UFSAR Figure: 14.2.5-1 | Unit: 2

Title: **Variation of Reactivity with Core Temperature at 1050 psia for the End of Life Rodded Core with One Control Rod Assembly Stuck (Assumes Zero Power)**

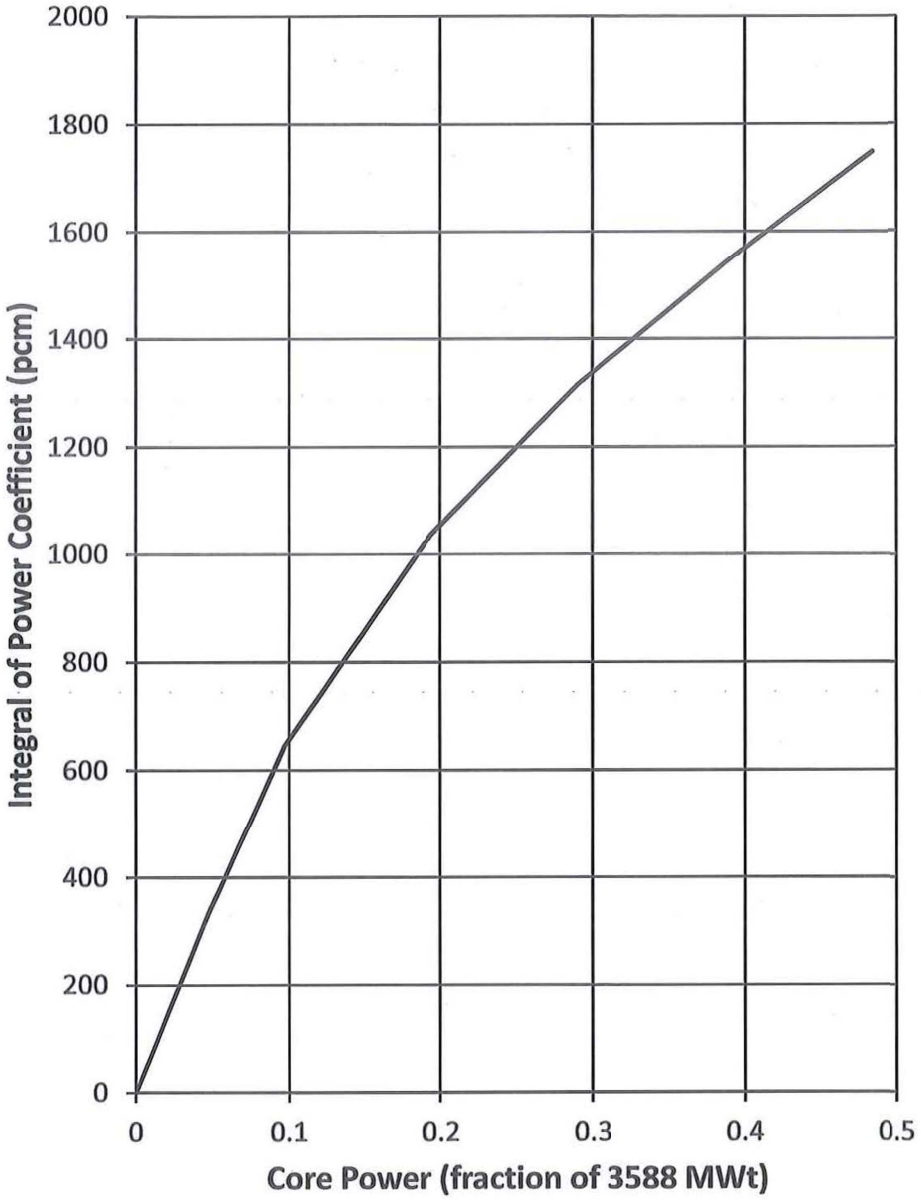


