Virginia Electric and Power Company Surry Power Station 5570 Hog Island Road Surry, Virginia 23883

May 18, 2022

U. S. Nuclear Regulatory Commission	Serial No.: 22-160
Attention: Document Control Desk	SPS: SCN
Washington, D. C. 20555-0001	Docket No.: 50-280
-	50-281
	License No.: DPR-32
	DPR-37

Dear Sir or Madam:

Pursuant to 10CFR50.73, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to Surry Power Station Unit 1 and Unit 2.

Report No. 50-280 / 2022-001-00

This report has been reviewed by the Station Facility Safety Review Committee and will be forwarded to the Management Safety Review Committee.

Very truly yours,

Fred Mladen Site Vice President Surry Power Station

Enclosure

Commitment contained in the LER: None

cc: U.S. Nuclear Regulatory Commission, Region II Marquis One Tower, Suite 1200 245 Peachtree Center Ave., NE Atlanta, GA 30303-1257

> NRC Senior Resident Inspector Surry Power Station

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION						ISSION	APPROVED BY OMB: NO. 3150-0104 EXPIRES: 08/31/2023											
(08-2020) (See Page 3 for required number of digits/characters for each block) (See NUREG-1022, R.3 for instruction and guidance for completing this form http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/)							) his form <u>//r3/)</u>	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk all: <u>oira submission@omb.eop.gov</u> . The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.										
1. Facility Name									2. Docket	Number			3. Pa	ge				
Surry Power Station Unit 1									05	000	280			1	OF	4		
4. Title																		
Failure of Two Intake Canal Level Probes Due to Biofouling																		
5. Eve	ent Date			6. LER Number			7.	Repo	rt Date			8. Otł	ier Fa	acilities Invo	lved			
Month [	Day Y	′ear	Year	Sequential	Rev	ision o.	Month	Da	y Y	ear	Facility Name					Do	cket Number	
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9. Operating h				N					IU. FOWE	Lev		10	0					
			11. This I	Report is Subr	nitted	l Pur	suant to th	e Re	quireme	nts o	of 10 CFR §: (Ch	eck all th	at aj	oply)				
10 CFF	R Part 20	)	20	.2203(a)(2)(vi)			50.36(c)	(2)			50.73(a)(2)(iv	)(A)	50.73(a)(2)(x)					
20.220	1(b)		20	.2203(a)(3)(i)			50.46(a)	(3)(ii)			50.73(a)(2)(v)	(A)		10 CFR Part 73				
20.220	1(d)		20	.2203(a)(3)(ii)			50.69(g)				] 50.73(a)(2)(v)	(B)		73.71(a)(4)				
20.220	3(a)(1)		20	.2203(a)(4)			50.73(a)	(2)(i)(	A)		50.73(a)(2)(v)	(C)		73.71(a)(	5)			
20.220	3(a)(2)(i)		10	CFR Part 21		$\checkmark$	50.73(a)	(2)(i)(	B)		50.73(a)(2)(v)	a)(2)(v)(D) 73.77(a)(1)(i)						
20.2203(a)(2)(ii) 21.2(c) 50.73(a)(2)(i)(C)					C)	$\checkmark$	] 50.73(a)(2)(vi	a)(2)(vii) 73.77(a)(2)(i)										
20.220	3(a)(2)(iii)		10 (	CFR Part 50			50.73(a)	(2)(ii)	(A)		50.73(a)(2)(viii)(A) 73.77(a)(2)(ii)							
20.220	3(a)(2)(iv)		50	.36(c)(1)(i)(A)			50.73(a)	(2)(ii)	(B)		50.73(a)(2)(vi	ii)(B)						
20.220	3(a)(2)(v)		50	.36(c)(1)(ii)(A)			50.73(a)	( <b>2)(iii</b> )	)		50.73(a)(2)(ix)	)(A)						
	R (Specify I	here, ii	n abstrac	t, or NRC 366A)														
						12	. Licensee	Cont	act for t	his L	.ER							
Licensee Contact Fred Mladen, Surry Power Station Site Vice-President												Phone Num (75	ber (1 57) 3	nclude 865-20	area code) )01			
				13. Complete (	Dne l	ine f	for each Co	ompo	nent Fai	lure	Described in thi	s Report						
Cause	Syster	m	Compor	ent Manufact	urer	Repo	ortable to IRI	s	Cau	se	System	Compor	nent Manufacturer F		Report	able to IRIS		
С	JE		LE	F132	2		Y											
		14.	Suppleme	ental Report Expe	cted				<u>-</u>					Month		Day	Year	
No Yes (If yes, complete 15. Expected Submission Date)							e)	15.	Expected Submiss	sion Date								
16. Abstract (I	16. Abstract (Limit to 1560 spaces, i.e., approximately 15 single-spaced typewritten lines)																	

The non-essential Service Water (SW) automatic isolation function ensures adequate intake canal inventory can be maintained by the emergency service water pumps following a design basis accident. This is accomplished using intake canal level probes which will isolate the non-essential equipment once the canal level drops below a predetermined level. On 3/22/2022 during routine testing, 2 of the 4 probes failed their response time test. The probes were cleaned, retested satisfactory, and returned to service.

