



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 29, 2022

Mr. Oliver Potts, Director
Office of the Federal Register (F)
The National Archives and Records Administration
8601 Adelphi Road
College Park, MD 20740-6001

Dear Director Potts:

In accordance with 1 CFR part 51, we request that you approve the incorporation by reference of the material listed below into Title 10 of the *Code of Federal Regulations* (CFR). We have included the IBR material and draft final rule. We have uploaded copies of all IBR material. The IBR material will be incorporated into Sections II, III, VI, VIII, and X of appendix G of 10 CFR part 52 as follows.

NuScale Power, LLC: Thomas Bergman, Vice President, Regulatory Affairs, NuScale Power, LLC, 6650 SW Redwood Lane, Suite 210, Portland, Oregon, 97224, telephone: 1-971-371-1592, email: RegulatoryAffairs@nuscalepower.com.

1. NuScale Standard Plant Design Certification Application; Certified Design Descriptions and Inspections, Tests, Analyses, & Acceptance Criteria (ITAAC); Part 2 - Tier 1, Revision 5, July 2020.
2. NuScale Standard Plant Design Certification Application, Part 2 - Tier 2, Revision 5, July 2020:
 - a. Chapter One, Introduction and General Description of the Plant.
 - b. Chapter Two, Site Characteristics and Site Parameters.
 - c. Chapter Three, Design of Structures, Systems, Components and Equipment.
 - d. Chapter Four, Reactor.
 - e. Chapter Five, Reactor Coolant System and Connecting Systems.
 - f. Chapter Six, Engineered Safety Features.
 - g. Chapter Seven, Instrumentation and Controls.
 - h. Chapter Eight, Electric Power.
 - i. Chapter Nine, Auxiliary Systems.
 - j. Chapter Ten, Steam and Power Conversion System.
 - k. Chapter Eleven, Radioactive Waste Management.
 - l. Chapter Twelve, Radiation Protection.
 - m. Chapter Thirteen, Conduct of Operations.
 - n. Chapter Fourteen, Initial Test Program and Inspections, Tests, Analyses, and Acceptance Criteria.
 - o. Chapter Fifteen, Transient and Accident Analyses.
 - p. Chapter Sixteen, Technical Specifications.

[ML21326A069]

- q. Chapter Seventeen, Quality Assurance and Reliability Assurance.
 - r. Chapter Eighteen, Human Factors Engineering.
 - s. Chapter Nineteen, Probabilistic Risk Assessment and Severe Accident Evaluation.
 - t. Chapter Twenty, Mitigation of Beyond-Design-Basis Events.
 - u. Chapter Twenty-One, Multi-Module Design Considerations.
3. DCA Part 4, Volume 1, Revision 5.0, Generic Technical Specifications, NuScale Nuclear Power Plants, Volume 1: Specifications.
 4. DCA Part 4, Volume 2, Revision 5.0, Generic Technical Specifications, NuScale Nuclear Power Plants, Volume 2: Bases.
 5. ES-0304-1381-NP, Human-System Interface Style Guide, December 2019, Revision 4.
 6. RP-0215-10815-NP, Concept of Operations, May 2019, Revision 3.
 7. RP-0316-17614-NP, Human Factors Engineering Operating Experience Review Results Summary Report, December 7, 2016, Revision 0.
 8. RP-0316-17615-NP, Human Factors Engineering Functional Requirements Analysis and Function Allocation Results Summary Report, December 2, 2016, Revision 0.
 9. RP-0316-17616-NP, Human Factors Engineering Task Analysis Results Summary Report, April 2019, Revision 2.
 10. RP-0316-17617-NP, Human Factors Engineering Staffing and Qualifications Results Summary Report, December 2, 2016, Revision 0.
 11. RP-0316-17618-NP, Human Factors Engineering Treatment of Important Human Actions Results Summary Report, December 2, 2016, Revision 0.
 12. RP-0316-17619-NP, Human Factors Engineering Human-System Interface Design Results Summary Report, April 2019, Revision 2.
 13. RP-0516-49116-NP, Control Room Staffing Plan Validation Results, December 2, 2016, Revision 1.
 14. RP-0914-8534-NP, Human Factors Engineering Program Management Plan, April 2019, Revision 5.
 15. RP-0914-8543-NP, Human Factors Verification and Validation Implementation Plan, April 2019, Revision 5.
 16. RP-0914-8544-NP, Human Factors Engineering Design Implementation Implementation Plan, November 2019, Revision 4.
 17. RP-1018-61289-NP, Human Factors Engineering Verification and Validation Results Summary Report, July 2019, Revision 1.
 18. RP-1215-20253-NP, Control Room Staffing Plan Validation Methodology, December 2, 2016, Revision 3.
 19. TR-0116-20781-NP, Fluence Calculation Methodology and Results, July 2019, Revision 1.
 20. TR-0116-20825-NP-A, Applicability of AREVA Fuel Methodology for the NuScale Design, June 2016, Revision 1.
 21. TR-0116-21012-NP-A, NuScale Power Critical Heat Flux Correlations, December 2018, Revision 1.

