



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 4, 2022

Mr. James Barstow
Vice President, Nuclear Regulatory
Affairs and Support Services
Tennessee Valley Authority
1101 Market Street, LP 4A-C
Chattanooga, TN 37402-2801

SUBJECT: WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2 – ISSUANCE OF
AMENDMENT NOS. 152 AND 61 REGARDING REVISION TO TECHNICAL
SPECIFICATIONS TO DELETE A REDUNDANT UNIT OF MEASURE FOR
CERTAIN RADIATION MONITORS (EPID L-2021-LLA-0100)

Dear Mr. Barstow:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 152 to Facility Operating License No. NPF-90 and Amendment No. 61 to Facility Operating License No. NPF-96 for the Watts Bar Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated June 1, 2021.

The amendments revise Technical Specification (TS) 3.3.6, "Containment Vent Isolation Instrumentation," and TS 3.3.7, "Control Room Emergency Ventilation System (CREVS) Actuation Instrumentation," to delete a redundant unit of measure associated with the trip setpoint for containment purge exhaust and control room air intake radiation monitors. The amendments also correct a typographical error in abbreviation of the associated unit of measure.

J. Barstow

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A copy of our related safety evaluation is also enclosed. Notice of issuance will be included in the Commission's monthly *Federal Register* notice.

Sincerely,

/RA/

Kimberly J. Green, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-390 and 50-391

Enclosures:

1. Amendment No. 152 to NPF-90
2. Amendment No. 61 to NPF-96
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-390

WATTS BAR NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 152
License No. NPF-90

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (TVA, the licensee) dated June 1, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-90 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 152 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

David J. Wrona, Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License
and Technical Specifications

Date of Issuance: May 4, 2022

ATTACHMENT TO AMENDMENT NO. 152

WATTS BAR NUCLEAR PLANT, UNIT 1

FACILITY OPERATING LICENSE NO. NPF-90

DOCKET NO. 50-390

Replace page 3 of Facility Operating License No. NPF-90 with the attached revised page 3. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove Pages

3.3-55
3.3-60

Insert Pages

3.3-55
3.3-60

- (4) TVA, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis, instrument calibration, or other activity associated with radioactive apparatus or components; and
- (5) TVA, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

TVA is authorized to operate the facility at reactor core power levels not in excess of 3459 megawatts thermal.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 152 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Safety Parameter Display System (SPDS) (Section 18.2 of SER Supplements 5 and 15)

Prior to startup following the first refueling outage, TVA shall accomplish the necessary activities, provide acceptable responses, and implement all proposed corrective actions related to having the Watts Bar Unit 1 SPDS operational.

(4) Vehicle Bomb Control Program (Section 13.6.9 of SSER 20)

During the period of the exemption granted in paragraph 2.D.(3) of this license, in implementing the power ascension phase of the approved initial test program, TVA shall not exceed 50% power until the requirements of 10 CFR 73.55(c)(7) and (8) are fully implemented. TVA shall submit a letter under oath or affirmation when the requirements of 73.55(c)(7) and (8) have been fully implemented.

Table 3.3.6-1 (page 1 of 1)
Containment Vent Isolation Instrumentation

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Manual Initiation	2	SR 3.3.6.6	NA
2. Automatic Actuation Logic and Actuation Relays	2 trains	SR 3.3.6.2 SR 3.3.6.3 SR 3.3.6.5	NA
3. Containment Purge Exhaust Radiation Monitors	2	SR 3.3.6.1 SR 3.3.6.4 SR 3.3.6.7	$\leq 2.8E-02 \mu\text{Ci/cc}$
4. Safety Injection	Refer to LCO 3.3.2, "ESFAS Instrumentation," Function 1, for all initiation functions and requirements.		

Table 3.3.7-1 (page 1 of 1)
CREVS Actuation Instrumentation

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Manual Initiation	2 trains	SR 3.3.7.3	NA
2. Control Room Radiation Control Room Air Intakes	2	SR 3.3.7.1 SR 3.3.7.2 SR 3.3.7.4	$\leq 1.647E-04 \mu\text{Ci/cc}$
3. Safety Injection	Refer to LCO 3.3.2, "ESFAS Instrumentation," Function 1, for all initiation functions and requirements.		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-391

WATTS BAR NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 61
License No. NPF-96

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (TVA, the licensee) dated June 1, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-96 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 61 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

David J. Wrona, Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License
and Technical Specifications

Date of Issuance: May 4, 2022

ATTACHMENT TO AMENDMENT NO. 61
WATTS BAR NUCLEAR PLANT, UNIT 2
FACILITY OPERATING LICENSE NO. NPF-96
DOCKET NO. 50-391

Replace page 3 of Facility Operating License No. NPF-96 with the attached revised page 3. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove Pages

3.3-58
3.3-62

Insert Pages

3.3-58
3.3-62

C. The license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act, and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

TVA is authorized to operate the facility at reactor core power levels not in excess of 3459 megawatts thermal.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 61 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) TVA shall implement permanent modifications to prevent overtopping of the embankments of the Fort Loudon Dam due to the Probable Maximum Flood by June 30, 2018.

