SYSTEM DATA					
	MAX GPM	PSIG	° F	BY	REMARKS
1	087	100	120	KRC	
2	967	100	120	KRC	
4	262	80	145	KRC	
5	240	80	125	KRC	START-UP ONLY
6	150	80	145	KRC	
7	5	80	145	KRC	
8	120	100	120	KRC	
9	120	130	120	KRC	
10	40	80	145	KRC	
11	087	70	125	KRC	

* THIS TEMP IS A MAXIMUM EXPECTED FOR 95°F SERVICE WATER TEMP. AND LOSS OF ONE CCW HEAT EXCHANGER DURING PLANT COOL DOWN MODE.

CC	19500	19613
CC	9500	9699
SYSTEM SUFFIX	FIRST NO.	LAST NO.
VALVE NUMBERING		

9691-9699 DWG. E-911-108

HYDRO TEMP - COLD WATER										
46-47	150	168	2405	650	2 HR	3188	DTK			
45	150	168	298	188	2 HR	258	DTK			
2,13 48-55	120	168	158	188	2 HR	198	DTK			
■	PSIG	°F	PSIG	°F	DURA	HY-DRO	BY	CHKD	REMARKS	
	NORMAL		UPSET							
DESIGN DATA										

CONTAINMENT ISOLATION PHASE "B" SIGNAL

USE NEXT (N)

ESSENTIAL

INSIDE REACTOR CONTAINMENT / OUTSIDE REACTOR CONTAINMENT

REACTOR COOLANT PUMPS (3)

THERMAL BARRIER XPP-30A-RC / XPP-30B-RC / XPP-30C-RC

UPPER BEARINGS LOWER 'A' / 'B' / 'C'

REACTOR COOLANT DRAIN TANK HEAT EXCHANGER XHE-12-WL

EXCESS LETDOWN HEAT EXCHANGER XHE-9-CS

COMPONENT COOLING BOOSTER PUMPS 3-100% 120 GPM 110' TDH

LO START STANDBY PUMP

ITL-LO PRESSURE

HI FLOW CLOSE

ALARM DEFEAT ITL

NOTES:

1. ALL PIPING TO BE ANS SAFETY CLASS 2b EXCEPT AS NOTED.
2. ALL NUC SAFETY CLASS PIPING TO BE LINE SPEC 152N UNLESS OTHERWISE NOTED.
3. ALL NON-NUC S.C. PIPING TO BE LINE SPEC 151X UNLESS NOTED OTHERWISE.
4. DUAL ORIFICES - ONE FLOW RESTRICTING THE OTHER BLIND FOR USE IN LEAK TESTING ISOLATION VALVE.
5. ALL ◇ STATUS LIGHTS INDICATE BOTH OPEN AND CLOSED, UNLESS NOTED OTHERWISE.

RN 18-023

THIS IS A NUCLEAR SAFETY RELATED DOCUMENT. NO DEVIATION SHALL BE INITIATED OR PERFORMED WITHOUT PRIOR DOCUMENTATION AND WRITTEN APPROVAL

DRAWING LEGIBILITY CLASS 1

NO.	DATE	BY	REVISION	CKD. BY	LE APPROVAL
24	5/5/02	RHM	REVISED PER ECR-50263	DDJ	LWS
23	6/23/99	DDJ	REVISED PER CGSS-99-0332	MGR	ALB
28	6/29/18	JMR	REVISED PER ETBT-458	DKC	AL
27	04/02/12	DDJ	REVISED PER ECR-50239	MGR	GFK
26	04/14/08	RHM	REVISED PER ECR-71263	MGR	RHM
25	2/11/08	JMR	REVISED PER ECR-50239	MGR	DJ

FSAR Figure 9.2-5

SOUTH CAROLINA ELECTRIC & GAS COMPANY

VIRGIL C. SUMMER NUCLEAR STATION

PIPING — SYSTEM FLOW DIAGRAM

COMPONENT COOLING SYSTEM

INSIDE REACTOR BUILDING

DESIGN ENGINEERING

V. C. SUMMER NUCLEAR STATION JENKINSVILLE, S. C.

MADE	CHECKED	LE APPROVAL
A.V.N.	M.G.R.	R.S.B.

SCALE: NO SCALE — D-302-612 — 28 (DRAWING NUMBER / SHT. NUMBER / REV)