



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

May 10, 2022

Mr. John J. Grabnar  
Site Vice President  
Energy Harbor Nuclear Corp.  
Beaver Valley Power Station  
Mail Stop P-BV-SSEB  
P.O. Box 4, Route 168  
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNITS 1 AND 2 – ISSUANCE OF  
AMENDMENT NOS. 316 AND 206 RE: REVISE TECHNICAL  
SPECIFICATION 5.6.3, “CORE OPERATING LIMITS REPORT (COLR)”  
(EPID L-2021-LLA-0075)

Dear Mr. Grabnar:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 316 and 206 to Renewed Facility Operating License Nos. DPR-66 and NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2, (Beaver Valley) respectively. These amendments consist of changes to the technical specifications (TS) in response to your application dated April 26, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML21117A076), as supplemented by letter dated October 27, 2021 (ADAMS Accession No. ML21300A359). The amendments revise TS 5.6.3 to allow the use of feedwater venturis that have been normalized to prior leading-edge flow meter measurements when calculating reactor thermal power.

A copy of the related safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's monthly *Federal Register* notice.

Sincerely,

***/RA/***

Brent Ballard, Project Manager  
Plant Licensing Branch I  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosures:

1. Amendment No. 316 to DPR-66
2. Amendment No. 206 to NPF-73
3. Safety Evaluation

cc: Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY HARBOR NUCLEAR CORP.  
ENERGY HARBOR NUCLEAR GENERATION LLC  
DOCKET NO. 50-334  
BEAVER VALLEY POWER STATION, UNIT NO. 1  
AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 316  
Renewed License No. DPR-66

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Energy Harbor Nuclear Corp., acting on its own behalf and as agent for Energy Harbor Nuclear Generation LLC\* (the licensees), dated April 26, 2021, as supplemented by letter dated October 27, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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\* Energy Harbor Nuclear Corp. is authorized to act as agent for Energy Harbor Nuclear Generation LLC and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-66 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 316, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

James G. Danna, Chief  
Plant Licensing Branch I  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Renewed Facility  
Operating License and Technical  
Specifications

Date of Issuance: May 10, 2022



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY HARBOR NUCLEAR CORP.  
ENERGY HARBOR NUCLEAR GENERATION LLC  
DOCKET NO. 50-412  
BEAVER VALLEY POWER STATION, UNIT 2  
AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 206  
Renewed License No. NPF-73

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Energy Harbor Nuclear Corp., acting on its own behalf and as agent for Energy Harbor Nuclear Generation LLC\* (the licensee), dated April 26, 2021, as supplemented by letter dated October 27, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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\* Energy Harbor Nuclear Corp. is authorized to act as agent for Energy Harbor Nuclear Generation LLC and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-73 is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 206, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto and hereby incorporated in the license. Energy Harbor Nuclear Corp. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

James G. Danna, Chief  
Plant Licensing Branch I  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Renewed Facility  
Operating License and Technical  
Specifications

Date of Issuance: May 10, 2022

ATTACHMENT TO LICENSE AMENDMENT NOS. 316 AND 206

BEAVER VALLEY POWER STATION, UNITS 1 AND 2

RENEWED FACILITY OPERATING LICENSE NOS. DPR-66 AND NPF-73

DOCKET NO. 50-334 and 50-412

Replace the following pages of the Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Renewed Facility Operating License No. DPR-66

Remove  
Page 3

Insert  
Page 3

Renewed Facility Operating License No. NPF-73

Remove  
Page 4

Insert  
Page 4

Replace the following page of the Appendix A, Technical Specifications, with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the areas of change.

Appendix A, Technical Specifications

Remove  
5.6 - 3

Insert  
5.6 - 3

- (3) Energy Harbor Nuclear Corp., pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - (4) Energy Harbor Nuclear Corp., pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
  - (5) Energy Harbor Nuclear Corp., pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter 1: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level  
Energy Harbor Nuclear Corp. is authorized to operate the facility at a steady state reactor core power level of 2900 megawatts thermal.
  - (2) Technical Specifications  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 316, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.
  - (3) Auxiliary River Water System  
(Deleted by Amendment No. 8)



C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations set forth in 10 CFR Chapter 1 and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Energy Harbor Nuclear Corp. is authorized to operate the facility at a steady state reactor core power level of 2900 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 206, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. Energy Harbor Nuclear Corp. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

## 5.6 Reporting Requirements

5.6.3 CORE OPERATING LIMITS REPORT (COLR) (continued)

WCAP-16045-P-A, "Qualification of the Two-Dimensional Transport Code PARAGON,"

WCAP-16045-P-A, Addendum 1-A, "Qualification of the NEXUS Nuclear Data Methodology,"

WCAP-12610-P-A & CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLO™,"

WCAP-17661-P-A, "Improved RAOC and CAOC F<sub>Q</sub> Surveillance Technical Specifications."

