Form 2.3-3 Operating Test Review Worksheet (JPMs)

Facility: D.C. Cook												Exam Date: 7/18-7/29/2022
1 JPM # or Title	2 Type (S/P/A)	3 ALT (Y/N)	4 LOD (1–5)	5 JPM Errors							6 U/E/S	7 Explanation
				LOD	REF	IC	TSK	CUE	CS	TL		
(RO/CO1) Boron Volume Determination for RCS Temperature Change	A	N/A	3						X		~~E~~ S	NRC: Performance Standard for step 4.1.2h is incomplete. The answer should be -18.3768 pcm with a calculation shown as (-15.314pcm/F x 1.2F). The first part of the calculation is missing so the 18.3768 units are incorrectly listed as pcm/F and there is no -15.314 mentioned. Response: Corrected performance standard as noted above.
(RO/CO2) 1/M Plot of Data from Reactor S/U	A	N/A	3								S	NRC: None.
(RO/EC) Req'ts for Isolating a PAC Cooler NESW Leak **(FREE SAMPLE)**	A	N/A	2								~~E~~ S	NRC: Task Title, Task No., K/A Reference & Rating do not match (likely a copy/paste error). Typo Task standards must be very clear and precise … e.g. "The operator has identified all valves that must be repositioned (closed) to isolate the leak at 1-WRV-960 on drawing OP-2-5114." Adjusted as requested. The initial conditions should be more precise … "has broken" should be descriptive/credible … e.g. "has a crack in the valve body". Adjusted as requested. The initiating cue should be more precise about reporting when the task is complete "… and inform the US" should be more declarative … e.g. "and inform the US when the task is complete." Adjusted as requested. Ensure any changes made to the Initial Conditions and Initiating Cues are duplicated in the Task Briefing. Corrected Response: All comments noted above were incorporated into the JPM.
(RO/RC) Locked High Rad Area (LHRA) Entry Requirements	A	N/A	2								S	NRC: None.

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				LOD	REF	IC	TSK	CUE	CS	TL		
(SRO/CO1) Review Reactor Vessel Void Time Calculation	A	N/A	2				X				~~E~~ S	NRC: The Task Standard should indicate that the applicant will need to correct the Reactor Vessel Void Vent time. The word vent is missing, and the given range seems to be missing a decimal point as the calculated value is 9.7 minutes but the acceptance range is listed as 931-1012 minutes. Response: Added "Vent" to Task Standard. Modified task standard to include correct vent time range of 9.3-10.12 minutes.
(SRO/CO2) Determine if Core Alterations Can Continue	A	N/A	2								S	NRC: None.
(SRO/EC) Review Unit 2 LTOP Verification	A	N/A	3								~~E~~ S	NRC: Update Cover Sheet for SRO KA Importance Rating of 4.6 in accordance with Rev 3 of NUREG 1122. Response: Corrected cover sheet as noted above.
(SRO/RC) Authorize a gaseous radioactive waste release	A	N/A	2		X						~~E~~ S	NRC: A copy of reference OHP-4021-023-002, Attachment 2 completed through step 4.4 was not included with the JPM. How will step 4.2.5b be filled out which indicates that it was verified the release was approved in Section 2 of Data Sheet 1, since the Data Sheet is intentionally missing the Environmental manager signature? What is the purpose of including with the JPM the corrected copy of Data 1 which has the Environment approval signature? How is this supposed to be used with this JPM? Response: Added a completed copy of Attachment 2 to the JPM. Step 4.2.5.b is incorrectly marked N/A indicating the operator thought the release was approved even though the signature block is blank. The correctly marked Data Sheet 1 with an Environmental signature included would be used in accordance with the Cue on page 4. The purpose of this action is to require the Candidate to continue the review after the first error is found to ensure completion of sufficient Critical Steps.
(SRO/RC) Determine PAR requirements	A	N/A	3								S	**TIME CRITICAL** NRC: None.

