



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1200

December 2, 2022

Cheryl A. Gayheart
Regulatory Affairs Director
Southern Nuclear Company, Inc.
3535 Colonnade Parkway
Birmingham, AL 35243

SUBJECT: VOGTLE ELECTRIC GENERATING PLANT, UNITS 1&2 – DESIGN BASIS
ASSURANCE INSPECTION (TEAMS) INSPECTION REPORT 05000424/2022010
AND 05000425/2022010

Dear Cheryl A. Gayheart:

On October 21, 2022, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Vogtle Electric Generating Plant, Units 1&2 and discussed the results of this inspection with Mr. R. Norris and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Signed by Baptist, James
on 12/02/22

James B. Baptist, Chief
Engineering Br 1
Division of Reactor Safety

Docket Nos. 05000424 and 05000425
License Nos. NPF-68 and NPF-81

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV

SUBJECT: VOGTLE ELECTRIC GENERATING PLANT, UNITS 1&2 – DESIGN BASIS
 ASSURANCE INSPECTION (TEAMS) INSPECTION REPORT
 05000424/2022010 AND 05000425/2022010 DATED DECEMBER 2, 2022

DISTRIBUTION:

M. Kowal, RII
 S. Price, RII
 N. Doiley, RII
 M. Toth, RII
 RidsNrrPMVogtle Resource
 RidsNrrDro Resource
 PUBLIC

ADAMS ACCESSION NUMBER: **ML22334A231**

<input checked="" type="checkbox"/> SUNSI Review		<input checked="" type="checkbox"/> Non-Sensitive <input type="checkbox"/> Sensitive		<input checked="" type="checkbox"/> Publicly Available <input type="checkbox"/> Non-Publicly Available	
OFFICE	RII/DRS/EB1	RII/DRS/EB1	RII/DRS/EB1	RII/DRS/EB1	RII/DRS/EB1
NAME	C. Franklin	T. Fanelli	P. Braxton	J. Lizardi-Barreto	T. Chiun Su
DATE	11/15/2022	11/29/2022	11/14/2022	11/14/2022	11/15/2022
OFFICE	RII/DRS/EB1	RII/DRS/EB1			
NAME	A. Ruh	J. Baptist			
DATE	11/30/2022	11/2/2022			

OFFICIAL RECORD COPY

**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Numbers: 05000424 and 05000425

License Numbers: NPF-68 and NPF-81

Report Numbers: 05000424/2022010 and 05000425/2022010

Enterprise Identifier: I-2022-010-0047

Licensee: Southern Nuclear Company, Inc.

Facility: Vogtle Electric Generating Plant, Units 1&2

Location: Waynesboro, GA

Inspection Dates: October 03, 2022 to October 21, 2022

Inspectors: P. Braxton, Reactor Inspector
T. Fanelli, Senior Reactor Inspector
C. Franklin, Reactor Inspector
J. Lizardi-Barreto, Reactor Inspector
A. Ruh, Resident Inspector
T. Su, Reactor Inspector

Approved By: James B. Baptist, Chief
Engineering Br 1
Division of Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a design basis assurance inspection (teams) inspection at Vogtle Electric Generating Plant, Units 1&2, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.21M - Design Bases Assurance (DBA) Inspection

The inspectors evaluated the following components and listed applicable attributes, permanent modifications, and operating experience:

Design Review - Risk-Significant/Low Design Margin Components (IP Section 02.02) (7 Samples)

- (1) 4160 VAC bus 1AA02, transformer 1BB07X
 - Material condition and configuration during a walkdown
 - Operating environment
 - Corrective and preventive maintenance effectiveness
 - Protective relay setting and calibration
 - Overcurrent protection and coordination
 - Alignment of Plant Wilson after station blackout

- (2) Nuclear service cooling water spray header and cold weather bypass butterfly valves HV-1668A, HV-1669A, HV-1668B and HV-1669B
 - Thrust, set-point, weak-link, maximum differential pressure calculations
 - Required capability during normal, abnormal, and emergency operating procedures, including loss of nuclear service cooling water system
 - Results of recent surveillance tests and procedure adequacy
 - Material condition of valves and associated equipment during a walkdown
 - Corrective action effectiveness
 - Control circuit scheme and periodic verification of electrical interlocks