As described in reference documents listed above, when an initial assumed power level of 102% of RATED THERMAL POWER is specified in a previously approved method, 100.6% of RATED THERMAL POWER may be used when input for reactor thermal power measurement of feedwater flow is by the leading edge flow meter (LEFM) or feedwater venturi normalized to a prior LEFM flow measurement.

Caldon, Inc. Engineering Report-80P, "Improving Thermal Power Accuracy and Plant Safety While Increasing Operating Power Level Using the LEFM  $\sqrt{\text{TM}}$  System"

Caldon, Inc. Engineering Report-160P, "Supplement to Topical Report ER-80P: Basis for a Power Uprate with the LEFM  $\sqrt{\text{TM}}$  System"

- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6.4 Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

- a. RCS pressure and temperature limits for heat up, cooldown, low temperature operation, criticality, and hydrostatic testing, Overpressure Protection System (OPPS) enable temperature, and PORV lift settings as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:  
 LCO 3.4.3, "RCS Pressure and Temperature (P/T) Limits," and  
 LCO 3.4.12, "Overpressure Protection System (OPPS)"
- b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in WCAP-14040-A, Revision 4, "Methodology Used to Develop Cold Overpressure Mitigating System Setpoints and RCS Heatup and Cooldown Limit Curves," May 2004.



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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 316 AND 206 TO RENEWED

FACILITY OPERATING LICENSES NOS. DPR-66 AND NPF-73

ENERGY HARBOR NUCLEAR CORP.

ENERGY HARBOR NUCLEAR GENERATION LLC

BEAVER VALLEY POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By application dated April 26, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21117A076), as supplemented by letter dated October 27, 2021 (ADAMS Accession No. ML21300A359), Energy Harbor Nuclear Corp. (the licensee, Energy Harbor), requested changes to the technical specifications (TS) for Beaver Valley Power Station, Units 1 and 2 (Beaver Valley).

The proposed changes would revise TS 5.6.3, "Core Operating Limits Report (COLR)", to allow the use of feedwater venturis that have been normalized to prior leading edge flow meter (LEFM) measurements when calculating reactor thermal power. Specifically, the requirements of TS 5.6.3.b would be modified such that when an initial assumed power level of 102% of rated thermal power is specified in a previously approved method, as described in reference documents listed in TS 5.6.3.b, 100.6% of rated thermal power may be used when input for reactor thermal power measurement of feedwater flow is by the LEFM or feedwater venturi normalized to a prior LEFM measurement. Currently, TS 5.6.3.b does not permit use of feedwater venturi normalized to a prior LEFM measurement. If approved, section 3.3.8, Leading Edge Flow Meter, of the Beaver Valley Licensing Requirements Manual would be modified to allow the units to remain at rated thermal power for up to 72 hours after the last calorimetric heat balance measurement was performed using the LEFMs only if certain conditions are met.

The supplemental letter dated October 27, 2021, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on June 15, 2021 (86 FR 31742).

## 2.0 REGULATORY EVALUATION

The core thermal power must be accurately measured in order to ensure it does not exceed the licensed power level and invalidate the initial assumptions used in the design basis transient and accident analyses. There are two regulations and one generic communication which are most applicable to the proposed changes.

### 10 CFR 50.36(c)

Section 50.36 of Title 10 of the *Code of Federal Regulations* (10 CFR) requires that TS be included by applicants for a license authorizing operation of a production or utilization facility. Paragraph 10 CFR 50.36(c) requires that TS include (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. The proposed change to TS 5.6.3 is within the administrative controls category. The regulations in 50.36(c)(5), "Administrative controls," state that "[a]dministrative controls are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner."

### 10 CFR Part 50, Appendix K

Nuclear power plants are licensed to operate up to a specified reactor core thermal power limit. Appendix K to 10 CFR Part 50 requires loss of coolant accident and emergency core cooling system analyses to assume "that the reactor has been operating continuously at a power level at least 1.02 times the licensed power level" to allow for instrumentation error, with the maximum peaking factor allowed by the technical specifications. Alternately, Appendix K allows an assumption of lower than the specified 102 percent, but not less than the licensed thermal power level, "provided the proposed alternative value has been demonstrated to account for uncertainties due to power level instrumentation error."