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1 JPM # or Title	2 Type (S/P/A)	3 ALT (Y/N)	4 LOD (1–5)	5 JPM Errors LOD	REF	IC	TSK	CUE	CS	TL	6 U/E/S	7 Explanation
a. Full Length Operability test on Control Bank A	S	N	2								S	NRC: None.
b. Establishing Letdown **(ALT PATH)**	S	Y	3						X		~~E~~ S	NRC: The words, “are closed” are repeated in the performance standard for procedure step 2.b (verify letdown orifice valves – closed) 2-QRV-170 is incorrectly written as 2-***QVR***-170 in the performance standard associated with Attachment A Step 2.e. Response: Corrected performance standards for steps 2.b and 2.e as noted above.
c. Pressurizer Heater Capacity Check	S	N	2						X		~~E~~ S	NRC: The performance standard for procedure step 4.1.6 Record 11PHC Current, should say record amp reading from the 11PHC Current meter and not the 11***PHA*** Current meter. Performance standard for procedure steps 4.2.3 and 4.2.6 incorrectly call out HEATER GRC1, GRC2, GRC3 they should be GRA1, GRA2, GRA3. Response: Corrected performance standards for steps 4.1.6, 4.2.3, and 4.2.6 as noted above.
d. Switch HDP Alignment	S	N	3						X		~~E~~ S	NRC: Cover sheet references a different KA than the one listed on Form 3.2-2. Both KAs meet the intent of the JPM, select one and make the forms match. The performance standard for the critical step associated with procedure step 4.4.7.e.3 should indicate ***when 1-CRV-252 is CLOSED,*** the North Heater Drain Pump is placed in TRIP. Response: Adjusted KA on JPM cover sheet to match Form 3.2-2. Adjusted Performance standard as follows, including addition of a clarifying note: “NOTE: Stopping the Pump satisfied the Critical Step. Waiting for the pump discharge valve to close is not critical since no flow would pass through the valve even if partially open once the pump is stopped. STANDARD: (CS) When 1-CRV-252 is CLOSED, Candidate stops the North Heater Drain Pump by momentarily placing the control switch NORTH PUMP 1-PP-22N in TRIP”

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1 JPM # or Title	2 Type (S/P/A)	3 ALT (Y/N)	4 LOD (1–5)	5 JPM Errors LOD	REF	IC	TSK	CUE	CS	TL	6 U/E/S	7 Explanation
e. Establish Cooling Flow to a Reactor Coolant Pump **(RO-Only)**	S	N	2								S	NRC: None.
f. Perform actions in 4023-FR-Z.1 **(ALT PATH)** **(FREE SAMPLE)**	S	Y	3								~~E~~ S	NRC: "Annunciators have been silenced" is part of the initial conditions and should be moved to be the last line in the initial conditions. Adjusted as requested. The initiating cue should include a statement directing the applicant to reporting when the task is complete … e.g. "… starting at step 1. Inform the US when the task is complete." Adjusted as requested. The "STANDARD" for Critical Step FR-Z.1 3.d. RNO ("Manually align valve(s) as necessary.") indicates that the operator "manually opens 2-IMO-220 and 2-IMO221 by momentarily placing control switches in CLOSE" … is this a typo or how the system works? Typo The "NOTE" and "STANDARD" for step FR-Z.1 9. (JPM page 11) refer to "12, 13, & 14 SG pressures" and "12, 13, & 14 SG steam generators" … are these Unit 1 designations i.e. typos? Typo The "STANDARD" for step FR-Z.1 10. (JPM page 12) refers to "11 SG as faulted" is that a Unit 1 designation i.e. typo? Typo Response: All items above have been corrected as requested.
g. Remove a Failed PR Nuclear Instrument from Service **(ALT PATH)**	S	Y	3					X			~~E~~ S	NRC: Cover sheet references a different KA than the one listed on Form 3.2-2. Both KAs meet the intent of the JPM, select one and make the forms match. Please explain the status of the P-7, P-8, and P-10 interlocks in step 4. Cue indicates that P-7 and P-8 are not lit but P-10 is. The IC starts with the plant at 100%, why would there be a difference in the status of these interlocks with the plant at 100%? Response: Corrected Form 3.2-2 with KA used by the JPM, as it is the more appropriate KA. The status of the Permissive lights is correct for a 100% power lineup. The P-7 and P-8 permissives are met when power is less than their respective set point, whereas P-10 is met when power is greater than its set point. Reference Lesson Plan RO-C-01100 Pages 29-31 for more information.

