



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

January 20, 2022

Mr. David Rhoades
Senior VP, Exelon Generation Company, LLC
President and CNO, Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

**SUBJECT: LASALLE COUNTY STATION – BIENNIAL PROBLEM IDENTIFICATION AND
RESOLUTION INSPECTION REPORT 05000373/2021013 AND
05000374/2021013**

Dear Mr. Rhoades:

On December 20, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed a problem identification and resolution inspection at your LaSalle County Station and discussed the results of this inspection with Mr. Van Fleet, and other members of your staff. The results of this inspection are documented in the enclosed report.

The NRC inspection team reviewed the station's corrective action program and the station's implementation of the program to evaluate its effectiveness in identifying, prioritizing, evaluating, and correcting problems, and to confirm that the station was complying with NRC regulations and licensee standards for corrective action programs. Based on the samples reviewed, the team determined that your staff's performance in each of these areas adequately supported nuclear safety.

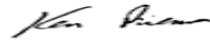
The team also evaluated the station's processes for use of industry and NRC operating experience information and the effectiveness of the station's audits and self-assessments. Based on the samples reviewed, the team determined that your staff's performance in each of these areas adequately supported nuclear safety.

Finally, the team reviewed the station's programs to establish and maintain a safety-conscious work environment, and interviewed station personnel to evaluate the effectiveness of these programs. Based on the team's observations and the results of these interviews the team found no evidence of challenges to your organization's safety-conscious work environment. Your employees appeared willing to raise nuclear safety concerns through at least one of the several means available.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,



Signed by Riemer, Kenneth
on 01/20/22

Kenneth R. Riemer, Chief
Branch 1
Division of Reactor Projects

Docket Nos. 05000373 and 05000374
License Nos. NPF-11 and NPF-18

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV®

Letter to David Rhoades from Kenneth R. Riemer dated January 20, 2022.

SUBJECT: LASALLE COUNTY STATION – BIENNIAL PROBLEM IDENTIFICATION AND RESOLUTION INSPECTION REPORT 05000373/2021013 AND 05000374/2021013

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Numbers: 05000373 and 05000374

License Numbers: NPF-11 and NPF-18

Report Numbers: 05000373/2021013 and 05000374/2021013

Enterprise Identifier: I-2021-013-0017

Licensee: Exelon Generation Company, LLC

Facility: LaSalle County Station

Location: Marseilles, IL

Inspection Dates: November 29, 2021 to December 17, 2021

Inspectors: J. Bozga, Senior Reactor Inspector
G. Hansen, Sr. Emergency Preparedness Inspector
G. Hausman, Senior Reactor Inspector
P. Smagacz, Resident Inspector

Approved By: Kenneth R. Riemer, Chief
Branch 1
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a biennial problem identification and resolution inspection at LaSalle County Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

OTHER ACTIVITIES – BASELINE

71152B - Problem Identification and Resolution

Biennial Team Inspection (IP Section 02.04) (1 Sample)

- (1) The inspectors performed a biennial assessment of the licensee's corrective action program, use of operating experience, self-assessments and audits, and safety conscious work environment.
 - **Corrective Action Program Effectiveness:** The inspectors assessed the corrective action program's effectiveness in identifying, prioritizing, evaluating, and correcting problems. The inspectors also conducted a five-year review of the 125V DC systems and Core Standby Cooling systems.
 - **Operating Experience, Self-Assessments and Audits:** The inspectors assessed the effectiveness of the station's processes for use of operating experience, audits and self-assessments.
 - **Safety Conscious Work Environment:** The inspectors assessed the effectiveness of the station's programs to establish and maintain a safety-conscious work environment.

INSPECTION RESULTS

| Assessment | 71152B |
|---|--------|
| Based on the samples reviewed, the team determined that the licensees' performance in each of these areas adequately supported nuclear safety. | |
| <u>Effectiveness of Problem Identification:</u> | |
| Overall, the licensee was effective at identifying issues at a low threshold and was properly entering them into the Corrective Action Program (CAP) as required by procedures. The inspectors determined that the station was generally effective at identifying negative trends that could potentially impact nuclear safety, at an appropriate threshold and at a healthy rate, team interviewed personnel, reviewed documents, walked down portions of the plant, and | |

assessed licensee problem identification. Staff were encouraged to identify issues and felt comfortable raising concerns with management. However, there were numerous descriptions of the workforce feeling that very low-level items never get fixed, even though numerous condition reports are entered year after year or every time certain surveillances are performed.