### RIS 2002-03 Guidance on the Content of Measurement Uncertainty Recapture Power Uprate Applications

The NRC issued Regulatory Issue Summary (RIS) 2002-03, "Guidance on the Content of Measurement Uncertainty Recapture Power Uprate [MUR-PU] Applications," dated January 31, 2002 (ADAMS Accession No. ML013530183), to provide guidance to licensees seeking an MUR-PU on the basis of improved FW flow measurement. RIS 2002-03 provides guidance to licensees on the scope and detail of the information that should be provided to the NRC staff for MUR-PU license amendment requests (LARs). While RIS 2002-03 does not constitute an NRC requirement, its use aids licensees in the preparation of their MUR-PU LAR, while also providing guidance to the NRC staff to conduct its review. The licensee stated in its LAR that its submittal followed the guidance of RIS 2002-03, as it provides the technical basis for allowed outage time of an instrument.

## 3.0 TECHNICAL EVALUATION

The NRC staff has reviewed the licensee's regulatory and technical analyses in support of the proposed changes to the Beaver Valley TS 5.6.3, as described in the application dated April 26, 2021, and supplemented by letter dated October 27, 2021. The NRC staff's technical evaluation is detailed below.

### 3.1 Introduction

In its LAR, the licensee stated that the core thermal power for Beaver Valley is calculated with a calorimetric energy balance around the plant nuclear steam supply system. The accuracy of this calculation primarily depends on the accuracy of steam generator feedwater flow and feedwater net enthalpy measurements. Both Beaver Valley units previously used venturi meters for feedwater flow measurement and resistance temperature detectors (RTDs) for temperature measurement. However, Beaver Valley license amendment Nos. 243 and 122 for Unit 1 and Unit 2, respectively, allowed the use of more accurate ultrasonic flowmeters to provide these measurements.

At Beaver Valley, the LEFM system consists of an electronic cabinet in the process control area of the plant and a measurement section (spool piece) that is permanently installed in the 26-inch main feedwater header. The LEFM system is a single digital system controlled by software using the ultrasonic transit time method to measure four-line integral velocities at precise locations with respect to the pipe center line. The system numerically integrates the four measured velocities to determine the mass flow rate and fluid temperature. These measurements are used by the plant computer to determine the reactor thermal output.

The Caldon, Inc. Leading Edge Flowmeter<sup>√™</sup> (LEFM <sup>√™</sup>) system uses eight transducers in a configuration of two on each of the four acoustic measurement paths in a single plane of the spool piece. The LEFM CheckPlus<sup>™</sup> system uses 16 transducers in a similar configuration in 2 orthogonal planes of the spool piece. As such, the LEFM CheckPlus<sup>™</sup> system is a combination of two LEFM<sup>√™</sup> systems taking the average of two numerical integrations of four measurements each in two orthogonal planes. This measurement is inherently more accurate than the integration of four measurements in a single plane by the LEFM <sup>√™</sup> system and, therefore, should provide a better measurement accuracy. Also, due to twice as many measurements by the LEFM CheckPlus<sup>™</sup> system, the error in the statistical treatment of the instrumentation uncertainty is reduced. However, the licensee did not take credit for the improved accuracy and stated only that the LEFM CheckPlus<sup>™</sup> system provides feedwater flow measurement that is at least as accurate as that provided by the NRC-approved LEFM<sup>√™</sup> system.

In its LAR, the licensee stated that the use of the LEFM <sup>√™</sup> at Beaver Valley, Unit 1 and the LEFM CheckPlus<sup>™</sup> at Beaver Valley Unit 2 (hereafter both are referred to as LEFM) in place of the venturi meters and RTDs to measure feedwater flow allowed a reduction in the thermal power measurement uncertainty from 2 percent to 0.6 percent, and therefore, a power uprate of 1.4 percent. Currently, if the LEFM systems are nonfunctional, Beaver Valley Licensing Requirements Manual (LRM) Section 3.3.8, Leading Edge Flow Meter, in part, requires the steady-state thermal power to be reduced to less than or equal to 98.6 percent of rated thermal power if the systems are not restored to functional status prior to the next daily calorimetric heat balance measurement. The mass flow rate and feedwater temperature are displayed on the local display panel and transmitted to the plant process computer for use in the calorimetric measurement. Alarms are provided in the control rooms to alert operators should the systems require maintenance.

### 3.2 Items G and H of Section I, Attachment 1 to RIS 2002-03

Items G and H request that licensees provide a proposed allowed outage time (AOT) for the instrument, along with the technical basis for the time selected, and to propose actions to reduce power level if the AOT is exceeded, including a discussion of the technical basis for the proposed reduced power level.

To address Items G and H, the licensee stated, in its LAR, in part, that:

Section 3.3.8, Leading Edge Flow Meter, of the [Beaver Valley Unit 1 and 2] LRMs<sup>1</sup> would be modified to allow the units to remain at rated thermal power for up to 72 hours after the last calorimetric heat balance measurement was performed using the LEFMs only if the following conditions are met:

- Thermal power is continuously maintained greater than or equal to 90 percent of rated thermal power since the last LEFM measurement, and
- Feedwater flow (used in the calorimetric energy balance calculation) is obtained from venturi-based feedwater flow measurements that have been previously normalized to the appropriate LEFM flow values used in the last calorimetric heat balance based on LEFM measurements.