Figure 14.2.5-2: Doppler Power Feedback for Steamline Break

UFSAR Figure: 14.2.5-2	Unit: 2

Title: **Doppler Power Feedback for Steam Line Break**

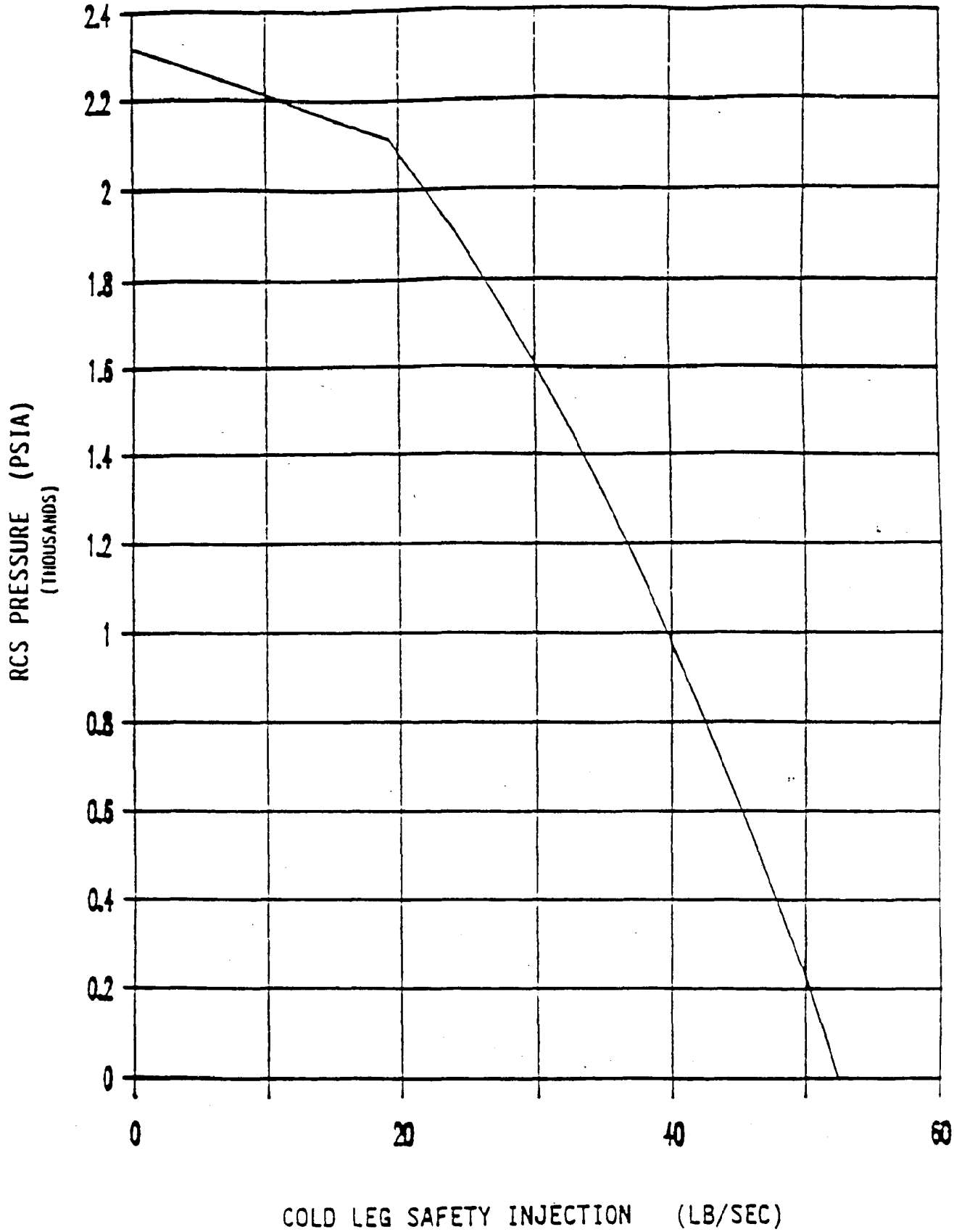
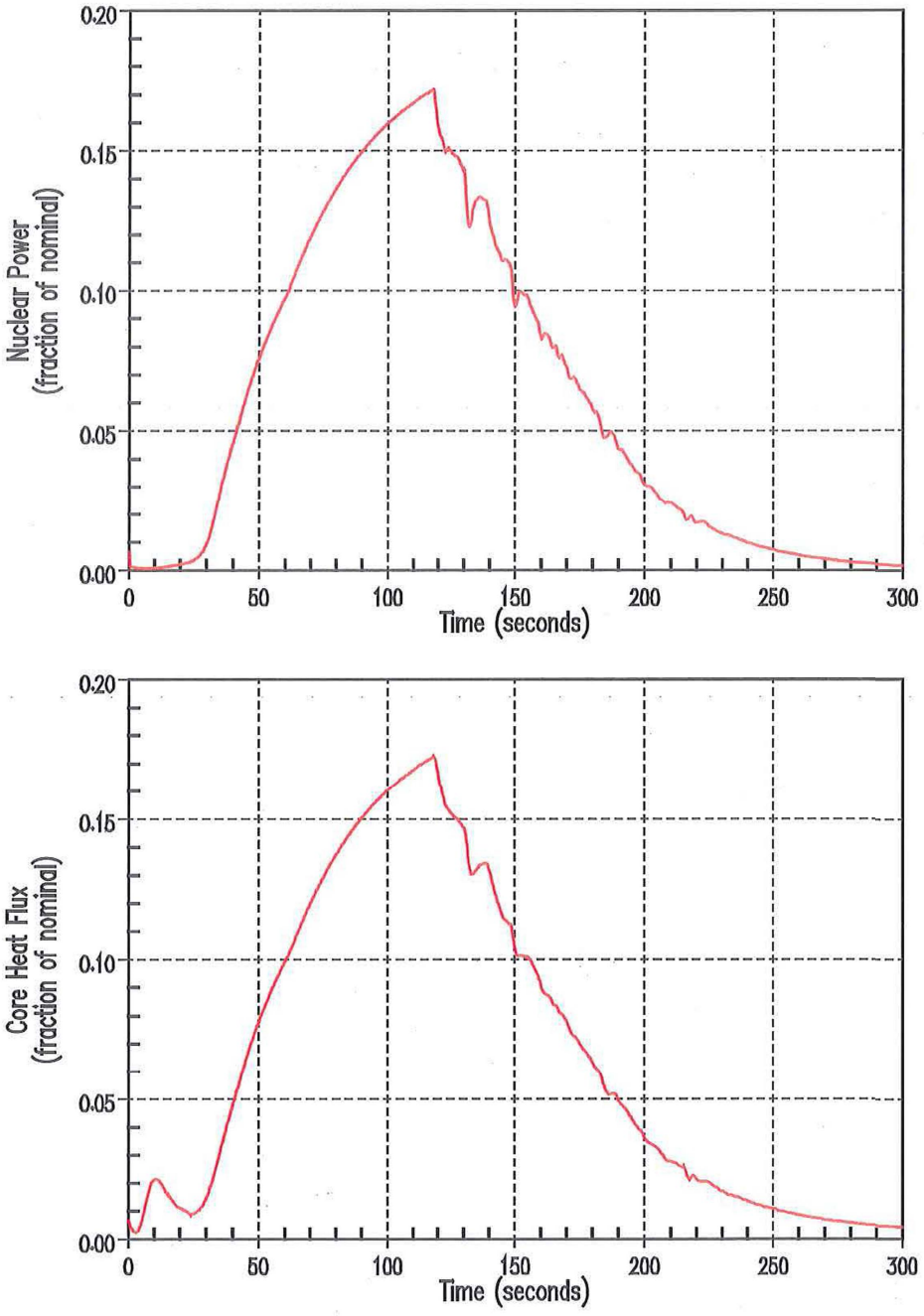
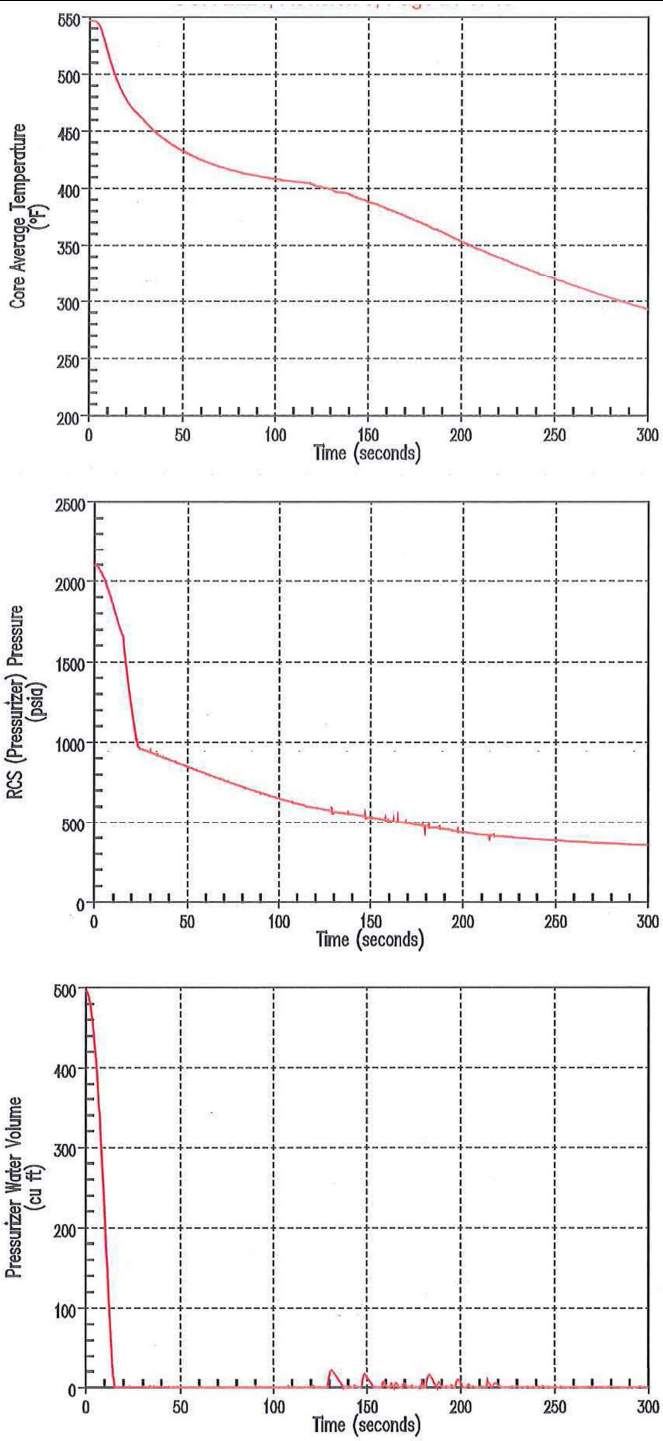


