



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 22, 2021

Mr. Daniel G. Stoddard
Senior Vice President and
Chief Nuclear Officer
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, VA 23060-6711

SUBJECT: NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF
AMENDMENT NOS. 289 AND 272 TO REVISE REACTOR CORE SAFETY
LIMIT TO REFLECT WCAP-17642-P-A (EPID L-2021-LLA-0011)

Dear Mr. Stoddard:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 289 and 272 to Renewed Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station (North Anna), Unit Nos. 1 and 2, respectively. These amendments are in response to your application dated January 7, 2021, as supplemented by letter dated November 16, 2021.

The amendments revise the North Anna, Unit Nos. 1 and 2, Technical Specifications "Reactor Core SLs [Safety Limits]" 2.1.1.2 to reflect the peak fuel centerline melt temperature specified in Topical Report WCAP-17642-P-A, Revision 1, "Westinghouse Performance Analysis and Design Model (PAD5)."

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

If you have any questions, please contact me at (301) 415-2481 or Ed.Miller@nrc.gov.

Sincerely,

/RA/

G. Edward Miller, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosures:

1. Amendment No. 289 to NPF-4
2. Amendment No. 272 to NPF-7
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 289
Renewed License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated January 7, 2021, as supplemented by letter dated November 16, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to paragraph 2.C (2) of Renewed Facility Operating License No. NPF-4, as indicated in the attachment to this license amendment, and is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendices A, as revised through Amendment No. 289, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Operation

Attachment:
Changes to Renewed Facility
Operating License No. NPF-4
and Technical Specifications

Date of Issuance: December 22, 2021



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 272
Renewed License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated January 7, 2021, as supplemented by letter dated November 16, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to paragraph 2.C (2) of Renewed Facility Operating License No. NPF-7, as indicated in the attachment to this license amendment, and is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 272, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Operation

Attachment:
Changes to Renewed Facility
Operating License No. NPF-7
and Technical Specifications

Date of Issuance: December 22, 2021

ATTACHMENT TO
NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2
LICENSE AMENDMENT NO. 289
RENEWED FACILITY OPERATING LICENSE NO. NPF-4
DOCKET NO. 50-338
AND LICENSE AMENDMENT NO. 272
RENEWED FACILITY OPERATING LICENSE NO. NPF-7
DOCKET NO. 50-339

Replace the following pages of the Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

NPF-4, page 3
NPF-7, page 3

Insert

NPF-4, page 3
NPF-7, page 3

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contain marginal lines indicating the areas of change.

Remove

2.0-1

Insert

2.0-1

- (2) Pursuant to the Act and 10 CFR Part 70, VEPCO to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report;
 - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I; Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

VEPCO is authorized to operate the North Anna Power Station, Unit No. 1, at reactor core power levels not in excess of 2940 megawatts (thermal).
 - (2) Technical Specifications

Technical Specifications contained in Appendix A, as revised through Amendment No. 289 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
 - (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations as set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

VEPCO is authorized to operate the facility at steady state reactor core power levels not in excess of 2940 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 272 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the insurance of the condition or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission:

- a. If VEPCO plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the North Anna Power Station, the

2.0 SAFETY LIMITS (SLs)

2.1 SLs

2.1.1 Reactor Core SLs

In MODES 1 and 2, the combination of THERMAL POWER, Reactor Coolant System (RCS) highest loop average temperature, and pressurizer pressure shall not exceed the limits specified in the COLR; and the following SLs shall not be exceeded.

2.1.1.1 The departure from nucleate boiling ratio (DNBR) shall be maintained greater than or equal to the 95/95 DNBR criterion for the DNB correlations and methodologies specified in Section 5.6.5.

2.1.1.2 The peak fuel centerline temperature shall be maintained < 5080°F, decreasing by 9°F per 10,000 MWD/MTU of burnup, for Westinghouse fuel and < 5173°F, decreasing by 65°F per 10,000 MWD/MTU of burnup, for Framatome fuel.

2.1.2 RCS Pressure SL

In MODES 1, 2, 3, 4, and 5, the RCS pressure shall be maintained ≤ 2735 psig.