If the conditions are not met, thermal power would be reduced within one hour to less than or equal to 98.6 percent of rated thermal power steady-state [2900 MWt] until the LEFM is restored to functional status and the calorimetric heat balance measurement has been performed using the LEFM .

In its email dated September 16, 2021 (ADAMS Accession No. ML21259A164), the NRC staff requested additional information on what the contingency plan will be in the event that the plant process computer fails. In its October 27, 2021, response to the NRC staff's request for additional information, Energy Harbor confirmed that its procedures for performing the daily heat balance does provide for alternative methods including manual calculations if the plant computer is not available.

To support the proposed 72-hour LEFM allowed outage time, Energy Harbor performed an analysis which compared the differences between the LEFM feedwater flow measurement and the venturi-based feedwater flow measurement. The NRC staff reviewed the analysis which tracked the difference in those measurements through all seasonal time periods in the course of 1 year. It is important to note that for this analysis, the venturi-based feedwater flow measurement was not corrected or calibrated in any way to better match the LEFM during the one-year analysis window. The results of this analysis demonstrate that the difference in the feedwater flow measurement was minimal, even after 1 year, and that the difference in measurement did not change appreciably over any period of 72 hours. This analysis also demonstrates that any fouling or instrument drift would be minimal over the 72-hour time frame.

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<sup>1</sup> The LRMs are incorporated into the Beaver Valley Updated Final Safety Analysis Report (UFSAR). Per 10 CFR 50.71(e), the licensee is required to update its UFSAR's LRM chapters as described in its application.

The NRC staff finds that by normalizing the venturi-based feedwater flow measurements to the appropriate LEFM value used in the last calorimetric, any small bias in the difference between the LEFM feedwater flow measurement and the venturi-based feedwater flow measurement will be removed. Further, the NRC staff finds that the analysis provided demonstrates that there is reasonable assurance that the venturi-based feedwater flow measurement will result in a measurement of the feedwater flow rate that is within the measurement uncertainty of the LEFMs for at least 72 hours.

The NRC staff also finds the following conservatism in the proposed method:

- When initially normalized to the LEFM measurements, the venturi-based measurements would maintain a very similar degree of uncertainty in the heat balance calculations for a time period of more than 72 hours as would have been obtained with the venturis which are normalized during that same time period.
- The average difference between corresponding LEFM and RTD temperature measurements is similar to the margin already applied to LEFM temperature measurements used for calculating power. Therefore, the proposed amendment would maintain a consistent degree of uncertainty in heat balance calculations during the proposed 72-hour LEFM outage period, thereby assuring that the units would not be operated above their rated thermal power limits.

### 3.3 Summary

Based on a technical and regulatory review of the proposed changes to the Beaver Valley TS 5.6.3, the NRC staff finds that the proposed Beaver Valley amendment continues to meet the loss of coolant accident and emergency core cooling system analysis assumptions of 10 CFR Part 50, Appendix K and the information provided by the licensee is consistent with the guidance in RIS 2002-03. Additionally, since the TS requirements are derived from the analyses and evaluation included in the licensee's safety analysis report, and the staff's technical evaluation concluded that the modified TS change would continue to meet the licensee's safety analysis, the proposed amendment to TS 5.6.3.b will continue to ensure all applicable limits of the safety analysis are met and continues to meet the administrative controls requirements of 10 CFR 50.36(c)(5) to assure operation of the facility in a safe manner.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments on April 8, 2022. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (86 FR 31742; June 15, 2021).

Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J.S. Kaizer  
C. Scott

Date: May 10, 2022



SUBJECT: BEAVER VALLEY POWER STATION, UNITS 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 316 AND 206 RE: REVISE TECHNICAL SPECIFICATION 5.6.3, “CORE OPERATING LIMITS REPORT (COLR)” (EPID L-2021-LLA-0075) DATED MAY 10, 2022

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**ADAMS Accession No.: ML22095A235**

OFFICE	NRR/DORL/LPL1/PM	NRR/DORL/LPL1/LA	NRR/DSS/SFNB/BC
NAME	BBallard	KZeletznock	RLukes
DATE	04/05/2022	04/11/2022	03/17/2022
OFFICE	NRR/DSS/STSB/BC	NRR/DSS/SNSB/BC	NRR/DEX/EICB/BC(A)
NAME	VCusumano	SKrepel	RStattel
DATE	04/07/2022	04/01/2022	04/11/2022
OFFICE	OGC – NLO*	NRR/DORL/LPL1/BC	NRR/DORL/LPL1/PM
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DATE	04/28/2022	05/05/2022	05/10/2022

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