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PG&E Letter DCL-22-048

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Docket No. 50-275, OL-DPR-80 Docket No. 50-323, OL-DPR-82 Diablo Canyon Units 1 and 2

10 CFR 50.46 Annual Report of Emergency Core Cooling System Evaluation Model Changes for Peak Cladding Temperature for 2021

References:

- PG&E Letter DCL-20-043, "Thirty-Day Notification Report of Significant Emergency Core Cooling System Evaluation Model Changes That Affect Peak Cladding Temperature and 10 CFR 50.46 Annual Report of Emergency Core Cooling System Evaluation Model Changes for Peak Cladding Temperature for 2019," dated May 20, 2020 [ADAMS Accession No. ML20141L498]
- NRC Letter "Diablo Canyon Nuclear Power Plant, Units 1 and 2 Issuance of Amendment Nos. 234 and 236 to Revise Technical Specification 5.6.5b, 'Core Operating Limits Report (COLR),' for Full Spectrum Loss-of-Coolant Accident Methodology (EPID L-2018-LLA-0730)," dated January 9, 2020
- 3. WCAP-13451, "Westinghouse Methodology for Implementation of 10 CFR 50.46 Reporting," October 1992

Dear Commissioners and Staff:

Pursuant to 10 CFR 50.46, Pacific Gas and Electric Company (PG&E) hereby submits this annual report of changes in the Westinghouse emergency core cooling system evaluation model that affects peak cladding temperature (PCT) calculations for Diablo Canyon Power Plant (DCPP), Units 1 and 2.

There are no changes from the PCT results provided in Reference 1; therefore, there are no enclosures provided in this letter. The sum of the PCT from the most recent analysis of record, using the acceptable evaluation model approved for DCPP in Reference 2, and the estimates of the net PCT effect for changes and errors remains well within the 2200°F limit specified in 10 CFR 50.46.

There is one general code maintenance related change to the 2016 Westinghouse FULL SPECTRUMTM LOCA Evaluation Model applicable for DCPP. Various changes have been made to enhance the usability of codes and to streamline future analyses. Examples of these changes include improving the input diagnostic checks, enhancing the code output, optimizing active coding, and eliminating inactive coding. These changes represent discretionary changes that will be implemented on a forward-fit basis in accordance with Section 4.1.1 of Reference 3. The nature of these changes has an estimated PCT impact of 0°F.

PG&E makes no new or revised regulatory commitments (as defined by NEI 99-04) in this letter.

If you have questions regarding this submittal, please contact Mr. James R. Morris, Regulatory Services Manager, at (805) 545-4609.

Sincerely,

Jeremy D. Cobbs

Manager, Nuclear Engineering

July13, 2022

Date

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cc: Scott A. Morris, NRC Region IV Administrator

Mahdi O. Hayes, NRC Senior Resident Inspector

Diablo Distribution