- (3) Main steam isolation valve 2HV3016B
 - Thrust, set-point, weak-link, maximum differential pressure calculations
 - Air supply sizing (time, and pressure)
 - Required capability during normal, abnormal, and emergency operating procedures, including faulted steam generator, steam generator tube rupture, and station blackout
 - Results of recent surveillance tests and procedure adequacy
 - Material condition of valves and associated equipment during a walkdown

- Corrective action effectiveness
- (4) Containment purge valves 1/2HV-2624A/B, 2626A/B, 2627A/B, 2628A/B, 2629A/B
- Material condition and configuration during a walkdown
 - Effectiveness of corrective and preventative maintenance to ensure containment isolation capability
 - Maintenance of seismic qualification
- (5) Auxiliary feedwater system pumps and valves 1/2-1302-P4-001, P4-002, P4-003, HV5132
- Material condition and configuration during a walkdown
 - Correlation of flowrate acceptance criteria to accident conditions considering instrument inaccuracies and diesel generator voltage and frequency uncertainties
 - Surveillance testing results and compliance with inservice testing code requirements (instrumentation selection, reference values, documentation)
 - System capability and protection during flooding from high energy line breaks
 - Modifications to safety analysis design basis cases and system assumptions
 - Control valve closing capability under reverse flow conditions described in failure modes and effects analysis
- (6) Residual heat removal system pumps 1/2-1205-P6-002
- Material condition and configuration during a walkdown
 - Design bases and design assumptions have been appropriately translated into design calculations, drawings, and procedures.
 - Corrective actions and preventative maintenance effectiveness
 - Maintenance of seismic qualification
 - Results of recent surveillance tests in accordance with applicable procedures
 - Termination of inadvertent safety injection
- (7) Residual heat removal to safety injection discharge motor operated gate valves 1/2-1205-HV-8804B
- Material condition and configuration during a walkdown
 - Design bases and design assumptions have been appropriately translated into design calculations, drawings, and procedures.
 - Results of recent surveillance tests in accordance with applicable procedures
 - Corrective actions and preventative maintenance effectiveness
 - Maintenance of seismic qualification

Design Review - Large Early Release Frequency (LERFs) (IP Section 02.02) (1 Sample)

- (1) 125 VDC panel 1BD11/1BD1 and circuit breaker 1BD105
- Corrective and preventive maintenance effectiveness

- Battery capacity calculation
- Effectiveness of corrective actions
- Periodic verification of electrical interlocks

Modification Review - Permanent Mods (IP Section 02.03) (4 Samples)

- (1) SNC959023, SNC959025 – Unit 1 and 2 Solid State Protection System Bridging Strategy: Low Margin, Increase Under-Voltage Trip Attachment Coil Circuit Voltage
 - Post-modification testing and operability
 - Coil voltage
- (2) SNC922254, SNC922258 – Unit 1 and 2 Auxiliary Feedwater Orifice Replacements
 - Orifice installation orientation
 - Post-modification flow tests and hydraulic model assumptions
 - Pumphouse internal flooding scenario impacts
- (3) SNC1046656 – Unit 2 A-train Main Steam Isolation Valve Actuator Replacements
 - Design drawings, installation adequacy and post-modification testing
 - Pipe stress analysis
 - Updated Technical Requirements Manual curves
 - Stroke times during high- and low-pressure operations
- (4) SNC1062134, SNC1062133 – Eliminate Single Point Vulnerabilities from Main Steam Isolation Valve Control Circuits for Normally Energized MDR Relays and Micro Switches
 - Design inputs and elementary drawing logic
 - Post-modification testing

Review of Operating Experience Issues (IP Section 02.06) (3 Samples)

- (1) Information Notice 2021-01, Lessons Learned from NRC Inspections of Design-Basis Capability of Power-Operated Valves at Nuclear Power Plants
- (2) Information Notice 2020-02, Flex Diesel Generator Operational Challenges
- (3) Information Notice 2019-10, Failures Reported in Eaton/Cutler Hammer A200 and Freedom Series Contactors