The inspectors reviewed root cause and apparent cause evaluations and determined those evaluations were adequate. The inspectors noted that adding more detail to the event timelines will aid in an independent reviewer understanding all known facts. For example, during the review of the root cause report for the refuel bridge mast foreign material issue (AR 4407109), a statement “troubleshooting was performed, and fuel moves recommenced” was made. This statement could have been expounded upon and detailed the troubleshooting efforts, what was or was not identified, and the criteria for recommencing with fuel moves. Also, detailed descriptions on the actual physical limitations of the refuel mast after each failure would have aided an independent reviewer.

Effectiveness of Prioritization and Evaluation of Issues:

In depth reviews of a risk-informed sampling of Action Requests (ARs), work orders (WOs), and cause evaluations were completed, including 5-year reviews of issues associated with the 125V DC Battery Systems and Core Standby Cooling Systems. The inspectors identified that the licensee generally, appropriately prioritized and evaluated issues for resolution. The inspectors also noted that while issue evaluations were generally sound, there were several examples where additional research was needed to understand the technical basis, primarily due to a lack of clear documentation. This theme had previously been identified in earlier licensee self-assessments and CAP audits and continued to be a concern.

Effectiveness of Corrective Actions:

The inspectors concluded that the licensee was generally effective in developing Corrective Actions that were appropriately focused to correct the identified problem and correct conditions adverse to quality. For issues considered significant conditions adverse to quality, the licensee took action to address the root and contributing causes and to preclude repetition. Corrective actions were generally completed in a timely manner and in accordance with procedural requirements commensurate with the safety significance of the issue.

5-year Reviews of the 125V DC Systems and Core Standby Cooling Systems:

The inspectors performed an expanded 5-year review of the Units 1 and 2 125 Vdc system; specifically, by performing system walk-downs and evaluating condition reports, engineering changes, and work orders. The inspectors also interviewed the system engineer responsible for this system. Overall, the inspectors determined that the licensee was effectively managing issues associated with this system.

As part of the review of the Core Standby Cooling System, the team identified a condition report (AR 4406388) which was associated with the use of operability limits (ASME Section III Appendix F) for determination of pressure boundary integrity/continued operation of FAC component 2RA023, a 4" tee in the reactor bottom head drain line 2RR28B-4. The inspection of this condition report will continue as a Selected Issue for Follow-Up (71152 inspection procedure) during NRC inspections in 2022.

Operating Experience, Self-Assessment and Audits:

The inspectors reviewed operating experience captured in the CAP and sampled operating experience from NRC, industry, vendors, and third-party groups. Overall, for the samples selected, the licensee was generally performing the appropriate assessments for station applicability. Through interviews and observations at selected station meetings, the inspectors also concluded that operating experience was being appropriately used in daily plant activities.

The inspectors determined that the licensee was adequately performing self-assessments and audits in accordance with licensee procedures and implementing corrective actions as needed. The inspectors did note not all self-assessment templates developed included sufficient guidance to ensure consistent evaluations. This was specifically identified in security self-assessments where one assessment included a discussion on the quantity of records reviewed, but other assessments contained blanket statements of passing.

Safety Conscious Work Environment:

As part of the NRC's safety conscious work environment evaluation during the PI&R biennial inspection, NRC inspectors completed a limited scope inspection sample of an assessment performed by the licensee that evaluated human performance behaviors exhibited during the recent refueling outage, L1R18, and the potential impact on safety conscious work environment at the site. The inspectors performed an independent review that evaluated the licensee's internal assessment of the site's SCWE program and the actions that were developed and implemented in response to the licensee's identified areas for improvement to promote a healthy organizational safety conscious work environment. The NRC inspectors reviewed licensee evaluation records, internal organizational assessment survey results, training plans, and conducted interviews with the Employee Concerns Program Manager, Organizational Effectiveness Manager, and management staff from various departments.

No violations or findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On December 20, 2021, the inspectors presented the biennial problem identification and resolution inspection results to Mr. Van Fleet, and other members of the licensee staff.

