UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

February 10, 2022

Mr. Michael Strope
Site Vice President
NextEra Energy Point Beach, LLC
6610 Nuclear Road
Two Rivers, WI 54241-9516

SUBJECT: POINT BEACH NUCLEAR PLANT, UNIT 1—NOTIFICATION OF NRC BASELINE INSPECTION AND REQUEST FOR INFORMATION; INSPECTION REPORT 05000266/2022002

Dear Mr. Strope:

The U.S. Nuclear Regulatory Commission (NRC) will soon begin the Baseline Inservice Inspection Procedure 71111.08. This onsite inspection is scheduled to be performed March 28, 2022, through April 8, 2022.

Experience has shown that this inspection is resource intensive both for the NRC inspector and your staff. In order to minimize the impact to your onsite resources, and to ensure a productive inspection for both sides, we have enclosed a request for documents needed for this inspection. These documents have been divided into two groups.

The first group identifies information necessary for inspection preparation. The second group identifies information the inspector will need upon arrival at the site. It is important that all of these documents are up-to-date and complete in order to minimize the number of additional documents requested during the preparation and/or the onsite portions of the inspection.

We have discussed the schedule for inspection activities with your staff and understand that our regulatory contact for this inspection will be Mr. T. Schneider of your organization. If there are any questions about this inspection or the material requested, please contact the lead inspector Mr. J. Park at 630-829-9715, or via e-mail at joon.park@nrc.gov.

This letter does not contain new or amended information collection requirements subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.). Existing information collection requirements were approved by the Office of Management and Budget, Control Number 3150-0011. The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget Control Number.

This letter and its enclosure will be made available for public inspection and copying at http://www.nrc.gov/reading-rm/adams.html and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations*, Part 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Joon S. Park Signed by Park, Joon on 02/10/22

Joon S. Park, Reactor Inspector
Engineering Branch 1
Division of Reactor Safety

Docket No. 50–266
License No. DPR–24

Enclosure:
Document Request for Inservice Inspection

cc: Distribution via LISTSERV®

Letter to Michael Strope from Joon S. Park dated February 10, 2022.

SUBJECT: POINT BEACH NUCLEAR PLANT, UNIT 1—NOTIFICATION OF NRC BASELINE INSPECTION AND REQUEST FOR INFORMATION; INSPECTION REPORT 05000266/2022002

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DOCUMENT REQUEST FOR INSERVICE INSPECTION

Inspection Report: 05000266/2022002

Inspection Dates: March 28, 2022 – April 8, 2022

Inspection Procedures: Inspection Procedure 71111-08, Inservice Inspection

Lead Inspector: Joon S. Park, DRS
630-829-9715
joon.park@nrc.gov

A. Information for the In-Office Preparation Week

The following information is requested by March 14, 2022, to facilitate the selection of specific items that will be reviewed during the onsite inspection week. The inspector will select specific items from the information requested below and request a list of additional documents needed onsite to your staff. We request that the specific items selected from the lists be available and ready for review on the first day of inspection. The following information is applicable to the outage Unit 1 unless otherwise indicated. If you have any questions regarding this information, please contact the inspector.

1. For the upcoming outage, a detailed schedule and description of:

   a. Non-Destructive Examinations (NDEs) planned for Class 1 and 2 Systems and containment performed as part of your Inservice Inspection (ISI) program (please indicate applicable code edition and addenda), and NDEs planned for other systems including risk-informed, augmented, license renewal, and industry initiative inspection programs;

   b. Reactor vessel upper head examinations required by Title 10 of the *Code of Federal Regulations* (CFR), Part 50.55a(g)(6)(ii)(D) and Code Case N-729;

   c. Steam generator (SG) tube inspection and repair activities for the upcoming outage or SG secondary side examinations[1]; and

   d. Welding on Code Class 1, 2, or 3 components. For each weld examination, include the weld identification number, description of weld (component name), category, class, type of exam and procedure number, and date of examination.

[1] *Note 1- If no SG examinations are planned for this outage, please confirm when the next SG examination will occur.*

2. A copy of the NDE procedures, SG eddy current examination (ET) acquisition and analysis procedures, and welding procedures used to perform the activities identified in Item A.1. For NDE procedures, please include calibration and flaw characterization/sizing procedures, and, for welding procedures, please include applicable qualification records with the weld procedure specification. For ultrasonic examination procedures qualified in accordance with Appendix VIII of Section XI of the American Society of Mechanical Engineers (ASME) Code, provide documentation supporting the procedure qualification (e.g., the Electric Power Research Institute performance demonstration qualification summary sheets).

