UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE RD, STE 102
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

September 8, 2022

Eric Carr
President and Chief Nuclear Officer
PSEG Nuclear, LLC-N09
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION – DESIGN BASIS ASSURANCE INSPECTION (PROGRAMS) INSPECTION REPORT 05000354/2022012

Dear Eric Carr:

On August 3, 2022, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Hope Creek Generating Station and discussed the results with Edward Casulli and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at http://www.nrc.gov/reading-rm/adams.html and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Melvin K. Gray

Digitally signed by Melvin K. Gray
Date: 2022.09.08 10:12:58 -04'00'

Mel Gray, Chief
Engineering Branch 1
Division of Operating Reactor Safety

Docket No. 05000354
License No. NPF-57

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV

SUBJECT: HOPE CREEK GENERATING STATION – DESIGN BASIS ASSURANCE INSPECTION (PROGRAMS) INSPECTION REPORT 05000354/2022012 DATED SEPTEMBER 8, 2022

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DOCUMENT NAME: https://usnrc.sharepoint.com/teams/RIDRSPSBSEC/Shared Documents/Inspection Reports/2022 IR Draft/SHC/HC POV Inspection Report 2022-012.docx
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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Number:	05000354
License Number:	NPF-57
Report Number:	05000354/2022012
Enterprise Identifier:	I-2022-012-0008
Licensee:	PSEG Nuclear, LLC
Facility:	Hope Creek Generating Station
Location:	Hancocks Bridge, NJ
Inspection Dates:	July 11, 2022 to August 3, 2022
Inspectors:	P. Cataldo, Senior Reactor Inspector J. Kulp, Senior Reactor Inspector A. Patel, Senior Reactor Inspector
Approved By:	Mel Gray, Chief Engineering Branch 1 Division of Operating Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a design basis assurance inspection (programs) inspection at Hope Creek Generating Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to https://www.nrc.gov/reactors/operating/oversight.html for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.21N.02 - Design-Basis Capability of Power-Operated Valves Under 10 CFR 50.55a Requirements

POV Review (IP Section 03) (10 Samples)

The inspectors:

a. Determined whether the sampled POVs are being tested and maintained in accordance with NRC regulations along with the licensee's commitments and/or licensing bases.

b. Determined whether the sampled POVs are capable of performing their design-basis functions.

c. Determined whether testing of the sampled POVs is adequate to demonstrate the capability of the POVs to perform their safety functions under design-basis conditions.

d. Evaluate maintenance activities including a walkdown of the sampled POVs (if accessible).

(1) High Pressure Coolant Injection (HPCI) Pump Outboard Discharge Isolation Valve (H1BJ-BJ-JV-F006)
(2) Turbine Auxiliary Cooling B Inboard Supply Isolation Valve (H1EG-EG-HV- 2522B)
(3) Reactor Core Isolation Cooling Turbine Steam Line Drain Valve (H1FC-FC-HV- F025)
(4) Containment Instrument Gas Supply Header Cross Connect Isolation Valve (H1KL-KL-5156A)
(5) Scram Discharge Volume Inboard Vent Valve (H1BF-BF-HV- F010)
(6) Containment Hardened Vent Valve (GSHV-11541)
(7) Main Steam Drain Line Isolation Valve (H1AB-AB-HV- F019)
(8) HPCI Lube Oil Cooling Water Motor Operated Valve (H1BJ-BJ-JV-F059)
(9) D Service Water Pump Discharge Valve (H1EA -EA-HV-2198D)
(10) HPCI Turbine Steam Supply Valve (H1FD -FD-HV-F001)

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On August 3, 2022, the inspectors presented the design basis assurance inspection (programs) results to Edward Casulli and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.21N.02	Calculations	1BJ-HV-F006	Midas Calc Results 1BJ-HV-F006	Revision 5a
		EG-0046	STACS Operation	Revision 10
		H1FC -FC-HV-F025	ACE Calculation for H1FC -FC-HV-F025	Revision 0
		H1KL -KL-HV-5156A	ACE Calculation for H1KL -KL-HV-5156A	Revision 0
	Corrective Action Documents	20875479		
	Corrective Action Documents Resulting from Inspection	20908912		
		20910472		
		20910478		
		20910479		
		20910499		
		20910500		
		20910508		
		20910610		
		20910783		
		20910945		
		20911103		
		20911109		
		20911189		
		20911206		
		20911657		
	Miscellaneous	22A6237AB	High Pressure Coolant Injection System Design Spec Data Sheet	Revision 1
	Procedures	ER-AA-301-1004	Motor Operated Valve Performance Trending	Revision 6
		ER-AA-410-1002	Air Operated Valve Testing Requirements	Revision 8
		HC.OP-IS.BJ-0101(Q)	High Pressure Coolant Injection System Valves – Inservice Test	Revision 73
		HC.OP-LR.AB-0005(Q)	Containment Isolation Valve Type C Leak Rate Test CIVs 1ABHV-F016 and 1ABHV-F019 Penetration P12: Main	Revision 3

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Steam Line Drain	
	Work Orders	30179718		
		50210838		
		50216105		
		50216349		