22. TR-0316-22048-NP, Nuclear Steam Supply System Advanced Sensor Technical Report, May 2020, Revision 3.
23. TR-0515-13952-NP-A, Risk Significance Determination, October 2016, Revision 0.
24. TR-0516-49084-NP, Containment Response Analysis Methodology Technical Report, May 2020, Revision 3.
25. TR-0516-49416-NP-A, Non-Loss-of-Coolant Accident Analysis Methodology, July 2020, Revision 3.
26. TR-0516-49417-NP-A, Evaluation Methodology for Stability Analysis of the NuScale Power Module, March 2020, Revision 1.
27. TR-0516-49422-NP-A, Loss-of-Coolant Accident Evaluation Model, July 2020, Revision 2.
28. TR-0616-48793-NP-A, Nuclear Analysis Codes and Methods Qualification, November 2018, Revision 1.
29. TR-0616-49121-NP, NuScale Instrument Setpoint Methodology Technical Report, May 2020, Revision 3.
30. TR-0716-50350-NP-A, Rod Ejection Accident Methodology, June 2020, Revision 1.
31. TR-0716-50351-NP-A, NuScale Applicability of AREVA Method for the Evaluation of Fuel Assembly Structural Response to Externally Applied Forces, April 2020, Revision 1.
32. TR-0716-50424-NP, Combustible Gas Control, March 2019, Revision 1.
33. TR-0716-50439-NP, NuScale Comprehensive Vibration Assessment Program Analysis Technical Report, July 2019, Revision 2.
34. TR-0815-16497-NP-A, Safety Classification of Passive Nuclear Power Plant Electrical Systems, January 2018, Revision 1.
35. TR-0816-49833-NP, Fuel Storage Rack Analysis, November 2018, Revision 1.
36. TR-0816-50796-NP, Loss of Large Areas Due to Explosions and Fires Assessment, June 2019, Revision 1.
37. TR-0816-50797, Mitigation Strategies for Loss of All AC Power Event [NuScale Nonproprietary], October 2019, Revision 3.
38. TR-0816-51127-NP, NuFuel-HTP2™ Fuel and Control Rod Assembly Designs, December 2019, Revision 3.
39. TR-0818-61384-NP, Pipe Rupture Hazards Analysis, July 2019, Revision 2.
40. TR-0915-17564-NP-A, Subchannel Analysis Methodology, February 2019, Revision 2.
41. TR-0915-17565-NP-A, Accident Source Term Methodology, February 2020, Revision 4.
42. TR-0916-51299-NP, Long-Term Cooling Methodology, May 2020, Revision 3.
43. TR-0916-51502-NP, NuScale Power Module Seismic Analysis, April 2019, Revision 2.
44. TR-0917-56119-NP, CNV Ultimate Pressure Integrity, June 2019, Revision 1.
45. TR-0918-60894-NP, NuScale Comprehensive Vibration Assessment Program Measurement and Inspection Plan Technical Report, August 2019, Revision 1.

46. NP-TR-1010-859-NP-A, NuScale Topical Report: Quality Assurance Program Description for the NuScale Power Plant, May 2020, Revision 5.
47. TR-1015-18177-NP, Pressure and Temperature Limits Methodology, October 2018, Revision 2.
48. TR-1015-18653-NP-A, Design of the Highly Integrated Protection System Platform, May 2017, Revision 2.
49. TR-1016-51669-NP, NuScale Power Module Short-Term Transient Analysis, July 2019, Revision 1.
50. TR-1116-51962-NP, NuScale Containment Leakage Integrity Assurance, May 2019, Revision 1.
51. TR-1116-52065-NP, Effluent Release (GALE Replacement) Methodology and Results, November 2018, Revision 1.

We have uploaded a draft of the final rule (RIN 3150-AJ98) in MS Word as well as an MS Word file for this letter.

Please contact Helen Chang of my staff at 301-415-3228 if you have any questions.

Sincerely,



Cindy Bladey,
Federal Register Liaison Officer

Docket No. NRC-2017-0029

Enclosures: As stated