3. A copy of ASME Section XI, Code Relief Requests applicable to the examinations identified in Item A.1.

4. A copy of the 10-year ISI Program showing those required exams scheduled to be performed this outage, and those which have been completed.

5. A list identifying NDE reports (ultrasonic, radiography, magnetic particle, or dye penetrant), which have identified relevant indications on Code Class 1 and 2 Systems since the beginning of the last refueling outage.

6. A list with short description of the welds in Code Class 1, 2 and 3 Systems, which have been fabricated due to component repair/replacement activities since the beginning of the last refueling outage. Please identify the system, weld number, and reference applicable documentation for each (e.g., NIS-2 forms with definitions of system and component acronyms).

7. If reactor vessel weld examinations required by the ASME Code are scheduled to occur during the inspection period, provide a detailed description of the welds to be examined, and the extent of the planned examination.

8. A list of corrective actions initiated since the beginning of the last refueling outage with description of ISI and SG-related issues such as piping or SG tube degradation or damage (e.g., cracks, wall thinning, wear, microbiologically induced corrosion) or errors identified during piping/SG tube examinations. Also, please include a list of corrective action records associated with foreign material introduced/identified in the reactor vessel, primary coolant system, SG or feed systems for the same timeframe.

9. A copy of any 10 CFR Part 21 reports applicable to your structures, systems, or components within the scope of Section XI of the ASME Code that have been identified since the beginning of the last refueling outage.

10. A copy of SG history documentation given to vendors performing ET of the SGs during the upcoming outage[1].

11. A copy of procedure containing screening criteria used for selecting tubes for in-situ pressure testing and the procedure to be used for in-situ pressure testing .

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[1] *Note 1- If no SG examinations are planned for this outage, please confirm when the next SG examination will occur.*

12. A copy of previous outage SG tube operational assessment completed following ET of the SGs (provide this document even if no SG ET is planned for the upcoming outage).

13. A copy of most recent SG Degradation Assessment (provide this document even if no SG ET is planned for the upcoming outage).

14. A copy of most recent SG Condition Monitoring Assessments .

15. A copy of the document defining the planned SG ET scope (e.g., 100 percent of unrepaired tubes with bobbin probe and 20 percent sample of hot leg expansion transition regions with rotating probe) and identify the scope expansion criteria, which will be applied. Also identify and describe any deviations in this scope or expansion criteria from the Electric Power Research Institute Guidelines .

16. A copy of the document describing the ET acquisition equipment to be applied including ET probe types. Also identify the extent of planned tube examination coverage with each probe type (e.g., rotating probe - 0.080 inches, 0.115 inches pancake coils and mid-range + point coil applied at the top-of-tube-sheet plus 3 inches to minus 12 inches) .

17. Provide procedures with guidance/instructions for identifying (e.g., physically locating the tubes that require repair) and plugging SG tubes[1].

18. Identify and quantify any SG tube leakage experienced during the previous operating cycle. Also provide documentation identifying which SG was leaking and corrective actions completed or planned for this condition.

19. Point of contact information (name and site number) for the following activities:

    a. ISI—Site and vendor leads;

    b. Boric Acid Inspections and Evaluations;

    c. Reactor Vessel Head Inspection—Site and vendor leads;

    d. SG Inspection—Site and vendor leads;

    e. Aging Management Programs; and

    f. Site Welding Engineer.

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[1] *Note 1- If no SG examinations are planned for this outage, please confirm when the next SG examination will occur.*

B. Onsite Information to be Provided to the Inspector on the First Day of the Inspection (e.g., Following the Entrance Meeting). Please Provide Hard Copies (e.g., Paper Records) of the Following Documents.

