ATTACHMENT 2

Point Beach Units 1 and 2
License Amendment Request to Revise Technical Specifications
to Adopt Risk Informed Completion Times TSTF-505, Revision 2,
"Provide Risk-Informed Extended Completion Times - RITSTF Initiative 4b"

Proposed Technical Specification Changes (Mark-Up)

(45 pages follow)

INSERT new Completion Time
Example 1.3-8 from next 2 pages.

1.3 Completion Times

IMMEDIATE COMPLETION TIME	When "Immediately" is used as a Completion Time, the Required Action should be pursued without delay and in a controlled manner.

INSERT these next pages (2) before "Immediate Completion Time" definition.

EXAMPLES (continued)

EXAMPLE 1.3-8

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One subsystem inoperable.	A.1 Restore subsystem to OPERABLE status.	7 days OR In accordance with the Risk Informed Completion Time Program
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3. AND B.2 Be in MODE 5.	6 hours 36 hours

When a subsystem is declared inoperable, Condition A is entered. The 7 day Completion Time may be applied as discussed in Example 1.3-2. However, the licensee may elect to apply the Risk Informed Completion Time Program which permits calculation of a Risk Informed Completion Time (RICT) that may be used to complete the Required Action beyond the 7 day Completion Time. The RICT cannot exceed 30 days. After the 7 day Completion Time has expired, the subsystem must be restored to OPERABLE status within the RICT or Condition B must also be entered.

The Risk Informed Completion Time Program requires recalculation of the RICT to reflect changing plant conditions. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.

If the 7 day Completion Time clock of Condition A has expired and subsequent changes in plant condition result in exiting the applicability of the Risk Informed Completion Time Program without restoring the inoperable subsystem to OPERABLE status, Condition B is also entered and the Completion Time clocks for Required Actions B.1 and B.2 start.

If the RICT expires or is recalculated to be less than the elapsed time since the Condition was entered and the inoperable subsystem has not been restored to OPERABLE status, Condition B is also entered and the Completion Time clocks for Required Actions B.1 and B.2 start. If the inoperable subsystems are restored to OPERABLE status after Condition B is entered, Condition A is exited, and therefore, the Required Actions of Condition B may be terminated.

3.3 INSTRUMENTATION

3.3.1 Reactor Protection System (RPS) Instrumentation

LCO 3.3.1 The RPS instrumentation for each Function in Table 3.3.1-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.1-1.

[OR

In accordance with the Risk Informed Completion Time Program]

ACTIONS

----------NOTE----------
Separate Condition entry is allowed for each Function.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one or more required channels or trains inoperable.	A.1 Enter the Condition referenced in Table 3.3.1-1 for the channel(s) or train(s).	Immediately
B. One Manual Reactor Trip channel inoperable.	B.1 Restore channel to OPERABLE status. ~~OR~~ ~~B.2 Be in MODE 3.~~	48 hours ~~54 hours~~
C. One Manual Reactor Trip channel inoperable.	C.1 Restore channel to OPERABLE status. OR C.2 Open reactor trip breakers (RTBs).	48 hours 49 hours

(continued)

[OR

In accordance with the Risk Informed Completion Time Program]

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. One channel inoperable.	D.1 Place channel in trip. ~~OR~~ ~~D.2 Be in MODE 3.~~	1 hour ~~7 hours~~
E. One channel inoperable.	E.1 Place channel in trip. ~~OR~~ ~~E.2 Reduce THERMAL POWER to < P-7.~~	6 hours ~~12 hours~~
F. One Intermediate Range Neutron Flux channel inoperable.	F.1 Reduce THERMAL POWER to < P-6. OR F.2 Increase THERMAL POWER to > P-10.	24 hours 24 hours
G. Two Intermediate Range Neutron Flux channels inoperable.	G.1 Suspend operations involving positive reactivity additions. AND G.2 Reduce THERMAL POWER to < P-6.	Immediately 2 hours
H. One Source Range Neutron Flux channel inoperable.	H.1 Suspend operations involving positive reactivity additions.	Immediately