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On October 21, 2022, the inspectors presented the design basis assurance inspection (teams) inspection results to Mr. R. Norris and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.21M	Calculations	GP-17105	Weak Link Analysis for Valves 8804 A & B	05/11/2000
		1/2X6AA06-661-4	Valve 4G Seismic Upgrade Evaluation Report	11/09/1988
		AX5AC01-00365	HV5132 Weak Link AFW Discharge Valve	09/03/1987
		AX5C03-00722-1	Fisher Weak Link Analysis and Calculation Procedure for 9280 Series Butterfly Valves	3/17/2000
		DC-1000-M	General Design Criteria (Mechanical)	15
		DC-1005	Seismic – Interdiscipline	4
		DC-1007	Environment - Interdiscipline	37
		DC-1009	Plant Single-Failure Criteria	9
		DC-1018	Pipe Break Criteria – Interdiscipline	7
		DC-1204	Safety Injection System	6
		DC-1205	Residual Heat Removal System	6
		SNC143798	NMP-ES-050-F01, RER Response Form to Fleet NMP-MA-010 Meets the Site's Seismic Criteria	C110878101
		VNP-1HV8804B	Thrust and Torque Calculation 1HV8804B (VNPS-1) AC Motor Operated GL-96-05 Gate Valve	2
		VNP-2HV8804B	Thrust and Torque Calculation 2HV8804B (VNPS-2) AC Motor Operated GL-96-05 Gate Valve	2.3
		X2CQ10.04	Civil/Seismic Requirements of Scaffolding in Safety Related Area per Procedure NMP-MA-010	1
		X3CA18	Evaluate Station Auxiliary System Loads	17
		X3CF02	Determine Class 1E Battery Sizing per IEEE STD 485-1983	25.0
		X3CT08	Fire Event Safe Shutdown Circuit Analysis	29
		X4C1000U01	Differential Pressure Calculations for REA VG-9049 (Generic Letter 89-10)	19
		X4C1000U01	Differential Pressure Calculations for REA VG-9049 (Generic Letter 89-10)	19
X4C1202S26	Ultimate Heat Sink Analysis	5		
X4C1202V02	NSCW Cooling Tower Fan Performance - After SBO	4		
X4C1202V03	Verification of NSCW Constant Heat Loads and Flows and Cooldown Heat Loads	9		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		X4C1204S07	ECCS Reference Summary	4
		X4C1204V23	Generic Decay Heat Power Curve Calculation	1
		X4C1205V01	Residual Heat Removal System Design	11/08/1983
		X4C1205V02	Cooldown Transient with One RHR Train	12/1/1993
		X4C1205V03	RHR Pump Runout	11/10/1987
		X4C1205V04	One and Two Train Cooldown of RCS	07/11/2013
		X4C1205V04	One and Two Train Cooldown of RCS	3
		X4C1301S00F1	FMEA - Main Steam System	1
		X4C1301U01	Main Steam Line Pressure Safety Valve Forces	0
		X4C1302S00F1	FMEA - Auxiliary Feedwater and Condensate Storage System	7
		X4C1302V15	AFW Hydraulic Analysis	4
		X4CPS.0075.411	Qualification of Valves for Generic Letter 89.10 (FQP-11AB-9)	0
		X5CP1668	NSCW Cooling Tower Spray Header Shutoff and Bypass Valve Controls Train "A" & Train "B"	6
		X5CP5150	Aux Feedwater Supply to SG Flow Indication	6
		X6CAJ.08	Main Steam Line Break Offsite Doses	1.0
		X6CXC36	Flooding Analysis - VNP Auxiliary Feedwater Pumphouse - Unit 1 & 2	9
		Corrective Action Documents	10648087	Adverse trend previously identified in 2B RHR Pump dP
	10667477		SI "B" Boric Acid buildup	11/26/19
	10667479		SI Accumulator Local Sample valve Boric Acid leak	11/26/19
	10698551		U1 CNMT Penetration 84 CNMT PURGE EXH LLRT 1HV-2628A failed	03/25/20
	10730669		2R21 Outage Scheduling MOV Delay Starts Lesson Learned	08/16/20
	10740052		14" Flexible connection on the U2 CTB Normal Purge exhaust line degraded	09/19/20
	10748136		SI Accumulator Local Sample valve & Swagelock Fitting Boric Acid leak	10/22/20
10748140	SI Accumulator Local Sample valve & Swagelock Fitting Boric Acid leak	10/22/20		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		10828191	Unit 1 Containment Mini-Purge Exhaust Tripped	09/18/21
		10829955	Unit 1 Train A RHR pump issue observed during start following maintenance	09/25/21
		10861521	2HV0607 - Minor to Moderate Dry Boric Acid on Flange and bolting	02/23/22
		10865776	Valve Needed for 2R22 N2 SI	03/11/22
		10866977	2-HIC-607A has no light indication	03/16/22
		10881597	Mini Purge Supply Fan trips on low flow causing QHVC alarm ALB52-F06	05/17/22
		276875, 277066, 10698290, 10698575, 10735584, 10651765, 10677066, 10694969, 10695405, 10695498, 10706799, 10731564, 10742075, 1070318, 1074681, 10702730, 10770466, 10777711, 10781950, 10799120, 10787581, 10708502, 10656219, 10719559, 10804425, 10811954,		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		10816524, 10819167, 10847364, 10852544, 10736252, 10727859, 10868529, 10774529, 10840714, 10840885, 10899662, 10826914, 10802172, 10797827, 10740785, 10856612, 10789981, 10788415, 10777575, 10746820, 10730606, 10725202, 10723290, 10868818, 10866094, 10834601, 10740784, 10735561, 10878409, 10821009, 10799528, 10768308, 10701339, 10695396,		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		10669859, 10656214, 10878379, 10870460, 10859659, 10792727, 10741214, 10690643, 10906295, 10870727, 10833215, 10775867, 10901500, 10902970		