(4) FULL SPECTRUM LOCA Methodology shall be implemented when the WBN Unit 2 steam generators are replaced with steam generators equivalent to the existing steam generators at WBN Unit 1.

(5) By December 31, 2019, the licensee shall report to the NRC that the actions to resolve the issues identified in Bulletin 2012-01, "Design Vulnerability in Electrical Power System," have been implemented.

(6) The licensee shall maintain in effect the provisions of the physical security plan, security personnel training and qualification plan, and safeguards contingency plan, and all amendments made pursuant to the authority of 10 CFR 50.90 and 50.54(p).

(7) TVA shall fully implement and maintain in effect all provisions of the Commission approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The TVA approved CSP was discussed in NUREG-0847, Supplement 28, as amended by changes approved in License Amendment No. 7.

(8) TVA shall implement and maintain in effect all provisions of the approved fire protection program as described in the Fire Protection Report for the facility, as described in NUREG-0847, Supplement 29, subject to the following provision:

Table 3.3.6-1 (page 1 of 1)
Containment Vent Isolation Instrumentation

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Manual Initiation	2	SR 3.3.6.6	NA
2. Automatic Actuation Logic and Actuation Relays	2 trains	SR 3.3.6.2 SR 3.3.6.3 SR 3.3.6.5	NA
3. Containment Purge Exhaust Radiation Monitors	2	SR 3.3.6.1 SR 3.3.6.4 SR 3.3.6.7	$\leq 2.8E-02 \mu\text{Ci/cc}$
4. Safety Injection	Refer to LCO 3.3.2, "ESFAS Instrumentation," Function 1, for all initiation functions and requirements.		

Table 3.3.7-1 (page 1 of 1)
CREVS Actuation Instrumentation

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Manual Initiation	2 trains	SR 3.3.7.3	NA
2. Control Room Radiation Control Room Air Intakes	2	SR 3.3.7.1 SR 3.3.7.2 SR 3.3.7.4	$\leq 1.647E-04 \mu\text{Ci/cc}$
3. Safety Injection	Refer to LCO 3.3.2, "ESFAS Instrumentation," Function 1, for all initiation functions and requirements.		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 152 AND 61

TO FACILITY OPERATING LICENSE NOS. NPF-90 AND NPF-96

TENNESSEE VALLEY AUTHORITY

WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-390 AND 50-391

1.0 INTRODUCTION

By letter dated June 1, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21153A071), the Tennessee Valley Authority (TVA, the licensee), submitted a license amendment request to the U.S. Nuclear Regulatory Commission (NRC, the Commission) for changes to the Watts Bar Nuclear Plant (Watts Bar), Units 1 and 2, Technical Specifications (TSs). The requested changes would revise Watts Bar, Units 1 and 2, TS 3.3.6, "Containment Vent Isolation Instrumentation," and TS 3.3.7, "Control Room Emergency Ventilation System (CREVS) Actuation Instrumentation," to delete a redundant unit of measure associated with the trip setpoint for TS Table 3.3.6-1, Function 3, "Containment Purge Exhaust Radiation Monitors," and TS Table 3.3.7-1, Function 2, "Control Room Radiation Control Room Air Intakes," respectively. The requested changes would also correct a typographical error in the unit of measure for the allowable value (AV) for TS Table 3.3.7-1, Function 2. The unit of measure, which is microcuries per cubic centimeter, is currently abbreviated as $\mu\text{C}/\text{cc}$. The correct abbreviation is $\mu\text{Ci}/\text{cc}$.

2.0 REGULATORY EVALUATION

2.1 System Description

Isolation of the containment occurs if the containment purge exhaust radiation monitor setpoint is exceeded to minimize the release of radioactivity in the event of an accident. The release of radioactivity in the event of an accident from the containment purge exhaust system has potential consequences to both members of the public and to plant operators in the control room.