2.2 SL Violations

2.2.1 If SL 2.1.1 is violated, restore compliance and be in MODE 3 within 1 hour.

2.2.2 If SL 2.1.2 is violated:

2.2.2.1 In MODE 1 or 2, restore compliance and be in MODE 3 within 1 hour.

2.2.2.2 In MODE 3, 4, or 5, restore compliance within 5 minutes.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 289 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-4

AND

AMENDMENT NO. 272 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-338 AND 50-339

1.0 INTRODUCTION

By application dated January 7, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21007A264), as supplemented by letter dated November 16, 2021 (ADAMS Accession No. ML21321A262), Dominion Energy Virginia (the licensee) submitted a license amendment request (LAR) requesting changes to the Technical Specifications (TSs) for North Anna Power Station, Units 1 and 2 (North Anna).

The proposed amendments would revise the TS "Reactor Core SLs [Safety Limits]" 2.1.1.2 peak fuel centerline temperature to reflect the fuel centerline melt temperature specified in Topical Report (TR) WCAP-17642-P-A, Revision 1, "Westinghouse Performance Analysis and Design Model (PAD5)," November 2017 (ADAMS Accession No. ML17338A396).

The supplement dated November 16, 2021, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the Federal Register on March 23, 2021 (86 FR 15506).

2.0 REGULATORY EVALUATION

2.1 System Description

The proposed amendments are related to the North Anna nuclear fuel. Reactor core SL 2.1.1.2 protects against overheating of the nuclear fuel and fuel cladding, which would lead to the fuel centerline temperature rising above the fuel melting point. The consequences of fuel and cladding overheating include fuel clad perforation, which leads to nuclear fission product release to the reactor coolant. Clad perforation can occur when the fuel centerline temperature reaches

the melting point for the fuel and the fuel pellet expands. Reactor core SL 2.1.1.2 is dependent on burnup and is applicable when the reactor is critical in MODES 1 and 2.

2.2 Regulations

The statutes and regulations that are applicable to the proposed TS SL 2.1.1.2 change include:

- Section 182a, "License Applications," of the Atomic Energy Act, which requires applicants for nuclear power plant operating licenses to include TSs as part of the license to enable the Commission to find that the utilization of special nuclear material will be in accord with the common defense and security and will provide adequate protection to the health and safety of the public.
- Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36, "Technical Specification," which requires that TSs include items in the following categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; (5) administrative controls; (6) decommissioning; (7) initial notification; and (8) written reports.
- Section 50.36(b) requires that the Technical Specifications will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto.
- Regulations in 10 CFR 50.90, "Application for amendment of license, construction permit, or early site permit," specifies an application must be filed with the Commission, as specified in §§ 50.4 or 52.3 of this chapter, as applicable, fully describing the changes desired, and following as far as applicable, the form prescribed for original applications.
- Appendix A to 10 CFR Part 50, "General Design Criteria for Nuclear Power Plants [GDC]," Criterion 10, "Reactor design," which requires that the "reactor core and associated coolant, control, and protection systems shall be designed with appropriate margin to assure that specified acceptable fuel design limits are not exceeded during any condition of normal operation, including the effects of anticipated operational occurrences." Section 3.1.6 of the North Anna Updated Final Safety Analysis Report discusses how the plant structures, systems, and components meet the intent of GDC Criterion 10.

2.3 Licensee's Proposed Changes

In its submittal dated January 7, 2021, the licensee proposed to revise the TS reactor core SL 2.1.1.2 peak fuel centerline temperature to reflect the fuel centerline melt temperature specified in TR WCAP-17642-P-A, Revision 1. The changed TS SL is indicated in **BOLD** font as provided below.

Current TS SL 2.1.1.2 states:

The peak fuel centerline temperature shall be maintained < 5080°F, decreasing by **58°F** per 10,000 MWD/MTU of burnup, for Westinghouse fuel and < 5173°F, decreasing by 65°F per 10,000 MWD/MTU of burnup, for Framatome fuel.

Revised TS SL 2.1.1.2 would state:

The peak fuel centerline temperature shall be maintained < 5080°F, decreasing by 9°F per 10,000 MWD/MTU of burnup, for Westinghouse fuel and < 5173°F, decreasing by 65°F per 10,000 MWD/MTU of burnup, for Framatome fuel.

3.0 TECHNICAL EVALUATION

The scope of the NRC staff's review is limited to the proposed change related to the SL for peak fuel centerline temperature. The proposed limit reflects the limit in TR WCAP-17642-P-A, Revision 1 (WCAP-17642-P-A).