	Corrective Action Documents Resulting from Inspection	10912618	Incorrect dP recorded for TDAFW in surveillance 14748-103-SNC788814	
		10912619	Minor oil residue 1B MDAFW pump motor	
		10912896	Aux building C46 overhead lighting not operating	
		10912898	Auxiliary building C46 floor degradation	
		10913164	Inconsistent use of of elbows on solenoid exhaust ports	
		10913181	Sections of flexible conduit do not have grounding cable installed	
		10913183	HVAC damper limit switch missing FME cover	
		10913184	Scaffold with outdated tag	
		10913185	Housekeeping around U1 and U2 containment purge valve actuators	
		10913186	Minor oil leak on 1HV2629B valve actuator	
		10913187	Scaffold clamp to be removed	
		10913188	Review installation of permanent scaffold underneath 1HV2629A/B	
		10913189	Flex joint tie rod missing nut	
		10913190	Coating peeling on containment purge valve actuator	
	10916850	NMP-MA-010 provides insufficient guidance for particular cases		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		10916866	Capability of AFW discharge MOV function not demonstrated	
	Drawings	1X3D-AA-A01A	Main One Line Unit 1	29
		1X3D-AA-D02A	One Line Diagram, 4160 Switchgear 1AA02	14
		1X3D-AA-E07A	One Line Diagram- 480V Switchgear 1BB07	10
		1X3D-AA-M)8A-56	Unit 1 Relaying Data	3
		1X3D-AA-M08A-034	Unit 1 Relaying Data	4
		1X3D-AA-M08A-035	Unit 1 Relaying Data	2
		1X3D-AA-M08A-32	Unit 1 Relaying Data	1
		1X3D-BD-K05U	Elementary Diagram for 1HV-1668A	17
		1X3D-BD-K05V	Elementary Diagram for 1HV-1668B	12
		1X3D-BD-K05W	Elementary Diagram for 1HV-1669A	14
		1X3D-BD-K05X	Elementary Diagram for 1HV-1669B	12
		1X4AR17-00208-2	Air and Hydraulic System Schematic & Actuator Control Wiring Diagram for Gate Valve 1HV-3026B	1.0
		1X4DB121	P&I Diagram Safety Injection System No. 1204	42
		1X4DB122	P&I Diagram Safety Injection System No. 1205	52
		1X4DB133-1	P&ID Nuclear Service Cooling Water System No. 1202	52
		1X4DB133-2	P&ID Nuclear Service Cooling Water System No. 1202	62
		1X4DB161-1	P & I Diagram Auxiliary Feedwater System , Condensate Storage & Degasifier System, System No. 1302	46
		1X4DB161-2	P & I Diagram Auxiliary Feedwater System , System No. 1302	29
		1X4DB161-3	P & I Diagram Auxiliary Feedwater System , (Aux Feedwater Pump Turbine Driver), System No. 1302	42
		1X4DB168-3	P & I Diagram Condensate & Feedwater System, System No. 1305	36
	1X6AA06-00411	Motor Op. Gate Valve Mod 08002GM84FEB010	6	
	2X4DB159-1	PI&D Main Steam System No. 1301	38	
	39A2461	14" Body NT316-SR2-M3-9280 Bettis Actuated Control	C	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Valve CW to Close Valve Shaft Rotation Horiz Pipe, Vert Shaft	
		39A2462	14" Body NT316-SR2-M3-9280 Bettis Actuated Control Valve CW to Close Valve Shaft Rotation Horiz Pipe, Vert Shaft	C
		AX6AF02-20007-4	Pump Size 8X20WDF Coupled Design	D
		AX6AU01-00526	Westinghouse Process I&C Volume 1	14
		AX6D025	Control Building Floor Plan Elevation 220'0" Level 1 Seismic Category Areas	7
		AX6D030	Control Building and Equipment Building Floor Plan EL. 220 FT. -0 IN. Level 1 Seismic Category Areas	3.0
		CX3DE001	Grounding – General Notes, Symbols & Details	29
		DC-1018	Design Criteria - Pipe Break Criteria - Interdiscipline	7
		DC-1302	Design Criteria - Auxiliary Feedwater System	
		SNC1062133E001	Elementary Diagram Main Steam System 1HV-3006A	1.0
		SNC1062133E002	Elementary Diagram Main Steam System 1HV-3016A	1.0
	Engineering Changes	SNC1046656	U2 A-train MSIV Actuator Replacements	A
	Engineering Changes	SNC1046656	U2 A-Train MSIV Actuator Replacement	4
	Engineering Changes	SNC1062133	Eliminate SPVs from MSIV control circuits for Normally Energized MDR relays and micro switches	0
	Engineering Changes	SNC1062134	Eliminate SPVs from MSIV control circuits for Normally Energized MDR relays and micro switches	0
	Engineering Changes	SNC922254	U1 AFW Orifice Replacement	3
	Engineering Changes	SNC922258	U2 AFW Orifice Replacement	3
	Miscellaneous	AX5AG02-00103	Test Report of Flow Determination Testing on Orifices Performed for Plant Vogtle	2
	Miscellaneous	DC-1000-E	General Design Criteria (Electrical)	23
	Miscellaneous	DC-1804	4160-V AC System	10
	Miscellaneous	DC-1805	480V AC System	13
	Miscellaneous	FCP-7	Butterfly Valve Sizing and Internal Component Calculation Procedures	B
	Miscellaneous	GP-19473	Transmittal of LTR-TA-16-148, "Vogtle Units 1 and 2: Auxiliary Feedwater Flow Assumptions Modeled in the Feedwater System Pipe Break	11/07/2016