1. For welds selected by the inspector from Item A.1.d and A.6 above, provide copies of the following documents:

   a. Document of the weld number and location (e.g., system, train, branch);

   b. Document with a detail of the weld construction (e.g., drawing);

   c. Applicable portions of the Design Specification and applicable Code of construction for the weldment (e.g., B31.1 or ASME Section III);

   d. Applicable Code Edition and Addenda for weld procedure qualification;

   e. Applicable weld procedure specifications and weld data sheets used to fabricate the welds;

   f. Copies of procedure qualification records supporting the weld procedure specifications;

   g. Copies of welders' performance qualification records;

   h. Copies of mechanical test reports identified in the weld procedure qualification records above;

   i. Copies of the nonconformance reports for the selected welds;

   j. Access to radiographs and equipment to view radiographs of the selected welds;

   k. ASME Code Section XI repair replacement plan and reconciliation for replacement components/materials;

   l. Certified Material Test Reports for replacement pressure boundary materials; and

   m. Copies of the NDE required by the construction Code and the pre-service examination records required by the ASME Code Section XI for the selected welds.

2. For the ISI-related corrective action issues selected by the inspector from Item A.8 above, provide a copy of the corrective actions and supporting documentation.

3. For the NDE reports with relevant indications on Code Class 1 and 2 Systems selected by the inspector from Item A.5 above, provide a copy of the examination records and associated corrective action documents.

4. Updated schedules for Item A.1 (including schedule showing contingency repair plans if available).

5. Fabrication Drawings (D size) of the reactor vessel welds (including vessel head J-groove welds) if any are to be examined during the outage. Also provide any drawings used by NDE vendors to locate these welds.

6. Copy of the procedures which govern the scope, equipment used, and implementation of the inspections required to identify boric acid leakage from systems and components above the vessel head.

7. Copy of:

   a. Engineering evaluations/assessments of boric acid related deposits and associated wastage or corrosion for safety-related components; and

   b. Corrective action records for coolant leakage including boric acid deposits on safety-related components identified since the beginning of the last refueling outage.

8. Copy of the plant procedures used to perform inspections to identify reactor coolant system leaks or boric acid deposits and the procedures for resolution of leaks or boric acid deposits.

9. Copy of the documents which demonstrate that the procedures to be used for volumetric examination of the reactor vessel head penetration J-groove welds were qualified by a blind demonstration test in accordance with 10 CFR 50.55a(g)(6)(ii)(D).

10. Copy of volumetric, surface and visual examination records for the prior inspection of the reactor vessel head and head penetration J-groove welds.

11. Provide a copy of the Electric Power Research Institute Examination Technique Specification Sheets and vendor related documents, which support qualification of the ET probes to be used during the upcoming SG tube inspections[1].

12. Provide a copy of the guidance to be followed if a loose part or foreign material is identified in the SGs .

13. Identify the types of SG tube repair processes which will be implemented for defective SG tubes (including any U.S. Nuclear Regulatory Commission (NRC) reviews/evaluation/approval of this repair process). Provide the flaw depth sizing criteria to be applied for ET indications identified in the SG tubes .

14. Copy of document describing actions to be taken if a new SG tube degradation mechanism is identified .

15. Provide document which defines the scope of SG secondary side examinations (if any are planned) and identify site specific operational history related to degradation of SG secondary side components (if any).

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[1] *Note 1- If no SG examinations are planned for this outage, please confirm when the next SG examination will occur.*

16. Provide procedures with guidance/instructions for identifying (e.g., physically locating the tubes that require repair) and plugging SG tubes[1].

17. Provide copies of the following standards at the onsite U.S. NRC inspection location for the duration of the inspection:

    a. Sections V, IX, and XI of the ASME Code with Editions applicable to the ISI Program and the Repair/Replacement Program;

    b. Copy of the performance demonstration initiative (PDI) generic procedures with the latest applicable revisions that support site qualified ultrasonic examination of piping welds and components (e.g., PDI-UT-1, PDI-UT-2, PDI-UT-3, PDI-UT-10 etc.);

    c. Electric Power Research Institute and industry standards referenced in the site procedures used to perform the SG tube ET, which includes Electric Power Research Institute documents: SG Examination Guidelines, SG Integrity Assessment Guidelines, SG In-Situ Pressure Test Guidelines[1]; and

    d. Boric Acid Corrosion Guidebook Revision 1—Electric Power Research Institute Technical Report 1000975.

18. Provide training (e.g., Scaffolding, Fall Protection, Foreign Material Exclusion) if required to access the NDEs selected by the inspector for observation.

If you have questions regarding the information requested, please contact the lead inspector.

[1] *Note 1- If no SG examinations are planned for this outage, please confirm when the next SG examination will occur.*