DOCUMENTS REVIEWED

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|---|-------------|---|------------------|
| 71152B | Calculations | L-003447 | LaSalle Units 1 and 2, 125Vdc System Analysis Dated June 11, 2020 | 002 |
| | Corrective Action Documents | 04062322 | Replace Battery 2DC18E Cell 44 | 10/12/2017 |
| | | 04114041 | Replace or Retest 1DC07E | 03/12/2018 |
| | | 04144731 | Generate Forced Outage Work Order for Battery Replacement | 06/06/2018 |
| | | 04203137 | U2 Division 1 Battery Replacement | 12/14/2018 |
| | | 04455972 | Request WR to Replace 2DC16E K306 Relay | 10/26/2021 |
| | | 04455973 | Request WR to Replace 2DC23E K306 Relay | 10/26/2021 |
| | | 1337255 | TRM Appendix G, Table 2 Enhancements Needed | 03/01/2012 |
| | | 141231 | VR Supply Duct Access Repair Patch Not Installed Per Design | 01/27/2003 |
| | | 1518242 | Ops Crew 6 4/17/13 LOOP Lessons Learned | 04/17/2013 |
| | | 2387941 | 2E22-F369A Relief Valve Failed As-Found Test | 09/29/2014 |
| | | 2530386 | FAC Program - OE Review of Davis Besse Steam Rupture | 05/19/2015 |
| | | 2618561 | Degraded Condition Observed on CSCS Piping Line 2HP55A-4 | 01/28/2016 |
| | | 3990967 | Degraded Condition Observed on CSCS Piping Line 2HP54A-4 | 03/29/2017 |
| | | 4048633 | RM - 1C71A-K10C Failed to De-Energize During LOS-RP-Q2 | 09/03/2017 |
| | | 4099862 | Limitation For 2HP55BB Per EC 403264 Flaw Evaluation | 02/01/2018 |
| | | 4172339 | Unit 2 LPCS Motor Cooler Inlet Pipe 2DG22A-1.5" is Degraded | 09/12/2018 |
| | | 4192778 | 2018 WANO AFI ER.2 | 11/06/2018 |
| | | 4221635 | VT-3 Exams Specified on Supports Exempted from Examination | 02/19/2019 |
| | | 4225076 | ACB 1414 Had a Lockout Trip While Performing LOS-AP-R2 | 03/01/2019 |
| 4228241 | Low Power TIP Set not Completed Due to TIPs OOS | 03/10/2019 | | |
| 4251021 | Review Code Requirements for Replacement VY Cooling Coils | 05/22/2019 | | |
| 4252624 | Code Non-Conformance Identified For VY Coolers 1(2)VY03A | 05/29/2019 | | |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|------|-------------|---|------------------|
| | | 4257366 | 4.0 Critique IR on Tendon Surveillances | 06/17/2019 |
| | | 4258604 | Late Surveillance (LIS-OG-408) | 06/21/2019 |
| | | 4261844 | Cardox Tank Compressor Tripped on Thermal Overloads | 07/04/2019 |
| | | 4262090 | NRC ID - Dry Pipe FP Lines not Sloped Per NFPA13 | 07/05/2019 |
| | | 4265814 | 1RE026 Closed Light Did Not Illuminate During LOS-PC-Q1 | 07/21/2019 |
| | | 4265960 | Large Steam Leak Found Downstream of 1G33-F040 | 07/22/2019 |
| | | 4266264 | Performance Level 3 OPEX Review of ICES 456532 for Applicability to LaSalle | 08/23/2019 |
| | | 4267062 | MSIV Pilot Solenoid Leads Crossed | 07/25/2019 |
| | | 4267121 | PCE Level 1 | 07/25/2019 |
| | | 4270252 | Historical 2C51-J004C App J Leakage Evaluations not Found | 08/07/2019 |
| | | 4271342 | Unexpected MCR Alarm: Unit 1 Hydrogen Panel Trouble | 08/12/2019 |
| | | 4273172 | Part 21 Flowserve Unqualified NDE Performance of Component | 08/20/2019 |
| | | 4273349 | NER NC-19-010-Y OE Review- Potential LaSalle Vulnerability | 08/20/2019 |
| | | 4276944 | Part 21 Contactors Failing to Release/Open When De-Energized | 09/05/2019 |
| | | 4277173 | NRC IN 2019-05: Over Pressurization of Alpha Sources | 09/06/2019 |