Facility: D.C. Cook												Exam Date: 7/18-7/29/2022
1 JPM # or Title	2 Type (S/P/A)	3 ALT (Y/N)	4 LOD (1–5)	5 JPM Errors							6 U/E/S	7 Explanation
				LOD	REF	IC	TSK	CUE	CS	TL		
h. Switch CW Pumps	S	N	2								S	NRC: None.
i. Local Actions for ATWS	P	N	2								S	NRC: None.
j. Locally control RHR and CVCS Air Operated valves **(ALT PATH)**	P	Y	3					X	X		~~E~~ S	NRC: The cue/performance standard associated with procedure step g.5 is confusing. The actual procedure indicates that this step's intention is to fail OPEN the valve (1-IRV-311) which was not able to be locally operated. The actual procedure step says to verify 1-IRV-311 is OPEN. But the cue/performance standard says the applicant is expected to verify it CLOSED. Please explain difference. In addition, the Task Standard requires 1-IRV-311 to be failed OPEN. Response: Modified performance standard for step 4.g for the candidate to verify IRV-311 OPEN and the cue to be that the valve is OPEN
k. Verify Control Room Pressurization Alignment	P	N	2			X		X			~~E~~ S	NRC: Initiating cue (on both the examiner and the applicant's copies) has a grammatical error (The Unit Supervisor ***has directs***). Cues are provided for counter-clockwise motion of the control switches, but there does not appear to be a time when CCW motion is expected. What is the purpose of these cues? Initial motion of the switches is full clockwise through AUTO and into CLOSE, but the next move is again clockwise motion to AUTO. Should this be CCW motion? Was this the purpose of the cues and the performance standards are incorrect? Response: Corrected initiating cues (both locations). Corrected performance standard for step 4.3.1.a and 4.3.2.a to indicate the candidate rotates the switch CCW.

Form 2.3-3 Instructions for Completing the JPM Table

1. Enter the JPM number and/or title.

2. Enter the type of JPM—(S)imulator, (P)lant, or (A)dministrative.

3. Enter (Y)es or (N)o for an Alternate Path JPM.

4. Rate the level of difficulty (LOD) of each JPM using a scale of 1–5 (easy–difficult). A JPM containing less than two critical steps, a JPM that tests solely for recall or memorization, or a JPM that involves directly looking up a single correct answer is likely LOD = 1 (too easy). Conversely, a JPM with over 30 steps or a JPM that takes more than 45 minutes to complete is likely LOD = 5 (too difficult).

5. Check the appropriate block for each JPM error type, using the following criteria:

   - LOD = 1 or 5 is unsatisfactory (U).
   - REF: The JPM lacks required references, tools, or procedures (U).
   - IC: The JPM initial conditions are missing or the JPM lacks an adequate initial cue (U).
   - CUE: The JPM lacks adequate evaluator cues to allow the applicant to complete the task, or the evaluator cues are subjective or leading (U).
   - TSK: The JPM lacks a task standard or lacks completion criteria for a task standard (U).
   - CS: The JPM contains errors in designating critical steps, or the JPM lacks an adequate performance standard for a critical step (U).
   - TL: The JPM validation times are unreasonable, or a time-critical JPM lacks a completion time (U).

6. Mark the JPM as unsatisfactory (U), satisfactory (S), or needs enhancements (E). A JPM is (U) if it has one or more (U) errors as determined in step 5. Examples of enhancements include formatting, spelling, or other minor changes.

7. Briefly describe any JPM determined to be unsatisfactory (U) or needing enhancement (E). Save initial review comments and detail subsequent comment resolution so that each exam bound JPM is marked by a satisfactory (S) resolution on this form.