(continued)

[OR

In accordance with the Risk Informed Completion Time Program]

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
I. Two Source Range Neutron Flux channels inoperable.	I.1	Open RTBs.	Immediately
J. One Source Range Neutron Flux channel inoperable.	J.1	Restore channel to OPERABLE status.	48 hours
	OR		
	J.2	Open RTBs.	49 hours
K. One channel inoperable.	K.1	Place channel in trip.	1 hour
	~~OR~~		
	~~K.2~~	~~Reduce THERMAL POWER to < P-7.~~	~~7 hours~~
L. One Reactor Coolant Flow-Low (Single Loop) channel inoperable.	L.1	Place channel in trip.	1 hour
	~~OR~~		
	~~L.2~~	~~Reduce THERMAL POWER to < P-8.~~	~~5 hours~~
M. One Reactor Coolant Pump Breaker Position (Single Loop) channel inoperable.	M.1	Restore channel to OPERABLE status.	1 hour
	~~OR~~		
	~~M.2~~	~~Reduce THERMAL POWER to < P-8.~~	~~5 hours~~

(continued)

[OR

In accordance with the Risk Informed Completion Time Program]

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
N. One inoperable channel.	N.1 Restore channel to OPERABLE status. ~~OR~~ ~~N.2 Reduce THERMAL POWER to < P-7.~~	1 hour ~~7 hours~~
O. One turbine trip channel inoperable.	O.1 Place channel in trip. ~~OR~~ ~~O.2 Reduce THERMAL POWER to < P-9.~~	1 hour ~~5 hours~~
P. One train inoperable.	----------NOTE---------- One train may be bypassed for up to 8 hours for surveillance testing provided the other train is OPERABLE. ---------- P.1 Restore train to OPERABLE status. ~~OR~~ ~~P.2 Be in MODE 3.~~	 6 hours ~~12 hours~~

(continued)

[OR

In accordance with the Risk Informed Completion Time Program]

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
Q. One RTB inoperable.	----------NOTE---------- One RTB may be bypassed for up to 8 hours provided the other RTB is OPERABLE. ----------	
	Q.1 Restore RTB to OPERABLE status.	1 hour
	~~OR~~	
	~~Q.2 Be in MODE 3.~~	~~7 hours~~
R. One or more channel(s) inoperable.	R.1 Verify interlock is in required state for existing unit conditions.	1 hour
	OR	
	R.2 Be in MODE 3.	7 hours
S. One or more channel(s) inoperable.	S.1 Verify interlock is in required state for existing unit conditions.	1 hour
	OR	
	S.2 Be in MODE 2.	7 hours
T. One RTB or trip mechanism for one RTB inoperable.	T.1 Restore RTB or RTB trip mechanism to OPERABLE status.	48 hours
	OR	
	T.2 Open RTBs.	49 hours

(continued)

[OR

In accordance with the Risk Informed Completion Time Program]

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
U. One trip mechanism inoperable for one RTB.	U.1	Restore ~~inoperable~~ trip mechanism to OPERABLE status.	48 hours
	~~OR~~		
	~~U.2~~	~~Be in MODE 3.~~	~~54 hours~~
V. One reactor trip bypass breaker (RTBB) or trip mechanism for one RTBB inoperable.	V.1	Restore RTBB or RTBB trip mechanism to OPERABLE status.	1 hour
	OR		
	V.2	Be in MODE 3.	7 hours
W. One reactor trip bypass breaker (RTBB) or trip mechanism for one RTBB inoperable.	W.1	Restore RTBB or RTBB trip mechanism to OPERABLE status.	48 hours
	OR		
	W.2	Open RTBs and RTBBs.	49 hours
X. One train inoperable.	X.1	Restore train to OPERABLE status.	48 hours
	OR		
	X.2	Open RTBs.	49 hours

INSERT NEXT PAGE

INSERT - Section 3.3.1, RPS Instrumentation ACTIONS

Y. Required Action and associated Completion Time of Condition B, D, P, Q, or U not met.	Y.1	Be in MODE 3	6 hours

Z. Required Action and associated Completion Time of Condition E, K or N not met.	Z.1	Reduce THERMAL POWER to < P-7.	6 hours