The cause of the test failure was determined to be biofouling. There were no safety consequences since an engineering analysis concluded that the intake canal level and inventory would have been sufficient to support all accident analysis assumptions. Planned corrective actions include increased testing to monitor the biofouling buildup and the continued testing of probe coatings to reduce the biofouling.

This event is being reported as a condition prohibited by the stations' Technical Specifications and a common cause failure.

NRC FORM 366A U.S. NUCLEAR REGULA	APPROVED BY OMB: NO. 3150-0104 EXPIRES: 08/31/2023												
(08-2020) LICENSEE EVENT REF CONTINUATION S (See NUREG-1022, R.3 for instruction and guidance fo	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U. S Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0104), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW Washington, DC 20503; e-mail: or a submission@omb.eep.gov. The NRC may not conduct or the NRC may no												
http://www.nrc.gov/reading-rm/doc-collections/nureg	js/staff/sr1022	<u>?/r3/)</u>	requesting or requiring the collection di	splays a currently v	valid OMB control number	niess the docum r.							
1. FACILITY NAME	t	2. DOCK			3. LER NUMBER								
Surry Power Station Unit 1	05000-		280	YEAR 2022	SEQUENTIAL NUMBER 001	REV NO. - 00							
NARRATIVE													
Plant Operating Conditions Prior to the Event	t												
Both units operating at 100%													
1.0 Description of Event													
The non-essential Service Water (SW) autom can be maintained by the emergency service (LOCA) with a coincident loss of offsite powe intake canal level probes [EIIS: JE, LE] sense feet. The probes are required to actuate in 5 intake canal level dropping to 23 feet under a accident analyses.	natic isolatio water pum r (LOOP). water levo 5 seconds all design b	on functi ıps [EIIS This fun el drop fi or less t asis sce	ion [EIIS: JE, KG] ensur S: BI, P] following a design totion is designed to actur rom the normal 28-foot I to ensure that the autom narios. The 23-foot leve	es adequat yn basis los uate when a level past a latic logic in el is the initi	te intake canal ss of coolant ad any three of the fixed level poinitiates prior to ial condition us	inventory ccident e four int of 23.5 the sed in the							
The four (4) intake canal level probes are rout typical period that the colonial hydroids grow range. Typically, the probes are found clean October cleanings. Although there is some v for the discovery and removal of colonial hyd	Itinely clear in the serv during the variance fro lroids.	ned and ice wate April and im year t	inspected from April to o r system, plus some but d May cleanings and fou to year, in general, this c	October. T ffer time on nd clean in cleaning pe	his is based or either side of the Septembe riod has worke	n the that time ∍r and ∍d best							
On 3/22/2022, the 18-month channel 1 and c scheduled. They were scheduled and perfor earlier (channel 1 on 7/10/2020 and channel found response time of 106 seconds. This w and cleaned per the calibration procedure. M actuation settings were checked per the calib The probe was retested in accordance with th seconds.	hannel 4 [E med seque 4 on 7/15/2 'as greater /linimal foul pration proc he calibrati	IIS: JE, ntially. I 2020). T than the ing was edure. I on proce	CHA] intake canal level Both probes had been ir The channel 1 probe was 55 second acceptance noted (recorded as 0%- No adjustments were ma edure and produced a sa	probe calil nstalled app tested firs criteria. Th 5% and hy ade, and no atisfactory r	brations were h proximately 18 at and produced ne probe was i droid-free). Th o problems we response time	both months d an as nspected ne re found. of 38							
Following this completed calibration, the char The probe was inspected and cleaned per the and hydroid-free). The actuation settings we no problems were found. The probe was rete of 53 seconds.	nnel 4 prob e calibration re checked ested in acc	e was te n procec per the cordance	ested, with an as-found r dure. Minimal fouling wa calibration procedure. I e with the procedure and	response tir as noted (al No adjustm d had a sati	me of 119 seco lso recorded as lents were mad isfactory respo	onds. s 0%-5% de, and onse time							
Since a satisfactory response time was obtain concluded that the slow time response on eac phenomenon that requires a period of a few of probes for a period exceeding 72 hours prior	ned on eac ch probe w days to bec to the test.	h probe as due t come est	with no actions perform to the common cause of tablished, it is likely that	ed, other th biofouling. the biofouli	nan the cleanin Since biofoul ing existed on	ig, it was ing is a both							