Form 2.3-3 Operating Test Review Worksheet (Scenarios)

Facility: D.C. Cook	**Scenario: 1**					**Exam Date: 7/18-7/29/2022**	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
1 VCT High Level Control Xmtr fails Low					X	~~E~~ S	NRC: TRM 8.1.1 Condition A is required to be entered since RWST auto transfer when VCT level fails off scale is adversely affected. This is a valid TS event and the 3.3-1 and 3.4-1 documents should be properly updated. Per NUREG 1021 TS events include those in which the TS and/or TRM are entered (ODCM entries do not count). This would also make the new event 3 – RCS Flow Instrument failure unnecessary. Response: Updated Form 3.3-1 to indicate Event 1 as a Tech Spec event. Deleted Event 3 and renumbered subsequent events. Revised applicable copies of From 3.4.1. Updated Form 3.3-2 to delete Event 3 and renumber subsequent events.
2 FW Flow Inst Fails High					X	S	Manual control of 21 SG level by BOP
3 Swap Stator Cooling Water Pumps						S	Normal event for BOP
4 Stator Cooling Water Leak; Rapid Power Reduction; VCT Level in Manual			X			~~E~~ S	Reactivity event for ATC Manual control of VCT Level and Pressure by ATC NRC: If the ATC elects to take the 2-QRV-303 to the CVCS HU TANK position this may not classify as manually controlling VCT level and pressure. Will need to assess during onsite validation how much ATC monitoring and adjusting is required. Response: If the ATC elects to open 2-QRV-303 to maintain VCT level and pressure, the valve will have to be closed and re-opened repeatedly during the boration in order to keep level and pressure within the control band.

Facility: D.C. Cook		Scenario: 1				Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
5 P_{PZR} Controlling Channel Fails High; Spray Valve Fails Open (EOP entry)						S	
6 LBLOCA; RHR Auto Start Failure; Xfer to Cold Leg Recirc				XX		S	

Facility: D.C. Cook		Scenario: 2 (FREE SAMPLE)				Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
1 Start DG2CD						S	Normal event for BOP
2 Loss of DG2CD ESW					X	S	
3 Uncontrolled Rod Motion						S	Reactivity event for ATC Manual control of control rods by ATC
4 L_{PZR} Fails High					X	~~E~~ S	Manual control of PZR Level by ATC NRC: TS 3.3.4, Condition A for Remote Shutdown Instrumentation only requires 1 channel of pressurizer level to be operable. Is the failed channel the only pressurizer level indication at the Remote Shutdown Panel? Response: Yes, this is the only channel that provides for remote shutdown monitoring.
5 Loss of Containment Cooling					X	S	

Facility: D.C. Cook		Scenario: 2 (FREE SAMPLE)				Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS	3 U/E/S	4 Explanation
6 21 RCP Trip; Auto RX Trip Failure				X		~~E~~ S	NRC: Please provide additional detail related to the UFSAR, about how DNB "could" occur if the reactor is not tripped "promptly" and how "within one minute" is "promptly" enough to prevent DNB. Why is "within one minute" the right amount of time and what documentation makes it defendable as the bounding time for the Critical Task? … Is the "one minute" written into the UFSAR? … is there an engineering/Westinghouse evaluation stating why within one minute is the proper bounding time criteria? Response: Analysis provided in the Critical Task description on Form 3.3-1. Time of 1 minute mutually agreed to by facility and Chief Examiner.
7 LOOP, Loss of T21A				X		~~E~~ S	NRC: Please provide additional detail that describes the bounding nature of "implementation of recovery strategy for Extended Loss of AC Power (ELAP)". Does this mean before the "ELAP Procedure" is implemented … or before at least one action is taken within the "ELAP Procedure" … or something else? What is/are the entry criteria for the "ELAP Procedure"? Response: Description provided on Form 3.3-1 for ELAP transition from ECA 0.0 and how this can be a measuring tool for operator performance.