Figure 14.2.5-3 Safety Injection Flow Supplied by One Charging Pump



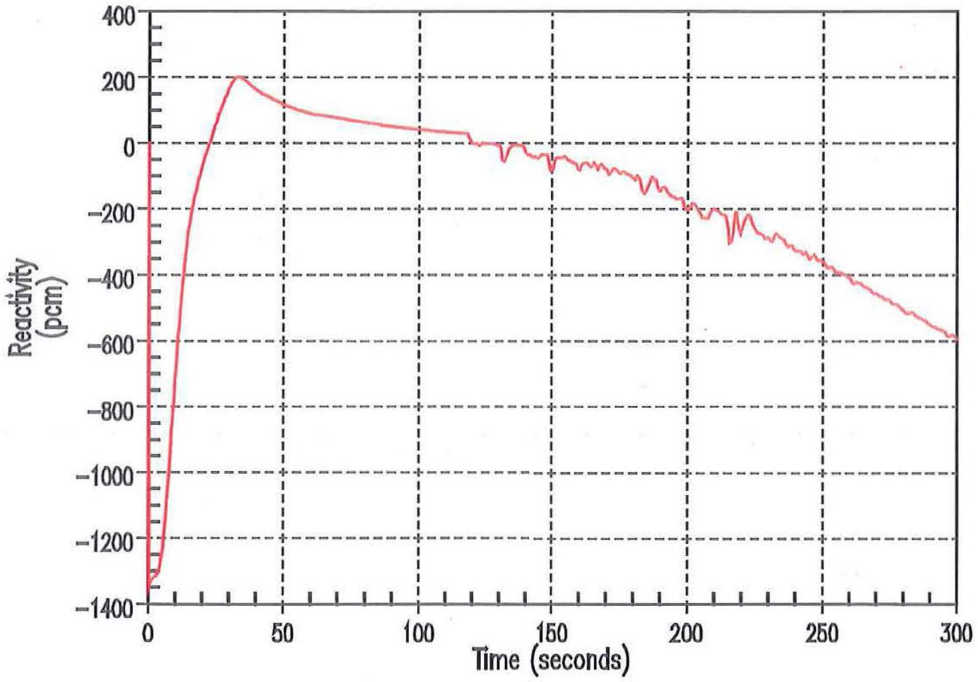
UFSAR Figure: 14.2.5-4 | Unit: 2

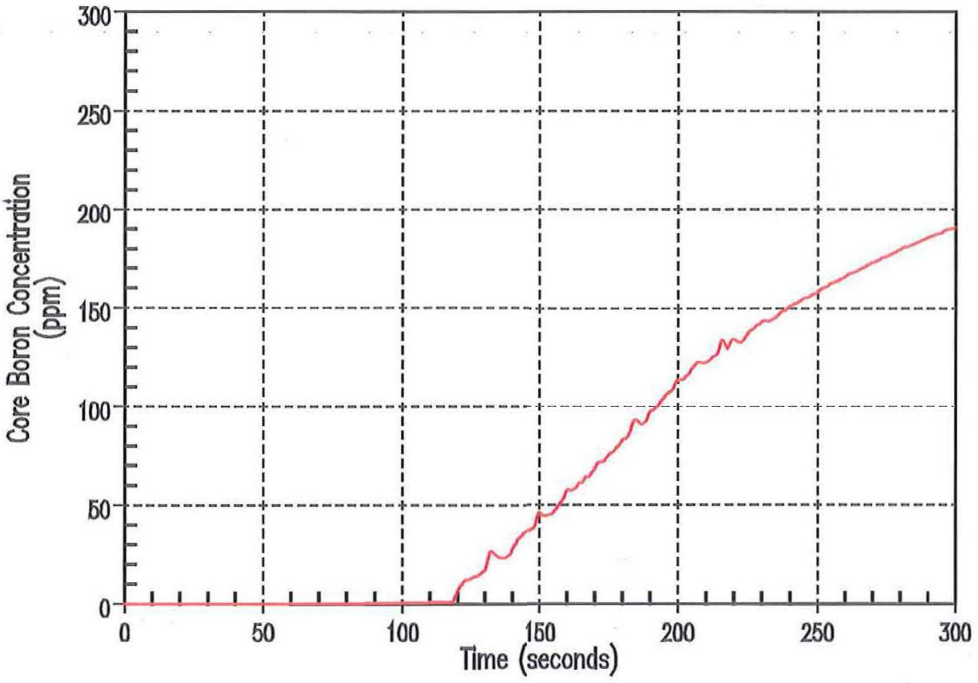
Title: **Steam Line Break DER Inside Containment with Power Nuclear Power and Core Heat Flux Versus Time**



UFSAR Figure: 14.2.5-5 | Unit: 2

Title: **Steam Line Break DER Inside Containment with Power Core Average Temperature, RCS Pressure, and Pressurizer Water Volume Versus Time**





UFSAR Figure: 14.2.5-6 | Unit: 2

Title: **Steam Line Break DER Inside Containment with Power Reactivity and Core Boron Concentration Versus Time**

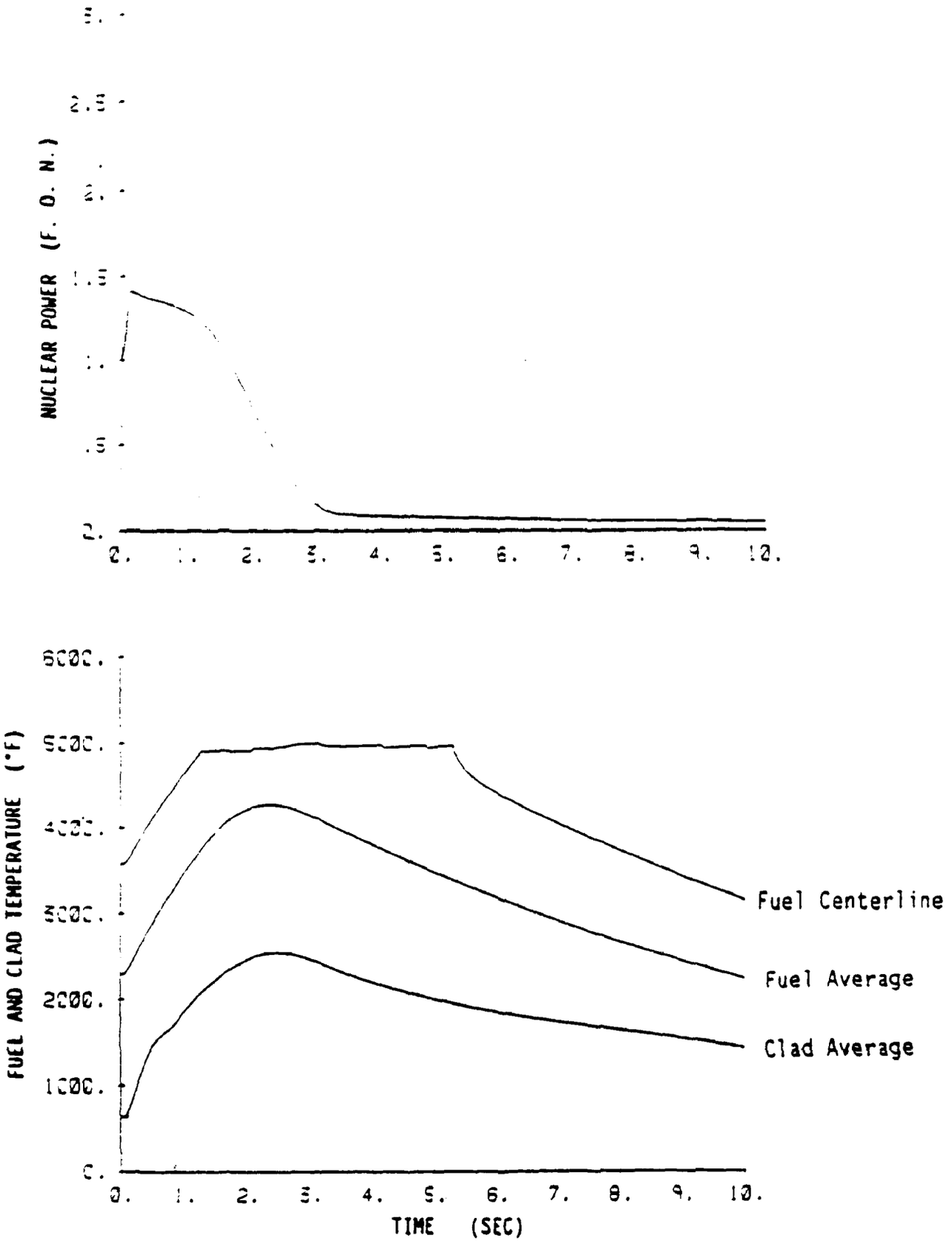


Figure 14.2.6-1 Rod Ejection
Nuclear Power and Fuel Clad Temperature Versus Time for Hot Full Power at Beginning of Life