In addition to the containment purge exhaust radiation monitors, a second set of radiation monitors, called the control room radiation control room air intake monitors, are used to isolate the plant control room air intake when the radioactivity concentrations in the air intake exceeds the AV setpoints. If the setpoint for the control room radiation control room air intakes monitor is exceeded the main control room habitability zone is isolated and the control room emergency

ventilation system is actuated. These actions provide an enclosed control room environment to allow the unit to be operated without excessive dose to the operators. The TS AVs for the radiation monitor setpoints ensure that both the offsite dose to members of the public and control room operators are appropriately limited. The AVs are set at values to ensure that the regulatory limits are not exceeded.

2.2 Requested Changes

The licensee proposed to delete the parenthetical cpm (counts per minute) unit of measure from the TS Table 3.3.6-1, Function 3 AV, and TS Table 3.3.7-1, Function 2 AV, as shown in redline/strikeout below.

Also, the licensee proposed to correct the abbreviation for the AV unit of measure in Table 3.3.7-1, Function 2, to $\mu\text{Ci}/\text{cc}$.

Watts Bar, Unit 1:

Table 3.3.6-1

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. Containment Purge Exhaust Radiation Monitors	2	SR 3.3.6.1 SR 3.3.6.4 SR 3.3.6.7	$\leq 2.8\text{E-}02 \mu\text{Ci}/\text{cc}$ ($2.8\text{E}+04 \text{cpm}$)

Table 3.3.7-1

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
2. Control Room Radiation Control Room Air Intakes	2	SR 3.3.7.1 SR 3.3.7.2 SR 3.3.7.4	$\leq 1.647\text{E-}04 \mu\text{Ci}/\text{cc}$ ($3,308 \text{cpm}$)

Watts Bar, Unit 2:

Table 3.3.6-1

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. Containment Purge Exhaust Radiation Monitors	2	SR 3.3.6.1 SR 3.3.6.4 SR 3.3.6.7	$\leq 2.8\text{E-}02 \mu\text{Ci}/\text{cc}$ ($1.14 \times 10^4 \text{cpm}$)

Table 3.3.7-1

FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
2. Control Room Radiation Control Room Air Intakes	2	SR 3.3.7.1 SR 3.3.7.2 SR 3.3.7.4	$\leq 1.647\text{E-}04 \mu\text{Ci}/\text{cc}$ ($3,308 \text{cpm}$)

2.3 Regulations and Guidance

The NRC staff's evaluation is based upon the following regulations, regulatory guides, and standards:

Under Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.90, "Application for amendment of license, construction permit, or early site permit," whenever a holder of a license wishes to amend the license, including TSs in the license, an application for amendment must be filed, fully describing the changes desired. Under 10 CFR 50.92(a), determinations on whether to grant an applied-for license amendment are to be guided by the considerations that govern the issuance of initial licenses to the extent applicable and appropriate. Both the common standards for licenses in 10 CFR 50.40(a), and those specifically for issuance of operating licenses in 10 CFR 50.57(a)(3), provide that there must be reasonable assurance that the activities at issue will not endanger the health and safety of the public, and that the applicant will comply with the NRC's regulations.

Pursuant to 10 CFR 50.36, "Technical specifications," TSs for operating reactors are required, in part, to include items in the following five specific categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls.

Section 50.36(a)(1) of 10 CFR requires each applicant for a license authorizing operation of a production or utilization facility to include a summary statement of the bases or reasons for proposed TSs, other than those covering administrative controls; however, the bases shall not become part of the TSs.

Section 50.36(b) of 10 CFR requires that each license authorizing reactor operation include TSs derived from the analyses and evaluation included in the safety analysis report and amendments thereto.

Section 50.36(c) of 10 CFR requires that TSs include certain items. Per 10 CFR 50.36(c)(2)(i), the TSs must include LCOs, which are the lowest functional capability or performance levels of equipment required for safe operation of the facility. That provision also requires that when an LCO of a nuclear reactor is not met, the licensee must shut down the reactor or follow any remedial action permitted by the TSs until the condition can be met.

The NRC staff's guidance for the review of TSs is in NUREG-0800, "Standard Review Plan [SRP] for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR [Light Water Reactor] Edition," Chapter 16.0, "Technical Specifications," Revision 3, dated March 2010 (ADAMS Accession No. ML100351425).

3.0 TECHNICAL EVALUATION

3.1 Background

The radiation monitor setpoints are determined by the limiting accidents to assure that their radiological consequences do not result in an unacceptable release of radioactivity or unacceptable doses to the control room operators. The containment purge exhaust isolation setpoint is based on the small break loss-of-coolant accident, which is the limiting accident for this function. The steam generator tube rupture accident is the limiting event for the control

room air intake isolation function. These setpoints are governed by TSs to assure that regulatory limits will be met. The proposed changes do not affect the associated dose calculations because the AVs remain the same. Only the indirectly measured AVs, which are dependent on detector efficiency, are proposed for elimination.