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			(Feedline Break) Analysis"	
		NSAL-19-2	NSAL-19-2 Revision 0, "Contactors Failing to Release/Open When De-energized"	09/24/2019
		RAL 5168	Main Steam Isolation Valve Maintenance Manual	0
		RAL-7094	Fatigue Analysis for MSIV Actuator Assembly	0
		TE1051341	Westinghouse NSAL-19-2 "Contactors Failing to Release/Open When Deenergized"	9/6/2019
		Unit 1 SHR CTMT	CTMT System Health Report # 2101 Vogtle Unit 1	09/30/2022
		Unit 1 SHR RHR	RHR System Health Report # 1205 Vogtle Unit 1	09/30/2022
		Unit 1 SHR SI	SI System Health Report # 1204 Vogtle Unit 1	09/30/2022
		Unit 2 SHR CTMT	CTMT System Health Report # 2101 Vogtle Unit 2	09/30/2022
		Unit 2 SHR RHR	RHR System Health Report # 1205 Vogtle Unit 2	09/30/2022
		Unit 2 SHR SI	SI System Health Report # 1205 Vogtle Unit 2	09/30/2022
		WCAP-17308-NP-A	Treatment of Diesel Generator (DG) Technical Specification Frequency and Voltage Tolerances	0
	Operability Evaluations	AX6AA15-00002	Vogtle Electric Generating Plant – Unit ½, Pump and Valve Operability Report	
	Procedures	11882-1	Unit 1 Outside Area Rounds	106
		14150A-1	Train A NSCW Fan/Spray Valve Surveillance	6
		14150B-1	Train B NSCW Fan/Spray Valve Surveillance	6
		14721C-1	ECCS Subsystem Flow Balance and Checkvalve Refueling Inservice Test	13
		14721D-1	ECCS Subsystem Flow Balance and Checkvalve Refueling Inservice Test	2
		14748-1	AFW Pump and Check Valve Cold S/D IST and TDAFWP Auto Start Test	35.1
		14805B-1	Train B Residual Heat Removal Pump IST and Response Time Test	5.3
		14807A-1	Train A Motor Driven Auxiliary Feedwater Pump / Check Valve Inservice and Response Time Test	10.1
		14807B-1	Train B Motor Driven Auxiliary Feedwater Pump / Check Valve Inservice and Response Time Test	5.2
		14825-1	Quarterly Inservice Valve Test	106.5