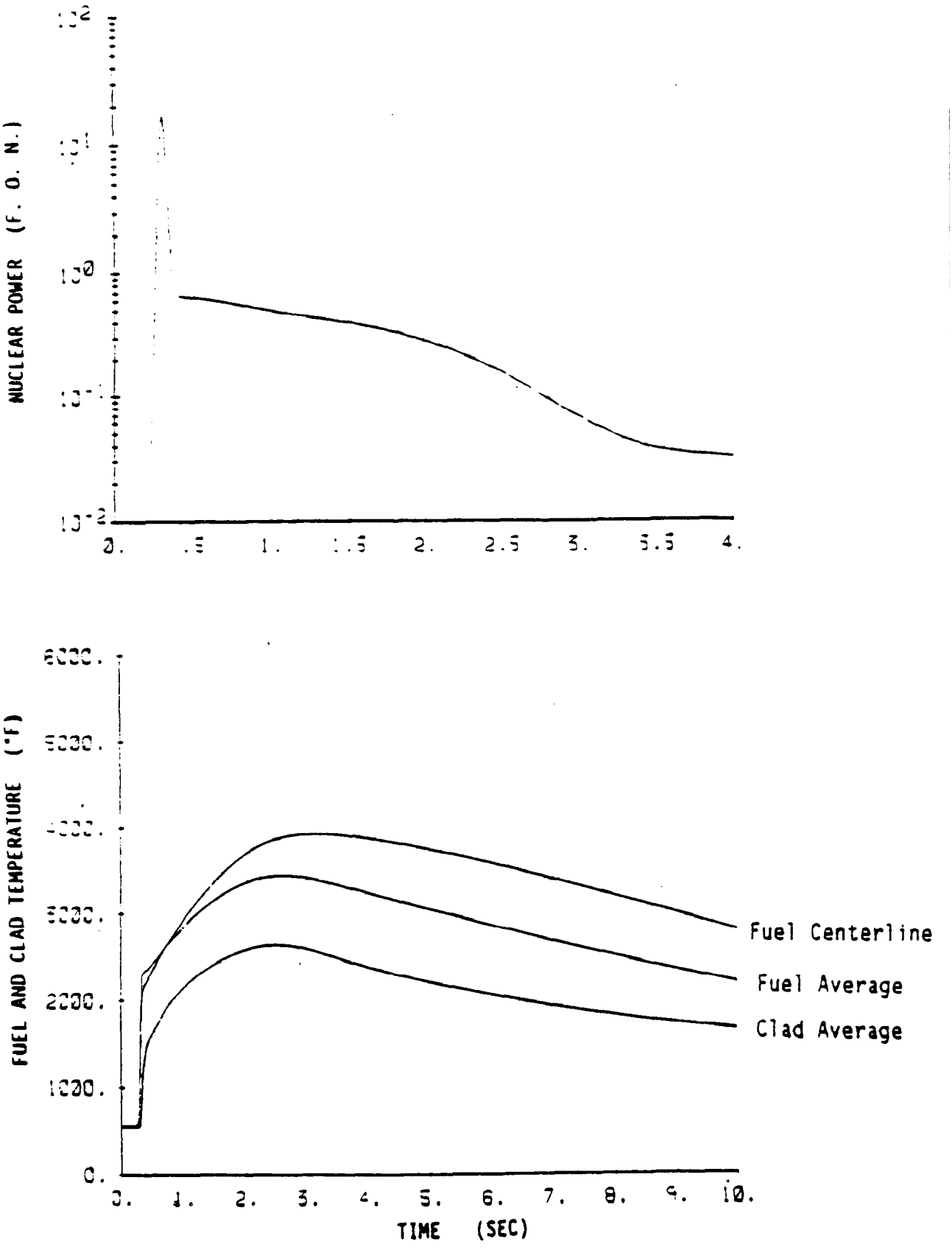


Figure 14.2.6-2 Rod Ejection
Nuclear Power and Fuel and Clad Temperatures Versus Time for Hot Zero Power at Beginning of Life

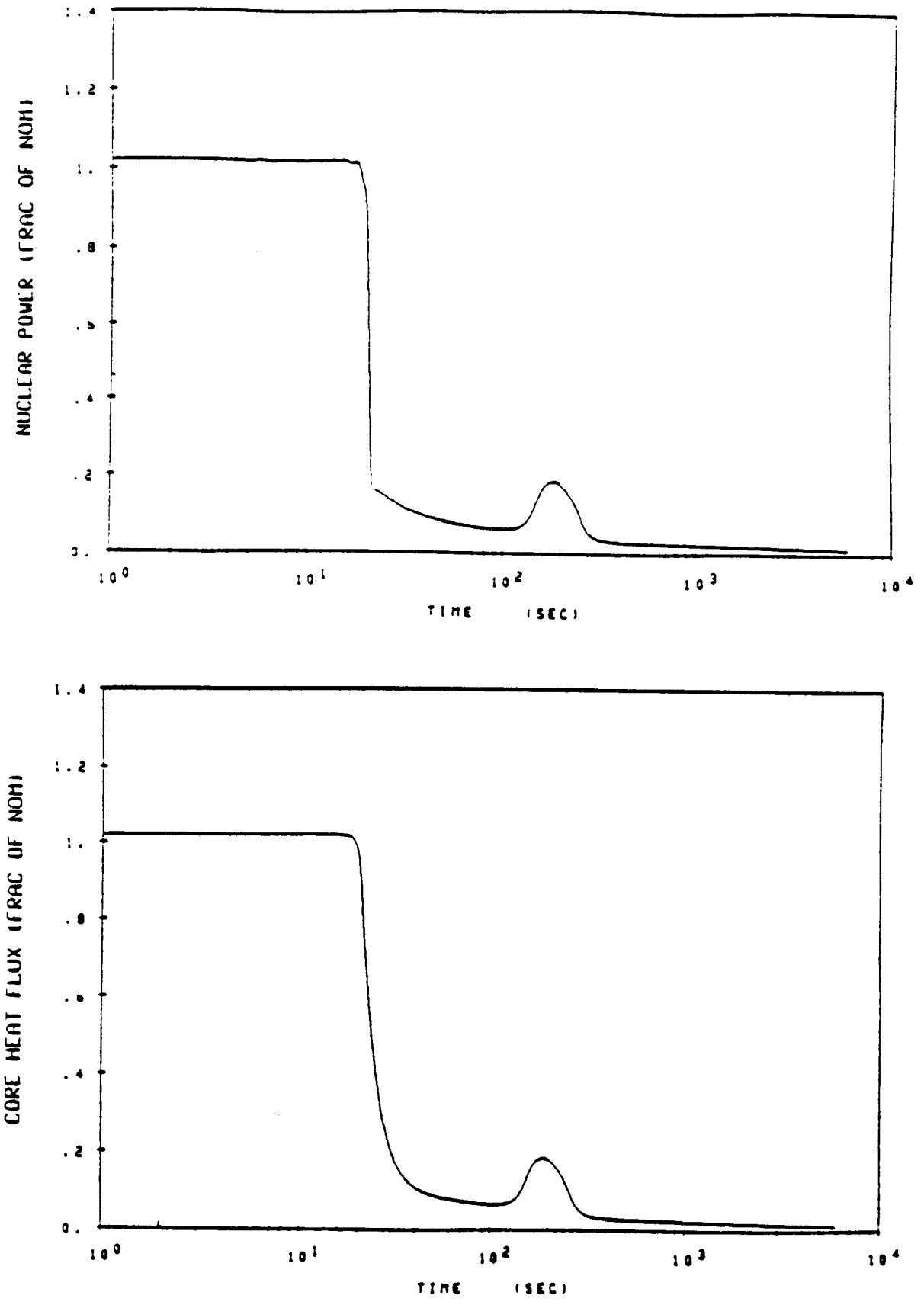


Figure 14.2.8-1 Feedline Break with Power
Nuclear Power and Core Heat Flux Versus Time

