



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 11, 2022

Mr. Joel P. Gebbie
Senior Vice President and Chief
Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT NO. 1 - CORRECTION TO PAGES ISSUED FOR AMENDMENT NO. 359 REGARDING ADOPTION OF TSTF-577, "REVISED FREQUENCIES FOR STEAM GENERATOR TUBE INSPECTIONS" (EPID L-2021-LLA-0205)

Dear Mr. Gebbie:

By letter dated May 2, 2022, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 359 to Renewed Facility Operating License No. DPR-58, for the Donald C. Cook Nuclear Plant (CNP), Unit No. 1. The amendment consists of changes to the license and technical specifications (TSs) in response to your application dated November 8, 2021, as supplemented by letter dated February 1, 2022. These changes revise the TSs to adopt Technical Specifications Task Force (TSTF) Traveler TSTF-577, Revision 1, "Revised Frequencies for Steam Generator Tube Inspections."

The NRC inadvertently issued amendment pages that had errors on pages 5.5-6 and 5.6-5 of the TSs for CNP Unit No. 1. Specifically, for:

- TS 5.5.7.d.3, on page 5.5-6, the phrase "(whichever results in more frequent inspections)" should have been removed.
- TS 5.6.7.d, on page 5.6-5, the phrase "relative to the application performance criteria" should have been "relative to the applicable performance criteria."

Both errors were introduced during the issuance of License Amendment No. 359 for CNP, Unit No. 1.

The NRC staff has determined that these were inadvertent typographical errors and are entirely editorial in nature. Correcting these errors will align TS 5.5.7.d.3 and TS 5.6.7.d with TSTF-577 and the marked-up pages provided in the licensee's application. The corrections do not change any of the conclusions in the safety evaluation associated with Amendment No. 359 for CNP, Unit No. 1, and do not affect the associated notice to the public.

The corrected pages are enclosed and should be replaced as follows:

<u>Remove</u>	<u>Insert</u>
Unit 1 amendment page 5.5-6	Unit 1 amendment page 5.5-6
Unit 1 amendment page 5.6-5	Unit 1 amendment page 5.6-5

If you have any questions concerning this action, please contact me at (301) 415-2855 or by e-mail to Scott.Wall@nrc.gov.

Sincerely,

Scott P. Wall, Senior Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-315

Enclosure:
Corrected Pages to Amendment No. 359 to DPR-58

cc: Listserv

ENCLOSURE

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-315

CORRECTED PAGES TO LICENSE AMENDMENT NO. 359

TECHNICAL SPECIFICATIONS

PAGES 5.5-6 and 5.6-5

5.5 Programs and Manuals

5.5.7 Steam Generator (SG) Program (continued)

3. The operational LEAKAGE performance criterion is specified in LCO 3.4.13, "RCS Operational LEAKAGE."
- c. Provisions for SG tube plugging criteria. Tubes found by inservice inspection to contain flaws with a depth equal to or exceeding 40% of the nominal tube wall thickness shall be plugged.
- d. Provisions for SG tube inspections. Periodic SG tube inspections shall be performed. The number and portions of the tubes inspected and methods of inspection shall be performed with the objective of detecting flaws of any type (e.g., volumetric flaws, axial and circumferential cracks) that may be present along the length of the tube, from the tube-to-tubesheet weld at the tube inlet to the tube-to-tubesheet weld at the tube outlet, and that may satisfy the applicable tube plugging criteria. The tube-to-tubesheet weld is not part of the tube. In addition to meeting the requirements of d.1, d.2, and d.3 below, the inspection scope, inspection methods, and inspection intervals shall be such as to ensure that SG tube integrity is maintained until the next SG inspection. A degradation assessment shall be performed to determine the type and location of flaws to which the tubes may be susceptible and, based on this assessment, to determine which inspection methods need to be employed and at what locations.
 1. Inspect 100% of the tubes in each SG during the first refueling outage following SG installation.
 2. After the first refueling outage following SG installation, inspect 100% of the tubes in each SG at least every 96 effective full power months, which defines the inspection period.
 3. If crack indications are found in any SG tube, then the next inspection for each affected and potentially affected SG for the degradation mechanism that caused the crack indication shall be at the next refueling outage. If definitive information, such as from examination of a pulled tube, diagnostic non-destructive testing, or engineering evaluation indicates that a crack-like indication is not associated with a crack(s), then the indication need not be treated as a crack.
- e. Provisions for monitoring operational primary to secondary LEAKAGE.

5.6 Reporting Requirements

5.6.7 Steam Generator Tube Inspection Report (continued)

- d. An analysis summary of the tube integrity conditions predicted to exist at the next scheduled inspection (the forward-looking tube integrity assessment) relative to the applicable performance criteria, including the analysis methodology, inputs, and results;
 - e. The number and percentage of tubes plugged to date, and the effective plugging percentage in each SG; and
 - f. The results of any SG secondary side inspections.
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