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L-PI-22-037

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant Units 1 and 2
Docket Nos. 50-282 and 50-306
Renewed Facility Operating License DPR-42 and DPR-60

Updated Approach for Prairie Island Unit 1 and Unit 2 Baffle Former Bolts

- References:
- 1) NSPM Letter L-PI-19-001, "Voluntary Submittal of Plant-Specific Evaluation to Extend the Re-Inspection Interval for Baffle Former Bolts," dated May 24, 2019. (NRC ADAMS Accession Number ML19144A244)
 - 2) NRC Letter re: Prairie Island Nuclear Generating Plant, Units 1 and 2 – Acknowledgement of Receipt of Analysis Related to Baffle Former Bolt Inspection Interval (EPID L-2019-LNS-008), dated November 19, 2019. (NRC ADAMS Accession Number ML19295C300)

In Reference 1, Northern States Power Company, a Minnesota corporation, doing business as Xcel Energy (hereafter "NSPM"), voluntarily submitted a plant-specific evaluation to extend the re-inspection interval for Prairie Island Nuclear Generating Plant (PINGP) Units 1 and 2 baffle former bolts (BFBs) from six years to ten years. The purpose of this letter is to inform the NRC that NSPM will be conducting BFB replacement campaigns on PINGP Units 1 and 2 in lieu of inspection. Specifically, a selection of PINGP Unit 1 and 2 BFBs will be replaced such that the resulting bolt pattern is in an analyzed configuration based on WCAP-15029-P-A, "Westinghouse Methodology for Evaluating the Acceptability of Baffle-Former-Barrel Bolting Distributions Under Faulted Load Conditions," and PWROG-18034-P-A, "Updates to the Methodology in WCAP-15029-P-A, Rev. 1, 'Westinghouse Methodology for Evaluating the Acceptability of Baffle-Former-Barrel Bolting Distributions Under Faulted Load Conditions,'" that ensures the baffle-former assembly is acceptable for continued operation. The replacement campaigns are planned for the Unit 1 and Unit 2 refueling outages in 2024 and 2023, respectively.

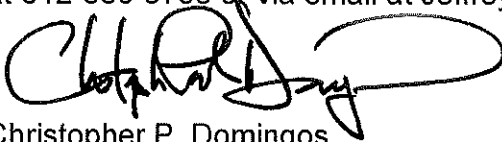
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Since the BFBs are a primary component for management of irradiation assisted stress corrosion cracking under the EPRI MRP-227, "Materials Reliability Program: Pressurized Water Reactor Internals Inspection and Evaluation Guidelines," Revision 1-A, hierarchy, NSPM has contracted with Westinghouse to develop an alternate aging management strategy and to document this strategy in a site-specific report. NSPM plans to submit this site-specific report to the NRC within approximately one year of the Unit 2 BFB replacement campaign.

Summary of Commitments

This letter makes no new commitments and no revisions to existing commitments.

If there any questions or if additional information is needed, please contact Mr. Jeff Kivi at 612-330-5788 or via email at Jeffrey.L.Kivi@xcelenergy.com.



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cc: Administrator, Region III, USNRC
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