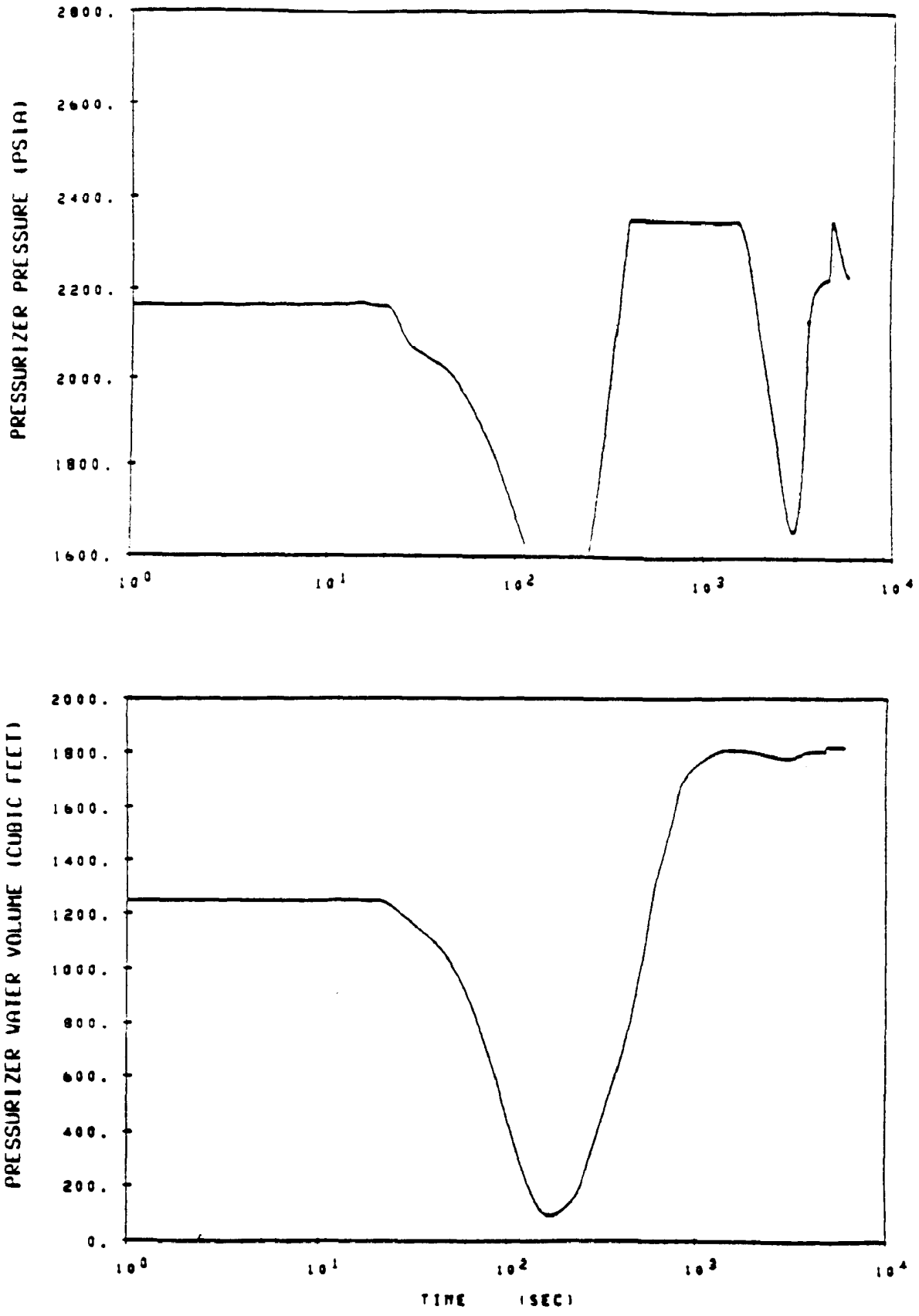


Figure 14.2.8-2 Feedline Break with Power
Pressurizer Pressure and Pressurizer Water Volume Versus
Time

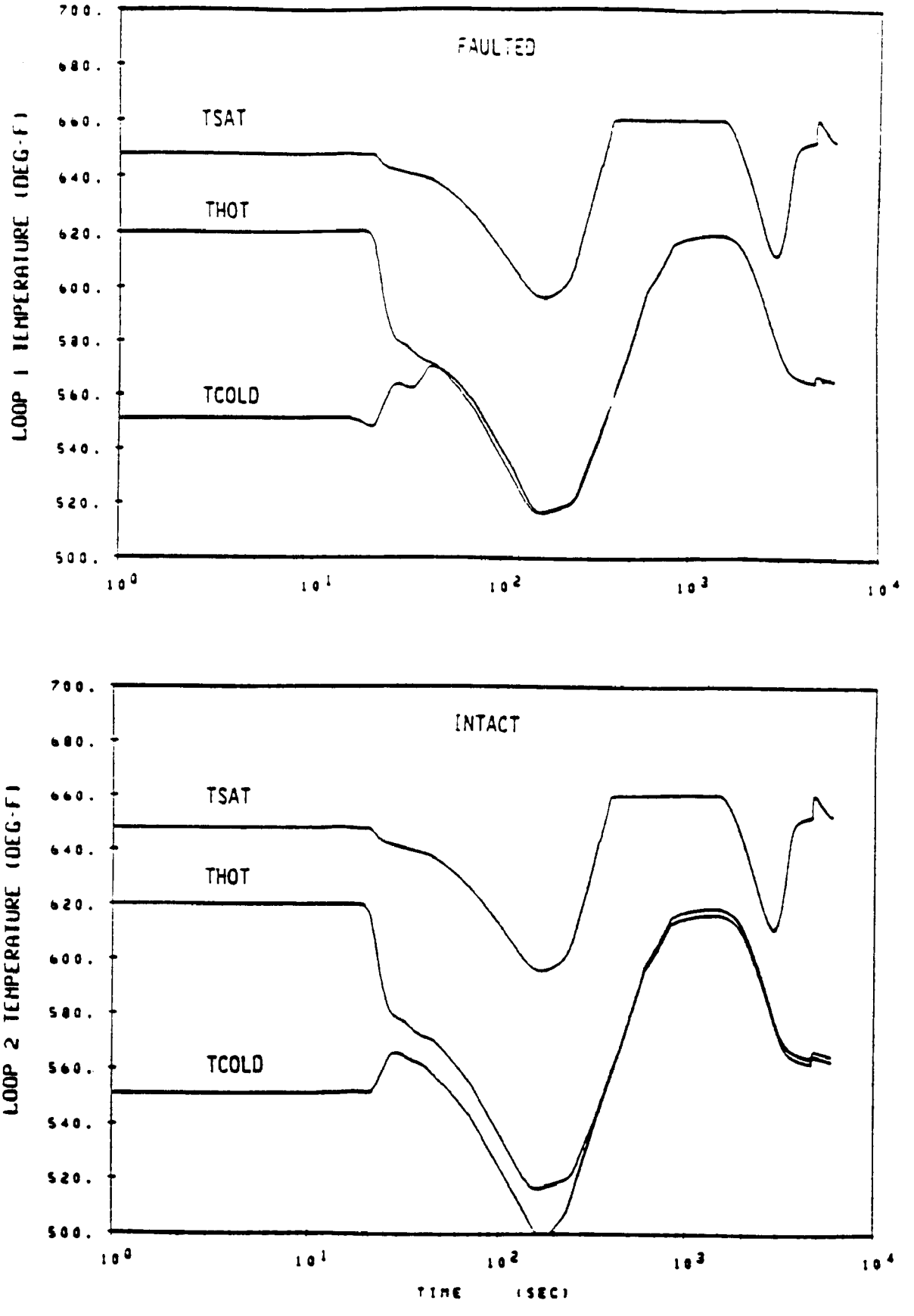


Figure 14.2.8-3 Feedline Break with Power
Faulted and Non-Faulted Loop Temperatures Versus Time