In addition, a correction to the abbreviation for microcuries per cubic centimeter is implemented by the change. This change is purely editorial and serves only to assure clarity in the unit of measure by changing it to an industry accepted abbreviation. There is no effect to the meaning of the TS from this change.

3.2 Staff Evaluation of Proposed TS Changes

3.2.1 Deletion of the Allowable Value in cpm (counts per minute) from Table 3.3.6-1, "Containment Vent Isolation Instrumentation," Function 3, "Containment Purge Exhaust Radiation Monitors"

The licensee proposed to delete the cpm value ($2.8E+04$ cpm, Unit 1 and 1.14×10^4 cpm, Unit 2) for the containment purge exhaust radiation monitors and retain the corresponding direct unit of measure in $\mu\text{Ci/cc}$. The cpm value is dependent on the efficiency of the radiation monitor and can change if the design of the monitor is modified or different monitors are installed. Since the direct unit of measure is not affected by the design of the radiation monitors and is consistent with the licensee's calculations for the dose limit, there is no effect on the current licensing basis with respect to design-basis accident analyses of interest with removal of the redundant unit of measurement (i.e., cpm). The radiation monitors will continue to perform their function as currently required by TS. Therefore, the proposed change is acceptable.

3.2.2 Deletion of the Allowable Value in cpm (counts per minute) from Table 3.3.7-1, "CREVS Actuation Instrumentation," Function 2, "Control Room Radiation Control Room Air Intakes"

The licensee proposed to delete the cpm value, 3,308 cpm, in each unit's TSs for the control room air intake radiation monitors and retain the corresponding direct unit of measure in $\mu\text{Ci/cc}$. The cpm value is dependent on the efficiency of the radiation monitor and can change if the design of the monitor is modified or different monitors are installed. Since the direct unit of measure is not affected by the design of the radiation monitors and is consistent with the licensee's calculations for the dose limit, there is no effect on the current licensing basis with respect to design-basis accident analysis with removal of the redundant unit of measurement (i.e., cpm). The radiation monitors will continue to perform their function as currently required by TS. Therefore, the proposed change is acceptable.

3.2.3 Correction of the Units for the Allowable Value in Table 3.3.7-1, "CREVS Actuation Instrumentation," Function 2, "Control Room Radiation Control Room Air Intakes"

The licensee proposed to change the unit of measure abbreviation for the control room radiation control room air intake radiation monitors from $\mu\text{C/cc}$ to $\mu\text{Ci/cc}$. This is an editorial change that corrects the unit of measure to an industry accepted format with no technical change to the TS requirements. Therefore, the correction is acceptable.

3.3 Summary and Conclusion

The NRC staff has reviewed the proposed changes to delete the AVs as stated in cpm and retain the equivalent AVs stated in $\mu\text{Ci/cc}$. In addition, the staff reviewed the editorial correction to the unit of measure abbreviation as discussed above.

There were no changes proposed for any SRs or the TS Bases. The NRC staff determined that changes were not required for the SRs or the Bases because they are not affected by the proposed change.

The NRC staff determined that the proposed changes will not impact the current licensing basis dose consequence analyses or radiation releases associated with the setpoints because there are no changes to the AVs and the existing values remain the same. Therefore, the NRC staff finds that the proposed TS changes are acceptable from a radiological dose consequence analysis and release perspective and, that the TSs, as revised, will continue to meet the requirements of 10 CFR 50.36 and the licensing basis.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendment on October 1, 2021. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission previously issued a proposed finding that the amendment involves no significant hazards consideration published in the *Federal Register* on August 10, 2021 (86 FR 43691), and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. Stattel, NRR
S. Meighan, NRR
S. Smith, NRR

Date: May 4, 2022

SUBJECT: WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 152 AND 61 REGARDING REVISION TO TECHNICAL SPECIFICATIONS TO DELETE A REDUNDANT UNIT OF MEASURE FOR CERTAIN RADIATION MONITORS (EPID L-2021-LLA-0100) DATED MAY 4, 2022

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OFFICE	NRR/DEX/EICB/BC	NRR/DRA/ARCB/BC	OGC – NLO w/comments
NAME	MWaters	KHsueh	AGhosh
DATE	02/16/22	03/01/22	03/14/22
OFFICE	NRR/DORL/LPL2-2/BC	NRR/DORL/LPL2-2/PM	
NAME	DWrona	KGreen	
DATE	05/04/2022	05/04/2022	

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