Inspection Procedure	Type	Designation	Description or Title	Revision or Date	
		14850-1	Cold Shutdown Valve Inservice Test	60.0	
		14850-2	Cold Shutdown Valve Inservice Test	54	
		17061-1	Annunciator Response Procedures for ALB 61 on Process Control Panel	19.4	
		18021-C	Loss of Nuclear Service Cooling Water System	20	
		19020-1	E-2 Faulted Steam Generator Isolation	1.1	
		19030-1	E-3 Steam Generator Tube Rupture	5	
		26007-C	Bettis Actuator Maintenance (Track Series)	13.3	
		AX5AC03-10001	Vogtle Electric Generating Plant – Unit No. 1/2 Operating and Maintenance Instructions Disassemble and Assembly for Nuclear Series Actuators	2.0	
		AX6AF02-20030	Vogtle Electric Generating Plant – Unit No. 1/2 Installation Operation & Maintenance Manual & Supplement RHR Pumps	18.0	
		MIS-17-009	Vogtle Electric Generating Plant Fourth 10-Year Interval Inservice Testing Program	13.0	
		NMP-AD-028	10 CFR 21 Evaluations and Reporting Requirements	7.0	
		NMP-AD-028	10 CFR 21 Evaluations and Reporting Requirements	7.0	
		NMP-ES-006	Preventive Maintenance Implementation and Continuing Equipment Reliability Improvement	13.0	
		NMP-ES-006-001	PM Template Management and PM Optimization Guidance	4.1	
		NMP-ES-006-002	Preventive Maintenance Change Requests	11.0	
		NMP-ES-006-003	Preventive Maintenance Feedback	6.1	
		NMP-ES-006-004	Preventive Maintenance Oversight Group	6.0	
		NMP-GM-002	Corrective Action Program	16.0	
		NMP-GM-002-001	Corrective Action Program Instructions	43.0	
		NMP-MA-010-001	Permanent Scaffolding Installation, Modification, and Removal Process	1.3	
			Work Orders	SNC1158288	B-RHR IST (Q) – I&C Support Req.
			SNC1171119, SNC1142967, SNC622778, SNC970653,		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		SNC832642, SNC795416, SNC979326, SNC832139, SNC832138, SNC1030647, SNC1030645, 14748-101T- 38119, 14748- 101-1101737801, SNC351861, SNC788814, SNC905686, SNC1159355, SNC1174182, SNC1189035, SNC1163251, SNC1178133, SNC1193831, SNC1105655, SNC1233676, SNC 396689, SNC494595, SNC731534, SNC904311, SNC909433, SNC913849, SNC981075, SNC377678, SNC774779, SNC832143, SNC856543, SNC972592, SNC1089908		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		SNC1172960	B-RHR IST (Q) – I&C Support Req.	0
		SNC1187666	B-RHR IST (Q) – I&C Support Req.	0
		SNC1258537	1205, 2HIC607A, Investigate and Correct Cause of no light Indication	0
		SNC809832	ECCS Flow/Bal-14721-C-1	0
		SNC809837	ECCS Flow/Bal-14721-C-1	0