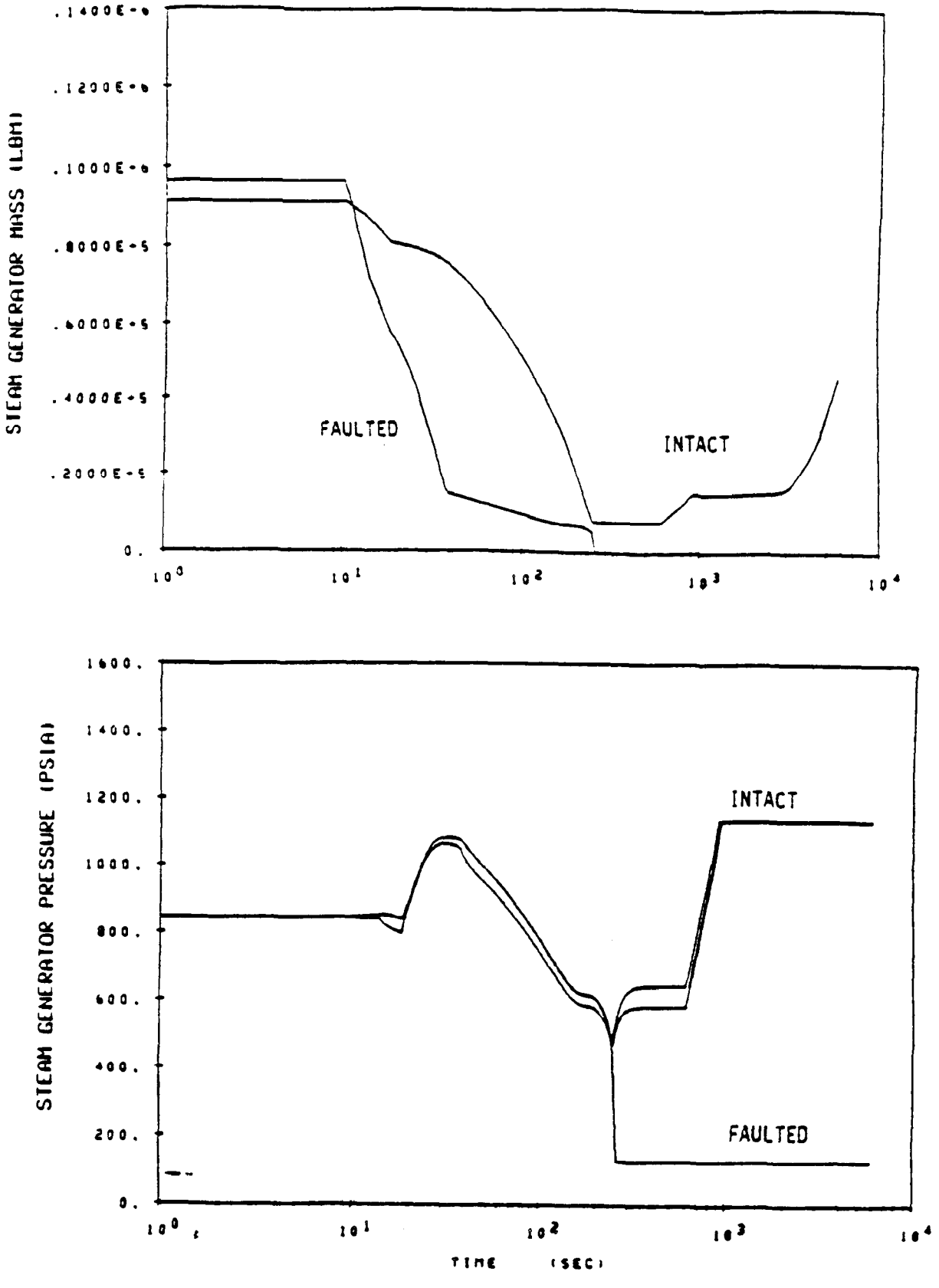


Figure 14.2.8-4 Feedline Break with Power
Steam Generator Mass and Steam Generator Pressure Versus Time

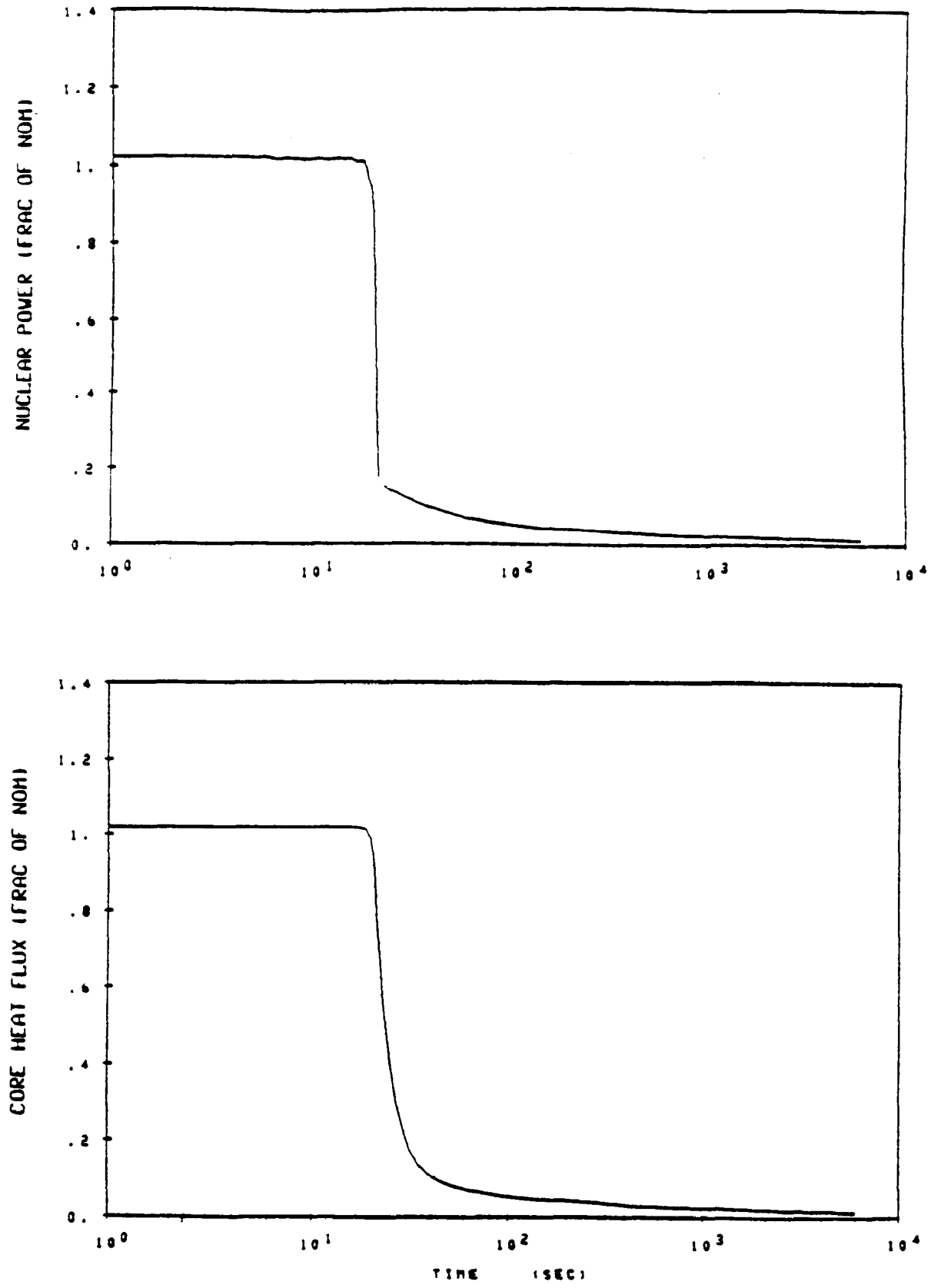


Figure 14.2.8-5 Feedline Break without Power
Nuclear Power and Core Heat Flux Versus Time

