



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 26, 2022

Mr. Daniel G. Stoddard
Senior Vice President
and Chief Nuclear Officer
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT 1 – CORRECTION TO
ISSUANCE OF AMENDMENT RELATED TO SURVEILLANCE
REQUIREMENT 4.0.5, ASSOCIATED BASIS, AND BASES 3/4.4.2,
3/4.4.6, AND 3/4.4.10 (TAC NO. MC0838)

Dear Mr. Stoddard:

On February 18, 2004 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML040620681), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 166 to Facility Operating License (FOL) No. NPF-12¹ for the Virgil C. Summer Nuclear Station, Unit 1. The amendment revised language and references in the Surveillance Requirement (SR) 4.0.5, associated Basis, and Bases 3/4.4.2, 3/4.4.6, and 3/4.4.10.

Subsequently, during the review of a recent amendment request, it was identified that the retyped page submitted with the amendment request and issued for amendment No. 166 omitted a word that had been requested by the proposed marked-up pages in the amendment request. Specifically, the word "with" was omitted from SR 4.0.5.a as follows:

- a. Inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance **with** the applicable ASME Code and Addenda as required by 10 CFR 50, Section 50.55a.

Accordingly, the corrected page 3/4 0-2 is enclosed with this letter. The correction does not change any of the conclusions associated with the issuance of Amendment No. 166 to Facility Operating License No. NPF-12, and does not affect the no significant hazards consideration published in the *Federal Register* on October 14, 2003 (68 FR 59219). No other amendments have affected Technical Specification Page 3/4 0-2 since issuance of amendment No. 166.

¹ Facility Operating License No. NPF-12 was renewed on April 23, 2004, and became Renewed Facility Operating License No. NPF-12.

D. Stoddard

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If you have any questions, please contact me at 301-415-2481, or via email at Ed.Miller@nrc.gov.

Sincerely,

/RA/

G. Edward Miller, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-395

Enclosure:
Revised Technical Specification Page 3/4 0-2

cc: Listserv

APPLICABILITY

SURVEILLANCE REQUIREMENTS

4.0.1 Surveillance Requirements shall be applicable during the OPERATIONAL MODES or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement.

4.0.2 Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval.

4.0.3 If it is discovered that a Surveillance was not performed within its specified frequency, as defined by Specification 4.0.2, then compliance with the requirement to declare the LCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified frequency, whichever is greater. This delay period is permitted to allow performance of the surveillance. A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed.

If the Surveillance is not performed within the delay period, the LCO must immediately be declared not met, and the applicable Action(s) must be entered.

When the Surveillance is performed within the delay period and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Action(s) must be entered.

4.0.4 Entry into an OPERATIONAL MODE or other specified condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the stated surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL MODES as required to comply with ACTION requirements.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, and 3 components shall be applicable as follows:

- a. Inservice inspection of ASME Code Class 1, 2 and 3 components and inservice testing of ASME Code Class 1, 2 and 3 pumps and valves shall be performed in accordance with the applicable ASME Code and Addenda as required by 10 CFR 50, Section 50.55a.
- b. Surveillance intervals specified for the inservice inspection and testing activities required by the applicable ASME Code and Addenda shall be applicable as follows in these Technical Specifications:

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3/4.4.6, AND 3/4.4.10 (TAC NO. MC0838) DATED SEPTEMBER 26, 2022

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