The empirically derived fuel centerline melt temperature described in the PAD5 methodology is based on fuel properties described in open literature. The description of the fuel properties can be found in (1) S.G. Popov; J.J. Carbajo; V.K. Ivanov; and G.L. Yoder, "Thermophysical Properties of MOX and UO₂ Fuels Including the Effects of Irradiation," ORNL/TM-2000/351 (2000) and (2) J.J. Carbajo; G.L. Yoder; S.G. Popov; and V.K. Ivanov, "A Review of the Thermophysical Properties of MOX and UO₂ Fuels," Journal of Nuclear Materials, 299, 181 (2001). As noted beneath the caption for Figure 59, on page 92 of the NRC staff's SE on WCAP-17642-P-A (ADAMS Accession No. ML21007A264), in its approval of WCAP-17642-P-A, the NRC staff determined that this melting limit is acceptable. The burnup dependent fuel centerline melt temperature is based on inherent fuel properties and does not depend upon any specific calculational methodology. Therefore, the NRC staff considers it acceptable as a standalone limit.¹

3.1 Consideration of WCAP-17642-P-A Limitations and Conditions

The licensee stated that it intends to implement the WCAP-17642-P-A methodology and described how applicable limitations and conditions in the NRC staff's safety evaluation of the methodology would be addressed. While the proposed change to the SL is independent of the implementation of the WCAP-17642-P-A methodology, the staff reviewed this section of the LAR to ensure that the licensee's approach would support the proposed SL. The staff notes that there are other limitations and conditions in the PAD5 safety evaluation that were not addressed in the LAR. Since the licensee is not requesting to implement the portions of WCAP-17642-P-A that would necessitate those limitations and conditions, the NRC staff determined that the LAR need not address those limitations and conditions.

In Section 3.1 of its submittal dated January 7, 2021, the licensee addressed limitations and conditions (a) and (b) in Section 4.1 of the NRC staff's SE (Section A of ADAMS Accession No. ML17338A396). Adherence to limitation and condition (a) ensures that the proposed SL applies to North Anna. If limitation and condition (a) is not satisfied, then a different SL for fuel melting would be needed. In addition, limitation and condition (b) requires that, should PAD5 be implemented at any point, the North Anna specific analyses are not to be used to predict fuel conditions beyond the melting point.

¹ The two identified references provide the data describing the fuel properties. The specific burnup dependence is provided in Section 6.1.5 of TR WCAP-17642-P-A based on an assessment of these data. The NRC staff determined that this burnup dependence was acceptable as described in Section 3.7.12 of the staff's safety evaluation approving TR WCAP-17642-P-A.

Limitation and condition (a) states, in part:

The NRC staff limits the applicability of the PAD5 code and methodology for cladding, fuel types and reactor for the ranges that are listed...

The NRC staff confirmed that North Anna uses Westinghouse fuel that meets the constraints identified in limitation and condition (a). As such, the NRC staff has reasonable assurance that the proposed SL is applicable to North Anna in that it reflects an acceptable limit on the peak fuel centerline temperature that would preclude fuel melting. Based on its independent review of the confirmation provided by the licensee, the staff determined that this approach to address limitation and condition (a) is acceptable to change TS SL 2.1.1.2 for North Anna.

Limitation and condition (b) states:

The application of PAD5 should at no time exceed the fuel melting temperature as calculated by PAD5 due to the lack of properties for molten fuel in PAD5 and other properties such as thermal conductivity and FGR [fission gas release].

In its submittal dated January 7, 2021, the licensee stated that it “will limit the peak fuel centerline temperature per this amendment request.” Because the LAR specifically proposes to change the peak fuel centerline temperature SL to ensure that fuel melt is precluded during conditions for normal operation and anticipated operational occurrences, the NRC has reasonable assurance that proposed change will continue to meet limitation and condition (b).

3.2 TS SL 2.1.1.2 Change Evaluation

In its submittal dated January 7, 2021, the licensee requests to change the North Anna TS SL 2.1.1.2 to reflect the fuel centerline melt temperature found in the WCAP-17642-P-A methodology. The North Anna TS SL 2.1.1.2 limits the peak fuel centerline temperature. For Conditions I and II events (i.e., normal operation and anticipated operational occurrences), the reactor protection system is designed to ensure that the peak fuel centerline temperature does not exceed the fuel melt temperature criterion. The intent of this criterion is to avoid gross fuel melting.

The NRC staff determined that the peak fuel centerline temperature is not dependent on the WCAP-17642-P-A methodology, as discussed in the previous section. The staff evaluated the peak fuel centerline temperature and determined that it is acceptable for North Anna, since the properties of the fuel design in use at North Anna are consistent with the property data used to generate the proposed SL.