| | | 4280047 | AT 04257366-04 Needs to be Reevaluated | 09/17/2019 |
| | | 4281169 | LOS-SY-SR1 Revision Needed | 09/22/2019 |
| | | 4284443 | Through Wall Leak on ST Piping in U2 Div 1 CSCS Pump Room | 10/02/2019 |
| | | 4286161 | Unacceptable Trend Identified on MSIV Timing Per LOS-PC-Q2 | 10/08/2019 |
| | | 4288235 | NRC IN 2019-08 OPEX Regarding FAC Causing RX Scrams | 10/16/2019 |
| | | 4288344 | NRC ID: NRC Question on 1G33-F040 | 10/16/2019 |
| | | 4288357 | Replace Piping 1DG23B-6" | 10/16/2019 |
| | | 4294984 | NRC IN 19-09: Spent Fuel Cask Movement Issues | 10/30/2019 |
| | | 4295415 | Revise EC 363524 to Extend VY Fan SBM C\S Replacement PM's | 11/07/2019 |
| | | 4305470 | Biennial Safety Culture Self-Assessment | 02/04/2020 |
| | | 4306254 | Level 3 OPEX Review Requested #465166 Diablo Canyon | 12/27/2019 |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|------|-------------|--|------------------|
| | | | Unit 2 | |
| | | 4306904 | 1B RR Pump Lower Oil Reservoir Hi Level | 12/30/2019 |
| | | 4307124 | Level 3 OPEX Review Requested #464385 Salem Unit 2 | 12/31/2019 |
| | | 4307131 | Level 3 OPEX Review Requested #458702 Hatch Unit 2 | 12/31/2019 |
| | | 4318198 | L1R18 FAC; 1RH006 Low Wall Thickness | 02/15/2020 |
| | | 4319242 | L1R18 FAC; Component 1RH014 Low Wall Thickness | 02/19/2020 |
| | | 4322219 | 1ES010B Indicating Closed | 02/28/2020 |
| | | 4322221 | 1ES010C Indicating Closed | 02/28/2020 |
| | | 4328259 | P21- Potential Deviation of Pressure Transmitters | 03/20/2020 |
| | | 4330162 | Rosemount Transmitters Part 21 for Changes in Heat Rise | 03/27/2020 |
| | | 4343229 | Flex Generator Issue at Quad: LaSalle has Similar Generator | 05/14/2020 |
| | | 4357505 | U1 EHC Pressure Low in Band with New Discharge Filter | 07/18/2020 |
| | | 4362593 | NRC Question on 2CW03A Piping Support | 08/10/2020 |
| | | 4365890 | RM - Step Change Reduction in U2 Jet Pump 15 Flow | 08/27/2020 |
| | | 4366965 | Degraded Piping Support | 09/02/2020 |
| | | 4368035 | LR Inspection of 2LP13B-3" Piping | 09/08/2020 |
| | | 4368037 | LR Inspection of 2LP23AA-1/2" Piping | 09/08/2020 |
| | | 4368040 | LR Inspection for 2LP19A-3" Piping | 09/08/2020 |
| | | 4380329 | SR 3.4.3.1 Jet Pump# 15 Curve No Conservative Change | 10/28/2020 |
| | | 4382365 | B DFP High Discharge Pressure | 11/05/2020 |
| | | 4383488 | 2A RHR Seal Cooler Flow Low | 11/10/2020 |
| | | 4389292 | RM - JP 5/6 ACMP Flow Mismatch | 12/10/2020 |
| | | 4391800 | Loss of U2 Div 3 Water Leg Pump | 12/23/2020 |
| | | 4391803 | NRC IN 2020-04 - FP Mn Yard Buried Cast Iron Piping Failures | 12/17/2020 |
| | | 4404161 | L2R18 FAC Component 2HJ016 Low Wall Thickness | 02/23/2021 |
| | | 4404521 | L2R18 FAC; 2RH004, Low Wall Thickness | 02/24/2021 |
| | | 4404903 | 2DG01A Heat Exchanger Tubes Material Degradation | 02/24/2021 |
| | | 4404903 | 2DG01A Heat Exchanger Tubes Material Degradation | 02/26/2021 |
| | | 4405141 | Unanticipated Does Rate Alarm in High Radiation Area | 02/27/2021 |
| | | 4406388 | FAC Inspection on 2RA023 not Performed | 03/03/2021 |
| | | 4406519 | NRC ID Failure to Write IR in Heat Exchanger Program | 03/03/2021 |
| | | 4406519 | NRC ID Failure to Write IR in Heat Exchanger Program | 03/04/2021 |