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1. FACILITY NAME		2. DOCK	(ET NUMBER		_	3. LER NUMBER	2	DEV				
Surry Power Station Unit 1	05000-		280	2022	-	= 001		NO. 00				
The probes were last cleaned and inspected satisfactory results (10-15 seconds) on 3/2/20 (17 seconds) on 4/19/2022.	on 10/5/20 022. The c	21. The hannel 2	e channel 3 probe had b 2 probe was time respor	een time nse teste	e re ed v	sponse testeo vith satisfacto	d wi iry i	ith results				
Reportability												
A review of each individual probe test (on 3/22/2022) showed that in both cases, the Technical Specification (TS) actions taken were appropriate based on a time of discovery failure determination. However, when the tests are taken in aggregate and assessed against the reporting requirements delineated in NUREG-1022, Rev. 3, "Event Report Guidelines 10 CFR 50.72 and 50.73," it was concluded that both probes were likely inoperable prior to their time of discovery and that the station had operated in a condition prohibited by the TS; Consequently, this report is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B). This report is also being reported pursuant to 10 CFR 50.73(a)(2)(vii)(D) as a common cause inoperability of independent trains or channels, i.e., a single cause resulted in two independent probe channels to become inoperable in a single system designed to mitigate the consequences of an accident.												
2.0 Significant Safety Consequences and Im	plications											
To determine the potential impact of the slow time response on the non-essential SW automatic isolation function, Engineering performed an evaluation to determine the intake canal minimum level that would have resulted from the slow time responses measured on 3/22/2022. This evaluation assessed the operating conditions present during the interval from the last cleaning (10/5/2021) to the time of the calibration tests (3/22/2022).												
The analysis concluded that the intake canal level and inventory would have been sufficient to support all accident analysis assumptions even if all four probes were conservatively assumed to actuate at the slowest of the four as found measured response times. In other words, the non-essential service water would have isolated prior to the intake canal level dropping below the assumed initial condition level of 23 feet.												
Additionally, intake canal level is also monitored by an independent system that provides level indication and high/low level alarms in the main control room. In the event of loss of intake canal level, station procedures direct corrective/ mitigative actions, including isolation of non-essential SW.												
In summary, this event resulted in no safety consequences or significant implications and the health and safety of the public was not affected at any time.												
3.0 Cause of the Event												
Since both intake canal level probes produced a satisfactory response time with no actions other than being cleaned, it has been concluded that the cause of the event was due to the material that was removed when cleaned. Observations indicate a muddy film that is suspected of being biofouling noted on both probes.												

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		2 DOCK			3. LER NUMBER					
Surry Power Station Unit 1	05000-		280	YEAR           2022	SEQUENTIAL NUMBER	REV NO. 00				
NARRATIVE										
The biofilm was easily removed with a clean found to be satisfactory. The channel 1 probe These response times are comparable to res	cotton cloth retested a ponse time	n. Once at 38 sec es measi	the fouling was remove conds and the channel 4 ured during bench testir	ed, the respo l probe rete ng (i.e., whe	onse time testi sted at 53 sec n clean).	ng was onds.				
4.0 Immediate Corrective Action										
The probes were cleaned, retested, and retur	rned to serv	vice.								
5.0 Additional Corrective Actions										
On 4/21/2022, all four intake canal probes were inspected and cleaned as part of the routine April to October cleaning cycle.										
6.0 Planned Corrective Actions										
<ol> <li>Several new coatings are being tested to r coatings will be monitored throughout the sur</li> </ol>	educe the b mmer and v	biofoulin winter se	ng material buildup. The easons for effectiveness	e performan s.	ce of these pro	bes and				
2. The cleaning frequency will be changed fro the cause of the biofouling is better understoo	om April to od.	October	r, which was based on c	alendar mo	nths, to year-re	ound until				
7.0 Similar Events										
1. LER S1 1997-009-01, "Intake Canal Level 2. LER S1 1998-010-00, "Intake Canal Level	Probes Inc Probes Inc	operable operable	Due to Marine Growth. Due to Marine Growth.	35 33						
8.0 Additional Information										
Units 1 and 2 continued to operate at 100% p	oower.									
Failed Component - Liquid Level Switch, Model 8-66, Manufactured by Fluid Components International (FCI).										

L