The proposed SL was empirically derived and covers the lower bound of the fuel property data, which is conservative. During its review of WCAP-17642-P-A methodology, the NRC staff compared the fuel centerline melting temperatures in WCAP-17642-P-A to the fuel centerline melting temperature in the NRC’s confirmatory calculation code, “FRAPCON.” The WCAP-17642-P-A fuel centerline melting temperature is less than that in FRAPCON, which is conservative. The NRC staff determined that its analysis and technical justification in the safety evaluation (SE, Section A of ADAMS Accession No. ML17338A396) of the WCAP-17642-P-A fuel centerline melting temperature is directly applicable to North Anna. Therefore, the NRC staff determined that the proposed SL is conservative for North Anna.

As discussed previously, the peak fuel centerline temperature SL is independent of the WCAP-17642-P-A methodology. Therefore, the NRC staff reviewed the proposed SL to ensure that it is supported by the current design and licensing basis safety analysis. The current safety analyses produced the existing SL 2.1.1.2 for fuel melt as an acceptable criterion, as required by the current methodology. Current licensing basis safety analyses that utilize the existing SL of 58 °F per 10,000 MWD/MTU will remain conservative in relation to the proposed SL of 9 °F per 10,000 MWD/MTU. Thus, the NRC staff determined that the proposed limit is acceptable under the North Anna current licensing basis.

In the licensee's UFSAR Section 4.2.1.1.1 (ADAMS Accession No. ML20309A607), the licensee discusses the fuel rod design bases for North Anna. Section 4.2.1.1.1 states that the thermal-hydraulic design assures that the maximum fuel temperature is below the UO₂ melting point of 5080 °F for unirradiated fuel, which decreases by 58 °F per 10,000 MWD/MTU. This UFSAR section also states that a calculated fuel centerline temperature of 4700 °F has been selected as the overpower limit. This conservative limit assures that the applicable safety analyses demonstrate that the proposed TS SL is not exceeded. Additionally, the licensee stated in its supplement dated November 16, 2021 that UFSAR Sections "Fuel Rods," 4.4.1.2, "Fuel Temperature Design Basis," and 4.4.2.2.6, "Treatment of Peaking Factors," will be revised as part of the implementation plan for the license amendments following their issuance to include the revised fuel melt safety limit and its technical basis. The licensee also stated in its supplement dated November 16, 2021, that UFSAR will be reviewed in its entirety and additional UFSAR changes will be implemented, if required, to reflect the revised peak fuel centerline melt temperature limit. Based on the above, including the LAR information that supports the use of the empirically derived and less restrictive temperature, the NRC staff concludes that the proposed increase in the peak fuel centerline temperature TS SL would continue to meet 10 CFR 50.36(b) in that the TSs would be derived from licensee safety analyses, as amended.

The proposed TS change meets the functional requirements of GDC 10 and meets the requirements of Section 182a of the Atomic Energy Act, because (1) the peak fuel centerline temperature is based on a conservative evaluation of test data that is applicable to the fuel design used at North Anna and (2) the proposed limit will ensure that fuel melt is precluded during conditions of normal operations and under anticipated operational occurrences. Therefore, the NRC Staff concludes that the proposed TS change would continue to meet 10 CFR 50.36(c)(1)(i)(A). The NRC finds there is reasonable assurance that the revised TS SL would maintain an important variable that is necessary to protect the integrity of a physical barrier that guards against the uncontrolled release of radioactivity and is, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Commonwealth of Virginia official was notified of the proposed issuance of the amendments. On November 18, 2021, the state official confirmed that the Commonwealth of Virginia had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational

radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration as published in the Federal Register on March 23, 2021 (86 FR 15506), and there has been no public comment on such finding . Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J. Miller, NRR
R. Sugrue, NRR

Date: December 22, 2021

SUBJECT: NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 289 AND 272 TO REVISE REACTOR CORE SAFETY LIMIT TO REFLECT WCAP-17642-P-A (EPID L-2021-LLA-0011) DATED DECEMBER 22, 2021

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OFFICE	NRR/DORL/LPL2-1/PM	NRR/DORL/LPL2-1/PM	NRR/DORL/LPL2-1/LA	NRR/DSS/SNRB/BC
NAME	SDevlin-Gill	GEMiller	KGoldstein	RPatton
DATE	09/02/2021	11/18/2021	12/03/2021	11/22/2021
OFFICE	NRR/DSS/STSB/BC	OGC - NLO	NRR/DORL/LPL2-1/BC	NRR/DORL/LPL2-1/PM
NAME	NJordan (A)	STurk	MMarkley	GEMiller
DATE	12/20/2021	12/20/2021	12/22/2021	12/22/2021

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