



Ho Nieh  
Site Vice President  
Vogtle Units 1-2

7821 River RD  
Waynesboro GA, 30830  
706-848-0001 tel  
706-848-3321 fax

June 24, 2022

Docket Nos.: 50-424

NL-22-0433

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D. C. 20555-0001

Vogtle Electric Generating Plant Unit 1  
Licensee Event Report 2022-001-00  
Manual Reactor Trip due to loss of a Main Feed Pump

Ladies and Gentlemen:

In accordance with the requirements of 10 CFR 50.73(a)(2)(iv)(A), Southern Nuclear Operating Company is submitting the enclosed Licensee Event Report, 2022-001-00 for Vogtle Electric Generating Plant Unit 1.

This letter contains no NRC commitments. If you have any questions, please contact Jeff Deal at (706) 848-3792.

Respectfully submitted,

A handwritten signature in blue ink, appearing to be "Ho Nieh", written over a light blue horizontal line.

Ho Nieh  
Site Vice President  
Vogtle 1&2

HN/KCW

Enclosure: Unit 1 Licensee Event Report 2022-001-00

cc: Regional Administrator  
NRR Project Manager – Vogtle 1 & 2  
Senior Resident Inspector – Vogtle 1 & 2  
RType: CVC7000

**Vogtle Electric Generating Plant Unit 1  
Licensee Event Report 2022-001-00  
Manual Reactor Trip due to loss of a Main Feed Pump**

**Enclosure**

**Unit 1 Licensee Event Report 2022-001-00**



**LICENSEE EVENT REPORT (LER)**

(See Page 3 for required number of digits/characters for each block)  
(See NUREG-1022, R.3 for instruction and guidance for completing this form  
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to [Infocollections.Resource@nrc.gov](mailto:Infocollections.Resource@nrc.gov), and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk ail: [oira\\_submission@omb.eop.gov](mailto:oira_submission@omb.eop.gov). The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.

<b>1. Facility Name</b> Vogtle Electric Generating Plant	<b>2. Docket Number</b> 05000                      00424	<b>3. Page</b> 1 OF 2
---	---	--------------------------

**4. Title**  
Manual Reactor Trip due to loss of a Main Feed Pump

5. Event Date			6. LER Number			7. Report Date			8. Other Facilities Involved	
Month	Day	Year	Year	Sequential Number	Revision No.	Month	Day	Year	Facility Name	Docket Number
05	03	2022	2022	- 001 -	00	06	24	2022	N/A	05000
									Facility Name	Docket Number
									N/A	05000

<b>9. Operating Mode</b> 1	<b>10. Power Level</b> 100
-------------------------------	-------------------------------

**11. This Report is Submitted Pursuant to the Requirements of 10 CFR §: (Check all that apply)**

<b>10 CFR Part 20</b>	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.36(c)(2)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<b>10 CFR Part 73</b>
<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.69(g)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(4)
<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> 73.71(a)(5)
<input type="checkbox"/> 20.2203(a)(2)(i)	<b>10 CFR Part 21</b>	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> 73.77(a)(1)(i)
<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 21.2(c)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> 73.77(a)(2)(i)
<input type="checkbox"/> 20.2203(a)(2)(iii)	<b>10 CFR Part 50</b>	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	<input type="checkbox"/> 73.77(a)(2)(ii)
<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)	

OTHER (Specify here, in abstract, or NRC 366A).

**12. Licensee Contact for this LER**

<b>Licensee Contact</b> Jeff Deal (Licensing Manager)	<b>Phone Number (Include area code)</b> 706-848-3792
--	---

**13. Complete One Line for each Component Failure Described in this Report**

Cause	System	Component	Manufacturer	Reportable to IRIS	Cause	System	Component	Manufacturer	Reportable to IRIS
X	SJ	SC	GE	Y	N/A				

**14. Supplemental Report Expected**

<input checked="" type="checkbox"/> No	<input type="checkbox"/> Yes (If yes, complete 15. Expected Submission Date)
--	--

**15. Expected Submission Date**

Month	Day	Year

**16. Abstract (Limit to 1560 spaces, i.e., approximately 15 single-spaced typewritten lines)**

On May 3, 2022 at 15:41 EDT, with Unit 1 in Mode 1 and 100 percent power, the Control Room Operators observed lowering steam generator water levels and manually tripped the reactor. The lowering steam generator levels were caused by a malfunction of the 1B Main Feed Pump control system. The trip was not complex, and all systems responded normally post trip. The Operators stabilized the plant in Mode 3, and decay heat was removed by the steam generators, through the steam dumps to the condenser. Unit 2 was not affected. The operating crew performed as expected. The Main Feed Pump control system was repaired and the unit was restarted.

Due to the manual Reactor Protection System (RPS) actuation and the automatic actuation of Auxiliary Feedwater (AFW) System, this event is being reported under 10 CFR 50.73(a)(2)(iv)(A).



**LICENSEE EVENT REPORT (LER)  
CONTINUATION SHEET**

(See NUREG-1022, R.3 for instruction and guidance for completing this form  
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to [Infocollects.Resource@nrc.gov](mailto:Infocollects.Resource@nrc.gov), and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503; e-mail: [oir\\_submission@omb.eop.gov](mailto:oir_submission@omb.eop.gov). The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.

1. FACILITY NAME	2. DOCKET NUMBER	3. LER NUMBER		
		YEAR	SEQUENTIAL NUMBER	REV NO.
Vogtle Electric Generating Plant	05000-00424	2022	001	00

**NARRATIVE**

**NARRATIVE**

**A. Event Description**

On May 3, 2022 at 15:41 EDT, with Unit 1 in Mode 1 and 100 percent power, the Control Room Operators observed lowering steam generator water levels and manually tripped the reactor. The lowering steam generator levels were caused by a malfunction of the 1B Main Feed Pump control system [JK]. The trip was not complex, and all systems responded normally post trip. The components that failed were General Electric IS420UCSBH1A control modules.

There were no Structures, Systems, or Components (SSCs) that were out of service at the beginning of the event that contributed to the event.

Due to the actuation of the Reactor Protection System (RPS)[JC] and the actuation of the Auxiliary Feedwater (AFW)[BA] System, this event is being reported in accordance with 10 CFR 50.73(a)(2)(iv)(A).

**B. Cause of Event**

The cause of the event was a failure of the 1B Main Feed Pump control system which caused the pump to slow down, resulting in lowering main feedwater flow to the steam generators.

**C. Safety Assessment**

The Main Feed Pumps [SJ] serve no safety-related function for the unit. When the Operators manually tripped the plant, all the appropriate safety systems responded as designed. The AFW system actuated as designed and all control rods [AA] fully inserted into the core. The operating crew performed as expected. The event was within the analysis of the Updated Final Safety Analysis Report, Chapter 15, and there was no release of radioactivity. This event did not pose a threat to the health and safety of the public, therefore the safety significance of this issue is very low.

**D. Corrective Actions**

The control system for the Main Feed Pump was repaired.

**E. Previous Similar Events**

None.