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|----------------------|---------------------|---|---|--|------------|
| | | 4406544 | NRC ID LR Selective Leaching Program Perf. Deficiencies | 03/01/2021 | |
| | | 4406555 | L2R18 FAC; 2MS66B-RT-01, Low Wall Thickness | 03/04/2021 | |
| | | 4407109 | Unit 2 Refuel Bridge Mast Issue During Fuel Moves | 03/07/2021 | |
| | | 4407109 | Unit 2 Refuel Bridge Mast Issue During Fuel Moves | 03/07/2021 | |
| | | 4407388 | "L" SRV Drop Test Failure | 03/08/2021 | |
| | | 4407915 | "L" SRV Failed LOS-MS-R& Drop Test | 03/10/2021 | |
| | | 4407917 | "R" SRV Failed Regulated Header Drop Test | 03/10/2021 | |
| | | 4409413 | NRC Identified-NRC Questioned Weld Procedure Qualification | 03/15/2021 | |
| | | 4417755 | RM - Unit 2 Jet Pump #6 is not Within Jet Pump Curves | 04/20/2021 | |
| | | 4422004 | FAC RWCU Bottom Head Drain Lessons Learned | 05/06/2021 | |
| | | 4423071 | NRC IN 2021-01: Design Bases Lled Power Operated Valves | 05/11/2021 | |
| | | 4426705 | RM - 2A RPS M/G Set Output Breaker Tripped | 05/31/2021 | |
| | | 4427494 | 2A RPS MG Set Voltage Adjustment Pot Bad | 06/04/2021 | |
| | | 4428890 | L1R19 Class 1 Small Bore Pipe Socket Welds | 06/11/2021 | |
| | | 4433739 | Tendon Grease Cap Area Unit 2 Concrete Inspections | 07/07/2021 | |
| | | 4441741 | 2021 PI&R SA Gap: OBJ 3A | 08/20/2021 | |
| | | 44447160 | No Flow Thru U1 EHC Fullers Earth Filters Via Hydraulic Pump | 09/17/2021 | |
| | | 4448905 | 10CFR Part 21. Ametek Equalize Relay | 09/27/2021 | |
| | | Corrective Action Documents Resulting from Inspection | 4465190 | NRC ID- 2HP54A-4" UT Inspection Results Per WO-4621240 | 12/07/2021 |
| | | | 4466247 | NRC Identified Issue With L-003447 Rev. 002 | 12/13/2021 |
| | 4466816 | | NRC ID - 2DG22A-1.5" Min Wall Design Limit | 12/15/2021 | |
| | 4467017 | | NRC Observation - 2021 PI&R - CR Type AR Generation | 12/16/2021 | |
| | 4467039 | | NRC Observation - PI&R 2021 - Security Self-Assessments | 12/16/2021 | |
| | 4467111 | | NRC Observation - PI&R 2021 - Enhancement Opportunity for RCR | 12/16/2021 | |
| | Engineering Changes | 403264 | Minimum Wall Thickness for Unit 2 Division 3 VY Cooler Piping for Extent of Condition | 1 | |
| | | 403264 | Minimum Wall Thickness for Unit 2 Division 3 VY Cooler Piping for Extent of Condition | 2 | |
| | | 628380 | L2R18 FAC Min Wall Evaluation | 4 | |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|-------------------------|--------------------------------|--|------------------|
| | | 628380 | L2R18 FAC Min Wall Evaluation | 5 |
| | Engineering Evaluations | LAS-02121 | Failure Analysis of Limit Switch from LaSalle Station | 08/14/2018 |
| | | LAS-24193 | Failure Analysis of a Namco Limit Switch from LaSalle Station | 11/16/2018 |
| | | LAS-62503 | Failure Analysis of a Namco limit Switch from Lasalle Station | 05/03/2019 |
| | | LAS-93383 | Failure Analysis of a NAMCO Limit Switch Assembly | 06/13/2012 |
| | Miscellaneous | 4405141 | Rad Worker Behaviors Root Cause | 05/27/2021 |
| | | 4407109 | Root Cause Investigation Report for Unit 2 Refuel Bridge Mast Issue During Fuel Moves | 06/25/2021 |
| | | IEEE STD 485-1997 | IEEE Recommended Practice for Sizing Lead-Acid Batteries for Stationary Applications | 09/11/2003 |
| | | IEEE STD 946-1985 | IEEE Recommended Practice for the Design of Safety-Related DC Auxiliary Power Systems for Nuclear Power Generating Stations | 03/21/1985 |
| | | NOSA-LAS-19-05 | Engineering Design Control Audit Report | 07/10/2019 |
| | | NOSA-LAS-20-05 | Engineering Programs Audit Report | 07/15/2020 |