Facility: D.C. Cook	Scenario: 3					Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
1 Swap NESW Pumps						S	Normal event for BOP
2 Power Reduction to 95%						S	Reactivity event for ATC
3 P_{PZR} Channel Fails Low					X	S	Manual control of PZR Pressure by ATC
4 Bus Duct Cooling Fan Failure with standby auto start failure						S	Manual start of bus duct cooling fan by BOP
5 Main Turbine Oil Temp Controller Failure						S	Manual control of Turbine lube oil temperature by BOP
6 SG Tube Leak					X	S	Manual control of PZR Level by ATC

Facility: D.C. Cook	Scenario: 3					Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
7 SG Tube Rupture (EOP entry); Steam Dump System Failure; PZR Spray Valve Fails Open During RCS Depress				XXXX		~~E~~ S	NRC: Steps to isolate feedwater to SG22 in step 8 on page 16 of Form 3.3-2 and in if required BOP action on page 18 of 3.3-2 should be marked "**(CT1 related action)**". These actions should also be specifically listed in Form 3.3-2 such as – Close AFW valves to affected SG: 2-FMO-221 and 2-FMO-222 Response: Changed RO/BOP action for step 8 on page 16 of Form 3.3-2 to read as follows: When 22 SG narrow range level is greater than 13%, then close AFW valves to 22 SG 2-FMO-221 and 2-FMO-222 (CT1 related action) Changed BOP action on page 18 of Form 3.3-2 to read as follows: When 22 SG narrow range level is greater than 13%, then close AFW valves to 22 SG 2-FMO-221 and 2-FMO-222 if not previously performed (CT1 related action)

Facility: D.C. Cook	**Scenario: 4**					**Exam Date: 7/18-7/29/2022**	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
1 Shift Condensate Booster Pumps						S	Normal event for BOP
2 Raise Reactor Power						S	Reactivity event for ATC
3 Charging Flow Instrument Fails Low					X	~~E~~ S	Manual control of Charging Flow/ Pressurizer Level by ATC NRC: This is a TS event as a TRM entry is required. Update Form 3.3-1 and 3.4-1 to indicate as such. Response: Updated Form 3.3-1 to indicate Event 3 as a Tech Spec event. Updated applicable Form 3.4-1 forms to add the event as a Tech Spec event and updated total.

Facility: D.C. Cook	Scenario: 4	Exam Date: 7/18-7/29/2022					
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
4 Running CCW Pump Fails (w/ Stby Pump Auto Start Failure)					X	~~E~~ S	Manual start of Standby CCW pump by BOP NRC: What will be the alignment of the standby CCW pump? Will it be aligned to the simulator unit (unit 2)? The east CCW pump is clearly inoperable. The west CCW pump is also inoperable (unless your TS basis say something else) since it did not auto start. Therefore, if the standby pump is not aligned then it can be argued that there are 2 required trains inoperable and therefore the crew would be in TS 3.0.3 until the spare pump is aligned and in standby. Response: The standby CCW pump is in Automatic prior to the event. It is dedicated as a Unit 2 pump. After manual start the pump will supply the Unit 2 CCW headers through discharge cross-tie valves without further operator action. Tech Spec Bases 3.7.7 (ITSA-U2-Bases.pdf) Page 488 states the following: "The pumps are also started on a low header pressure signal, but this is not required for OPERABILITY of the CCW System" Since the failure in the scenario is failure of the pressure switch, the West CCW pump remains operable.
5 VCT Level Channel Fails High					X	~~E~~ S	NRC: This is a TS event as a TRM entry is required. Update Form 3.3-1 and 3.4-1 to indicate as such. Response: Updated Form 3.3-1 to indicate Event 5 as a Tech Spec event. Updated applicable Form 3.4-1 forms to add the event as a Tech Spec event and updated total.
6 Controlling SG Level Channel Fails High					X	S	Manual control of SG Level by BOP