AA. Required Action and associated Completion Time of Condition L or M not met.	AA.1	Reduce THERMAL POWER to < P-8.	4 hours

BB. Required Action and associated Completion Time of Condition O not met.	BB.1	Reduce THERMAL POWER to < P-9.	4 hours

Table 3.3.1-1 (page 1 of 8)
Reactor Protection System Instrumentation

	FUNCTION	APPLICABLE MODES	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	NOMINAL TRIP SETPOINT
1.	Manual Reactor Trip	1,2	2	B	SR 3.3.1.13	NA	NA
		3(a), 4(a), 5(a)	2	C	SR 3.3.1.13	NA	NA
2.	Power Range Neutron Flux						
	a. High	1,2	4	D	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.7(m) SR 3.3.1.11(m)	≤ 108% RTP	106% RTP
	b. Low	1(b), 2	4	D	SR 3.3.1.1 SR 3.3.1.8(m) SR 3.3.1.11(m)	≤ 27% RTP	20% RTP
3.	Intermediate Range Neutron Flux	1(b), 2(c)	2	F,G	SR 3.3.1.1 SR 3.3.1.8(m) SR 3.3.1.11(m)	≤ 43% RTP	25% RTP
4.	Source Range Neutron Flux	2(d)	2	H,I	SR 3.3.1.1 SR 3.3.1.8(m) SR 3.3.1.11(m)	≤ 3.0 E5 cps	1.5 E5 cps
		3(a), 4(a), 5(a)	2	I,J	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≤ 3.0 E5 cps	1.5 E5 cps
5.	Overtemperature ΔT	1,2	4	D	SR 3.3.1.1 SR 3.3.1.3 SR 3.3.1.6 SR 3.3.1.7(m) SR 3.3.1.11(m)	Refer to Note 1 (Page 3.3.1-18)	Refer to Note 1 (Page 3.3.1-18)
6.	Overpower ΔT	1,2	4	D	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	Refer to Note 2 (Page 3.3.1-19)	Refer to Note 2 (Page 3.3.1-19)

(continued)

(a) With Reactor Trip Breakers (RTBs) closed and Rod Control System capable of rod withdrawal.
(b) Below the P-10 (Power Range Neutron Flux) interlock.
(c) Above the P-6 (Intermediate Range Neutron Flux) interlock.
(d) Below the P-6 (Intermediate Range Neutron Flux) interlock.
(m) Table 3.3.1-1 Notes 3 and 4 are applicable

Table 3.3.1-1 (page 2 of 8)
Reactor Protection System Instrumentation

	FUNCTION	APPLICABLE MODES	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	NOMINAL TRIP SETPOINT
7.	Pressurizer Pressure						
	a. Low	1(e)	4	K	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≥ 1860 psig	1925 psig
	b. High	1,2	3	D	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≤ 2380 psig	2365 psig
8.	Pressurizer Water Level — High	1(e)	3	K	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≤ 85%	80%
9.	Reactor Coolant Flow-Low						
	a. Single Loop	1(f)	3 per loop	L	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≥91%	93%
	b. Two Loops	1(g)	3 per loop	K	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≥91%	93%
10.	Reactor Coolant Pump (RCP) Breaker Position						
	a. Single Loop	1(f)	1 per RCP	M	SR 3.3.1.13	NA	NA
	b. Two Loops	1(g)	1 per RCP	N	SR 3.3.1.13	NA	NA
11.	Undervoltage Bus A01 & A02	1(e)	2 per bus	K	SR 3.3.1.9 SR 3.3.1.10(m)	≥ 3120 V	3170 V

(continued)

(e) Above the P-7 (Low Power Reactor Trips Block) interlock.
(f) Above the P-8 (Power Range Neutron Flux) interlock.
(g) Above the P-7 (Low Power Reactor Trips Block) interlock and below the P-8 (Power Range Neutron Flux) interlock.
(m) Table 3.3.1-1 Notes 3 and 4 are applicable

