos. STN 50-498, STN 50-499 Supplement to Application to Revise Technical Specifications to Adopt TSTF-554, "Revise Reactor Coolant Leakage Requirements" (EPID: L-2021-LLR-0075);" September 1, 2022; NOC-AE-22003913; ML22244A232.

STP Nuclear Operating Company (STPNOC) is supplementing the License Amendment Request (LAR) that was submitted via Reference 1 and supplemented via Reference 2. In Reference 2, a typographical error was introduced in the clean and marked up Technical Specification page for page 3/4 4-20. This supplement corrects that typographical error and does not alter the technical content of the LAR.

This letter contains no new regulatory commitments.

Should you have any questions regarding this submission, please contact Zachary Dibbern at (361) 972-4336 or me at (361) 972-4778.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on

1/26/2023

Kimberly Harshaw  
Executive VP and CNO

Attachments:

1. Corrected Proposed Technical Specification Changes (Markup) for 3/4 4-20
2. Corrected Revised Technical Specification Page for 3/4 4-20

cc:

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U.S. Nuclear Regulatory Commission  
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**Attachment 1**

**Corrected Proposed Technical Specification Changes (Markup) for 3/4 4-20**

## REACTOR COOLANT SYSTEM

### OPERATIONAL LEAKAGE

#### LIMITING CONDITION FOR OPERATION

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3.4.6.2 Reactor Coolant System operational leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE,
- b. 1 gpm UNIDENTIFIED LEAKAGE,
- c. 150 gallons per day of primary-to-secondary leakage through any one steam generator,
- d. 10 gpm IDENTIFIED LEAKAGE from the Reactor Coolant System, and
- e. 0.5 gpm leakage per nominal inch of valve size up to a maximum of 5 gpm at a Reactor Coolant System pressure of  $2235 \pm 20$  psig from any Reactor Coolant System Pressure Isolation Valve specified in Table 3.4-1.\*

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, isolate the affected component, pipe, or vessel from the RCS by use of a closed manual valve, closed and de-activated automatic valve, blind flange, or check valve within 4 hours or ~~or with primary-to-secondary leakage not within limit,~~ be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With Reactor Coolant System operational UNIDENTIFIED or IDENTIFIED LEAKAGE greater than the above limits, reduce leakage to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With primary-to-secondary leakage not within the limit, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- ed. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

\*Test pressures less than 2235 psig but greater than 150 psig are allowed. Observed leakage shall be adjusted for the actual test pressure up to 2235 psig assuming the leakage to be directly proportional to pressure differential to the one-half power.

**Attachment 2**

**Corrected Revised Technical Specification Page for 3/4 4-20**

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