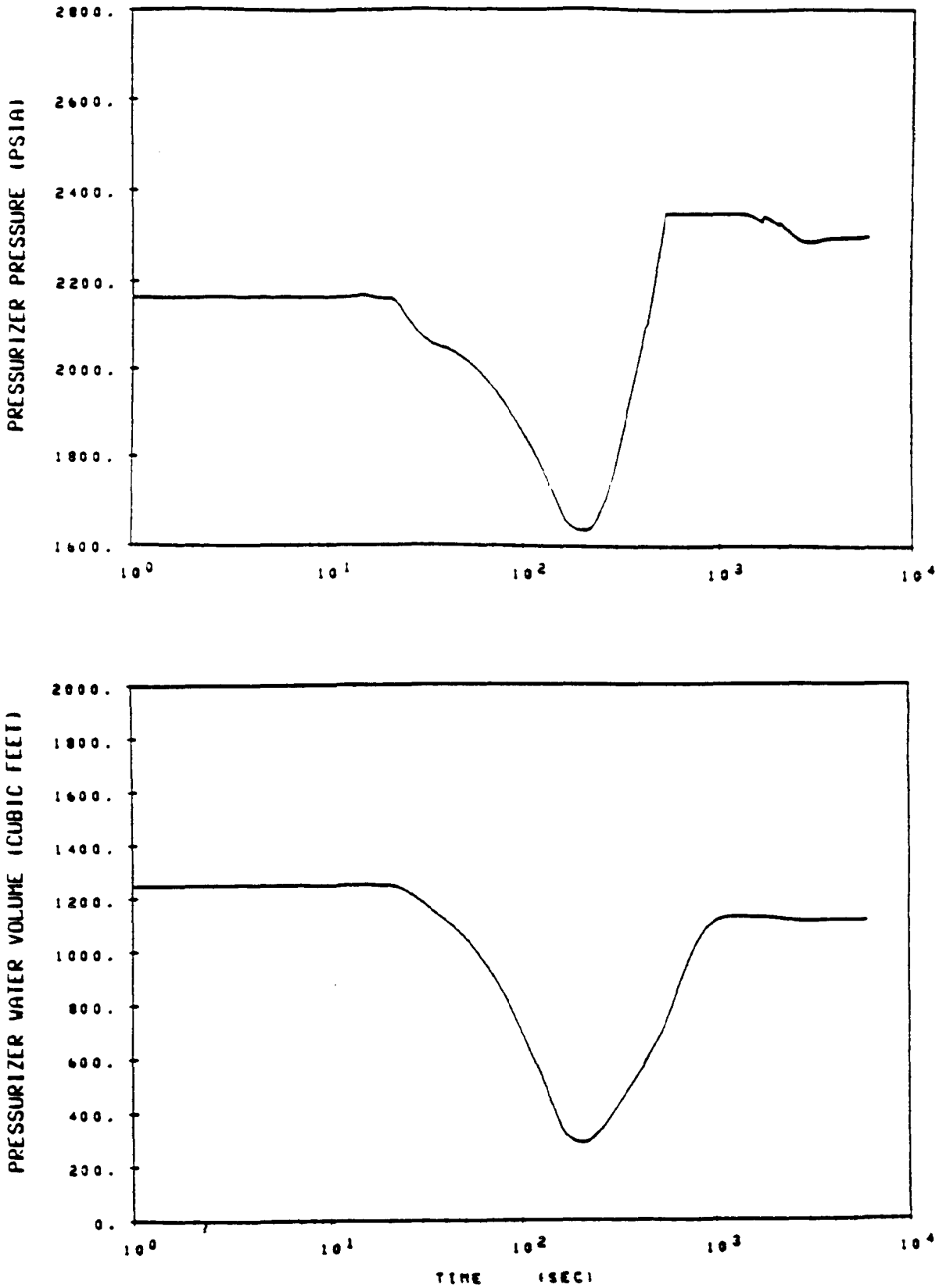


Figure 14.2.8-6 Feedline Break without Power
Pressurizer Pressure and Pressurizer Water Volume Versus Time