Table 3.3.1-1 (page 3 of 8)
Reactor Protection System Instrumentation

	FUNCTION	APPLICABLE MODES	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	NOMINAL TRIP SETPOINT
12.	Underfrequency Bus A01 & A02	1(e)	2 per bus	E	SR 3.3.1.10(m)	≥ 55.0 Hz	57 Hz
13.	Steam Generator (SG) Water Level — Low Low	1,2	3 per SG	D	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≥ 29.5%	31%
14.	SG Water Level — Low	1,2	2 per SG	D	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≥ 11%	31%
	Coincident with Steam Flow/Feedwater Flow Mismatch	1,2	2 per SG	D	SR 3.3.1.1 SR 3.3.1.7(m) SR 3.3.1.11(m)	≤ 1 E6 lbm/hr	0.8 E6 lbm/hr
15.	Turbine Trip						
	a. Low Autostop Oil Pressure	1(j)	3	O	SR 3.3.1.14	NA	NA
	b. Turbine Stop Valve Closure	1(j)	2	O	SR 3.3.1.14	NA	NA
16.	Safety Injection (SI) Input from Engineered Safety Feature Actuation System (ESFAS)	1,2	2 trains	P	SR 3.3.1.13	NA	NA

(continued)

(e) Above the P-7 (Low Power Reactor Trips Block) interlock.
(j) Above the P-9 (Power Range Neutron Flux) interlock.
(m) Table 3.3.1-1 Notes 3 and 4 are applicable

Table 3.3.1-1 (page 4 of 8)
Reactor Protection System Instrumentation

FUNCTION	APPLICABLE MODES	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	NOMINAL TRIP SETPOINT
17. Reactor Trip System Interlocks						
a. Intermediate Range Neutron Flux, P-6	2(d)	2	R	SR 3.3.1.11 SR 3.3.1.12	≥ 4E-11 amp	1E-10 amp
b. Low Power Reactor Trips Block, P-7						
(1) Power Range Neutron Flux	1	4	S	SR 3.3.1.11 SR 3.3.1.12	≤ 13% RTP	10% RTP
(2) Turbine First Stage Pressure	1	2	S	SR 3.3.1.11 SR 3.3.1.12	≤ 12.8% turbine power	10% turbine power
c. Power Range Neutron Flux, P-8	1	4	S	SR 3.3.1.11 SR 3.3.1.12	≤ 38% RTP	35% RTP
d. Power Range Neutron Flux, P-9	1(k)	4	S	SR 3.3.1.11 SR 3.3.1.12	(h)	(i)
e. Power Range Neutron Flux, P-10	1,2	4	R	SR 3.3.1.11 SR 3.3.1.12	≥ 6% RTP and ≤ 12% RTP	9% RTP
18. Reactor Trip Breakers (RTBs)	1,2	2 trains	Q	SR 3.3.1.4	NA	NA
	3(a), 4(a), 5(a)	2 trains	T	SR 3.3.1.4	NA	NA
19. Reactor Trip Breaker Undervoltage and Shunt Trip Mechanisms	1,2	1 each per RTB	U	SR 3.3.1.4	NA	NA
	3(a), 4(a), 5(a)	1 each per RTB	T	SR 3.3.1.4	NA	NA

(continued)

(a) With the RTBs closed and the Rod Control System capable of rod withdrawal.
(d) Below the P-6 (Intermediate Range Neutron Flux) interlock.
(h) ≤ 38% RTP for full design power T_{avg} < 572°F or ≤ 53% RTP for full design power T_{avg} ≥ 572°F. For EOC coastdown, P-9 is not reset if T_{avg} decreases to < 572°F.
(i) 35% RTP for full design power T_{avg} < 572°F or 50% RTP for full design power T_{avg} ≥ 572°F. For EOC coastdown, P-9 is not reset if T_{avg} decreases to < 572°F.
(k) With 1 of 2 circulating water pump breakers closed and condenser vacuum ≥ 22 "Hg.