Facility: D.C. Cook		Scenario: 4				Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS	3 U/E/S	4 Explanation
7 Loss of Feed ATWS Inadvertent SI Failure of AFW to start and the Turbine to Trip; Main Steam Isolation Failure				XX		~~E~~ S	NRC: For CT2 what is the minimum number of required AFW pumps under postulated plant conditions? Must all three pumps be started or is some lesser configuration acceptable if SG WR level remains above 17%? E-0, Attachment A the Form 3.3-2 indicates that the applicant should note that Containment Isolation Phase B is actuated. Why would Phase B be actuated and if it was wouldn't that require RCPs to be shutdown? Response: According to the ATWS analysis contained in the Westinghouse Emergency Response Guidelines Background (WERG-BD.pdf Page 2497), Auxiliary feed water flow is assumed to begin within 60 seconds at a rate of 1760 gpm (~880,000 pph). In order to achieve this flow rate, both MDAFPs and the TDAFP must be running. Therefore, in order to comply with the ATWS analysis, all three AFPs must be started if available. In this scenario, all three pumps are available. Modified Form 3.3-2 page 17 to the following: Check Containment Isolation Phase B status-Determine Containment Isolation Phase B is not actuated.
8 SI Termination						S	

Facility: D.C. Cook	Scenario: SPARE SCENARIO					Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
1 Shut down the East MFP						~~E~~ S	Normal event for BOP NRC: In event setup it does not indicate that steps 4.1-4.7 of Attachment 1 should be N/A'd (4.9-4.12 are to be marked with N/A per the setup) yet they are not performed as per Form 3.3-2 expected BOP actions. Should steps 4.1-4.7 also be N/A'd for the crew? Response: Changed setup conditions to the following: Provide a marked up copy of 2-OHP-4021-055-004, Attachment 1, as follows: • Steps 4.1, 4.4, and 4.7.1 complete • Steps 4.2, 4.3, 4.5, 4.6, and 4.7.2 marked as N/A • Steps 4.9 through 4.12 marked as N/A. Modified Form 3.3-1 Turnover information as follows: Complete shutdown of the East MFP using 2-OHP-4021-055-004, Feed Pump Turbine Shutdown, Attachment 1, starting at step 4.8, and then perform a power reduction using 2-OHP-4021-001-003, Power Reduction, to 50%. Performance of actions to break vacuum and perform an expedited cooldown are not required
2 Running Boric Acid Transfer Pump Failure						S	
3 Power Reduction						S	Reactivity event for ATC

Facility: D.C. Cook	Scenario: SPARE SCENARIO					Exam Date: 7/18-7/29/2022	
1 Scenario Event ID/Name:	2 Scenario event errors					3 U/E/S	4 Explanation
	Realism/ Credibility	Performance Standards	Verifiable Actions	Critical Task	TS		
4 Loss of Plant Air Compressor						S	
5 Steam Flow Channel Fails High					X	S	Manual control of SG Level by BOP
6 RCS Leak into an Accumulator					X	S	Manual control of Charging Flow/ Pressurizer Level by ATC
7 Inadvertent Feed Water isolation (EOP entry); Auto Turbine Trip Failure; Main Steam Isolation Failure				X		S	
8 Loss of AFW Suction/Loss of Secondary Heat Sink				X		S	

Form 2.3-3 Instructions for Completing the Scenario Table

1. For each scenario, enter the scenario event names and descriptions.

2. Review the individual events contained in each scenario, and identify and mark event errors:

   - The scenario guide event description is not realistic/credible—unsatisfactory (U).

   - The scenario guide event description lacks adequate crew/operator performance standards—needs enhancement (E).

   - The scenario guide event description lacks verifiable actions for a credited normal event, reactivity event instrument/component malfunction, or technical specification (TS) event (or a combination of these) (U).

   - The scenario guide event description incorrectly designates an event as a critical task (i.e., a noncritical task labeled as critical or a critical task labeled as noncritical). This includes critical tasks that do not meet the critical task criteria (i.e., the critical task does not have a measurable performance standard) (U).

   - The scenario guide event description incorrectly designates entry into TS actions when not required or does not designate entry into TS actions when required (U).

3. Based on the outcome in step 2, mark the scenario event as unsatisfactory (U), satisfactory (S), or needs enhancements (E). An event is (U) if it has one or more (U) errors as determined in step 2. Examples of enhancements include formatting, spelling, or other minor changes.

4. Briefly describe any scenario event determined to be unsatisfactory (U) or needing enhancement (E). Save initial review comments and detail subsequent comment resolution so that each exam bound scenario event is marked by a satisfactory (S) resolution on this form.