| | | NOSA-LAS-21-05 | LaSalle Corrective Action Program Audit Report | 09/01/2021 |
| | | OPEX Evaluation 2570259-02 | ICES 318466, Pin Hole Leak at Residual Heat Removal Service Water (RHRSW) Pipe Elbow | 11/25/2015 |
| | | OPEX Evaluation 4056244-02 | LER 2017-003, Division 2 Residual Heat Removal Service Water System (RHRSW) Inoperable Due to an Inoperable RHRSW Flow Control Valve (FCV) (Fermi 2) | 11/09/2017 |
| | | OPEX Evaluation 430760 | ESW Inop Due to Silt Buildup | 04/29/2020 |
| | NDE Reports | 20-141 | Ultrasonic Exam of 2DG22A-1 1/2 | 09/08/2020 |
| | Procedures | EI-AA-101 | Employee Concerns Program | 11 |
| | | ER-AA-320 | Maintenance Rule Implementation Per NEI 18-10 | 0 |
| | | NISP-PI-02 | Conduct of Self-Assessments and Benchmarks | 0 |
| | | NO-AA-10 | Quality Assurance Topical Report | 97 |
| | | PI-AA-101 | Conduct of Performance Improvement | 2 |
| | | PI-AA-1012 | Safety Culture Monitoring | 3 |
| | | PI-AA-115 | Operating Experience Program | 5 |
| | PI-AA-120 | Issue ID and Screening Process | 11 | |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|-------------------------------|---|-------------------------------|--|------------------|
| | | PI-AA-120 | Issue Identification and Screening Process | 11 |
| | | PI-AA-125-1001 | Root Cause Analysis Manual | 6 |
| | | PI-AA-125-1003 | Corrective Action Program Evaluation Manual | 6 |
| | | PI-AA-126-1001 | Self-Assessments | 5 |
| | Self-Assessments | 4091749 | RP Personnel Knowledge SA Report Assignment | 07/02/2019 |
| | | 4113395 | NRC SY Baseline Inspect-SA Report PASSPORT Assignment | 06/06/2019 |
| | | 4113396 | NRC EP Baseline Inspect-SA Report PASSPORT Assignment | 09/05/2019 |
| | | 4192133 | SA Report - M&TE Inventory Control, Access, & Handling | 10/08/2019 |
| | | 4192133 | Self-Assessment M&TE Inventory Control, Access, & Handling | 11/06/2018 |
| | | 4192145 | SA Report Assignment - M&T Effectiveness of ACAD 17-01 | 10/02/2019 |
| | | 4192182 | SA Report Assignment - NRC SY Baseline Inspection | 12/18/2019 |
| | | 4257452 | Self-Assessment of SA/BM NISP-PI-02 Implementation | 05/14/2020 |
| | | 4305470 | Biennial Safety Culture Self-Assessment | 02/04/2020 |
| | | 4329347 | LaSalle Pre-NRC Triennial Heat Sink | 02/24/2021 |
| | | 4329367 | SA Report - Security Baseline Inspection | 07/21/2021 |
| | | 4329672 | SA Report - RP Baseline Inspection | 07/16/2021 |
| | | 4329675 | SA Report - EP Baseline Inspection/Performance Indicator Verification | 10/01/2021 |
| | | 4343239 | Cyber Security Program Effectiveness Self-Assessment | 05/14/2020 |
| | | 4362946 | LaSalle M&T Comprehensive Self-Assessment Report | 06/30/2021 |
| | | 4378794 | 2020 INPO Performance Based Evaluation Results | 09/23/2021 |
| | | 4430137 | 2021 Preparation for NRC Problem Identification and Resolution (PI&R) Inspection | 08/23/2021 |
| | | LaSalle Station 01/27-31/2020 | Fleet Assessment Special Assessment Report | 02/24/2020 |
| | | LaSalle Station 03/23-27/2020 | Fleet Assessment T-6 Assessment Report (Maintenance) | 04/16/2020 |
| LaSalle Station 05/17-21/2021 | Fleet Assessment T-18 Assessment Report (Maintenance) | 06/14/2021 | | |
| LaSalle Station | Fleet Assessment Mid-Cycle Assessment Report | 09/18/2019 | | |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|-------------|-----------------|--|------------------|
| | | 08/16-28/2019 | (Maintenance) | |
| | | LaSalle Station | Fleet Assessment Midcycle Assessment Debrief | 11/15/2021 |
| | Work Orders | 4866827 | Unit 2 Refuel Bridge Contingency Repairs | 03/07/2021 |