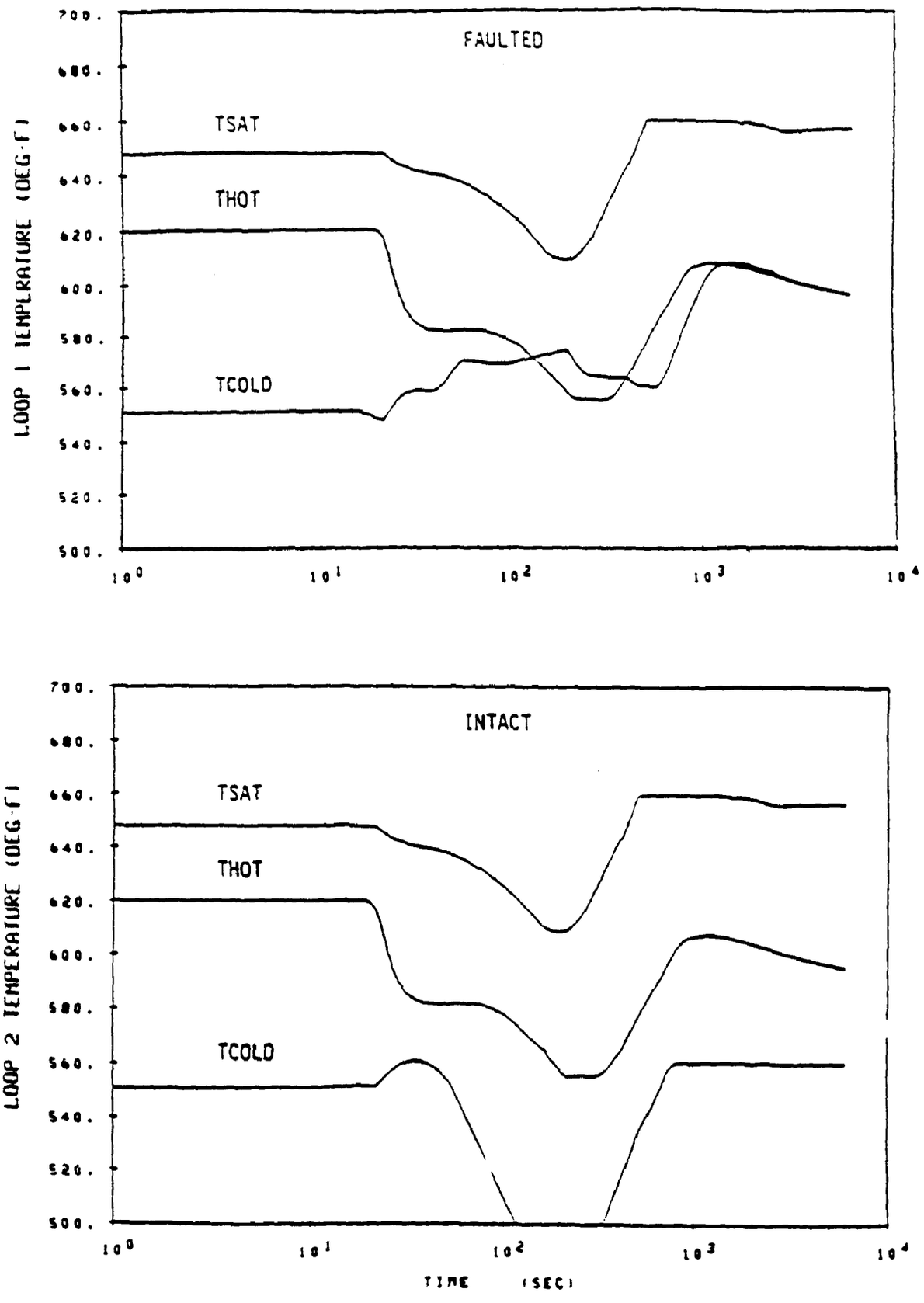
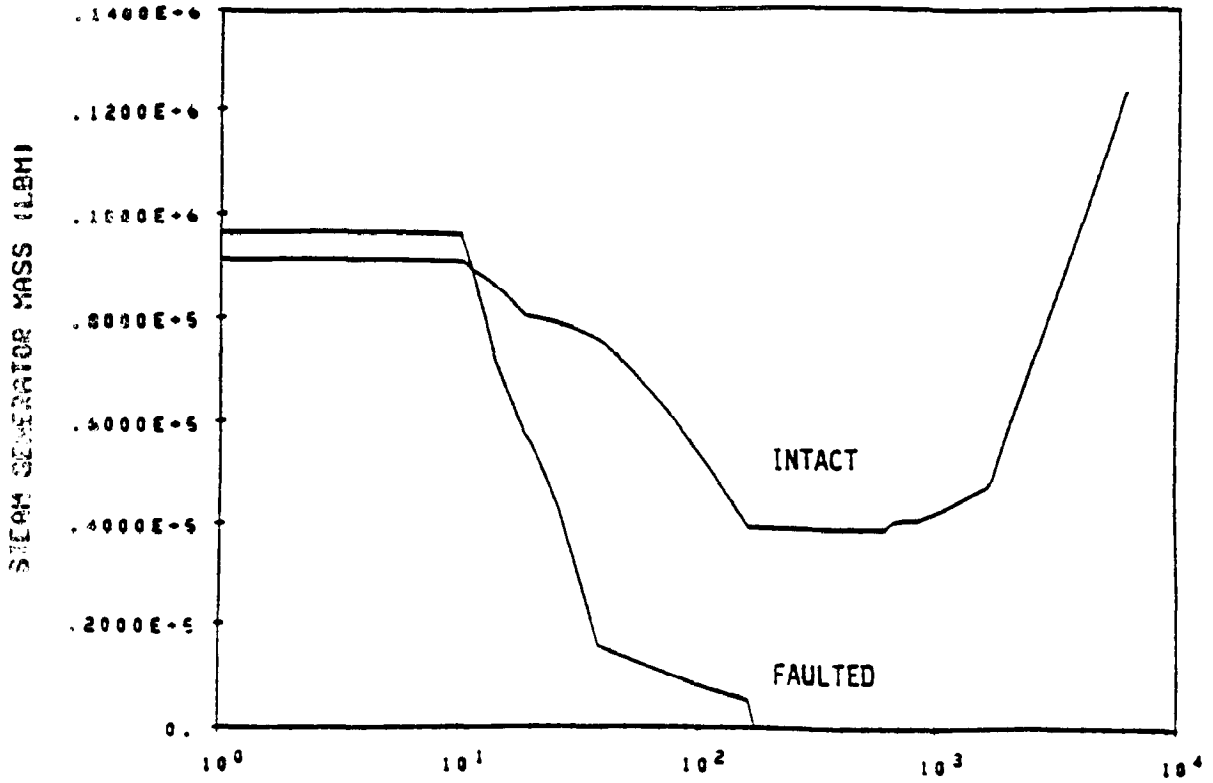


Figure 14.2.8-7 Feedline Break without Power
Faulted and Non-Faulted Loop Temperatures Versus Time



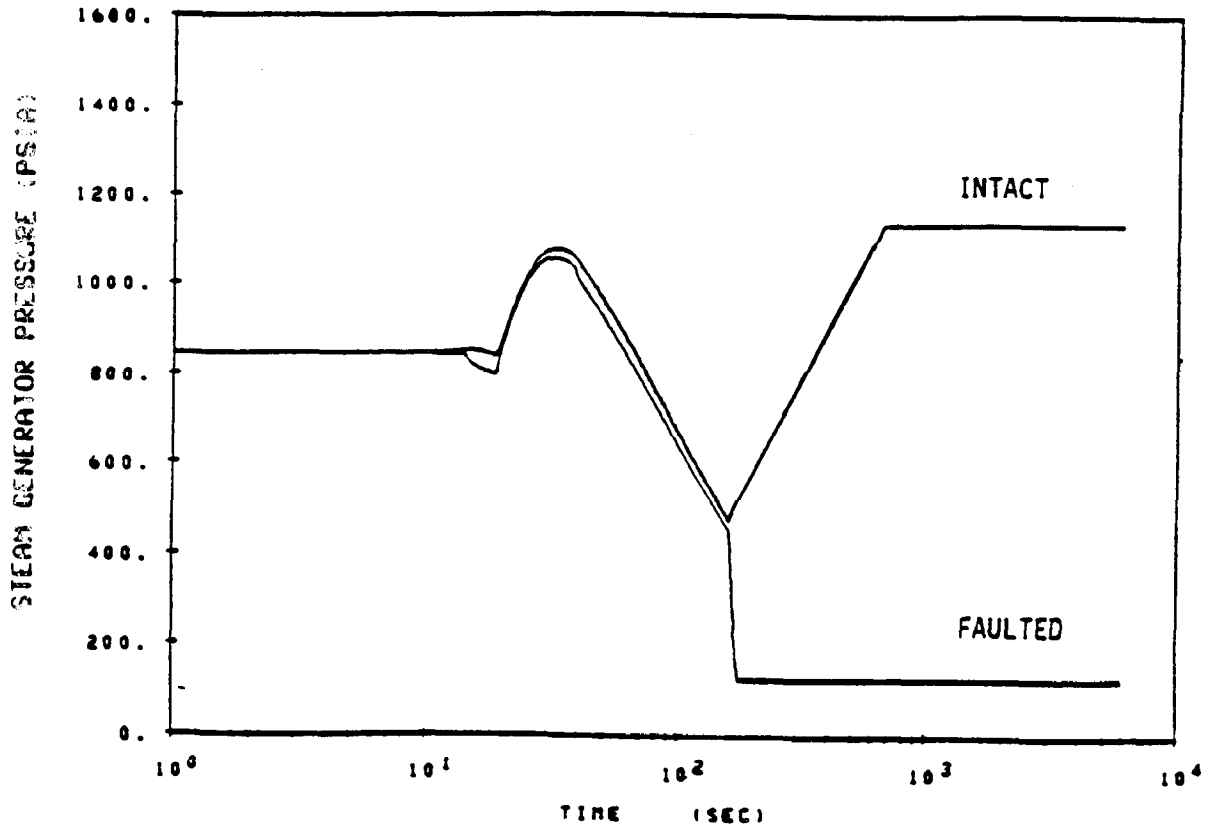


Figure 14.2.8-8 Feedline Break without Power
Steam Generator Mass and Steam Generator Pressure Versus Time