



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

HOLTEC DECOMMISSIONING INTERNATIONAL, LLC

HOLTEC PALISADES, LLC

DOCKET NO. 50-155

BIG ROCK POINT PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. XXX  
License No. DPR-6

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by Entergy Nuclear Operations, Inc., on behalf of itself; Entergy Nuclear Palisades, LLC; Holtec International; and Holtec Decommissioning International, LLC, dated December 23, 2020, as supplemented by letters dated December 23, 2020, December 23, 2020, and October 29, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-6 is amended as indicated in the attachment to this license amendment.
3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from that date.

FOR THE NUCLEAR REGULATORY COMMISSION

Yaira K. Diaz-Sanabria, Chief  
Storage and Transportation Licensing Branch  
Division of Fuel Management  
Office of Nuclear Material Safety and Safeguards

Attachments:  
Changes to Facility Operating  
License No. DPR-6  
and Technical Specifications

Date of Issuance:

DRAFT

ATTACHMENT TO LICENSE AMENDMENT NO. XXX

BIG ROCK POINT PLANT

FACILITY OPERATING LICENSE NO. DPR-6

DOCKET NO. 50-155

Replace the following pages of Facility Operating License No. DPR-6 and Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License No DPR-6

Remove Pages

-1- through -3-

Insert Pages

-1- through -3-

Appendix A, Technical Specifications

Remove Pages

6-1

6-2

Insert Pages

6-1

6-2

HOLTEC PALISADES, LLC

HOLTEC DECOMMISSIONING INTERNATIONAL, LLC

DOCKET NO. 50-155

BIG ROCK POINT PLANT

FACILITY OPERATING LICENSE

License No. DPR-06

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for a license amendment by Holtec Palisades, LLC (Holtec Palisades) and Holtec Decommissioning International, LLC (HDI), (the licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance that: (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
  - D. The licensees are technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the rules and regulations of the Commission and all applicable requirements have been satisfied;
  - F. The licensees have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
  - G. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - H. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70, including 10 CFR Sections 30.33, 40.32, 70.23, and 70.31.

Amendment No. 427, XXX  
[Month xx, Year]

2. Facility Operating License No. DPR-06, is hereby issued to Holtec Palisades and HDI in its entirety to read as follows:
  - A. This license applies to the Big Rock Point Plant (the facility) owned by Holtec Palisades. The facility is located in Charlevoix County, Michigan, and is described in the licensee's application dated January 14, 1960, and the Final Hazards Summary Report, as supplemented, updated, and amended by subsequent filings by the licensee.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Holtec Palisades and HDI:
    - B.(1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," to possess the facility at the designated location in Charlevoix County, Michigan, in accordance with the procedures and limitations set forth in this license;
    - B.(2) Pursuant to the Act and 10 CFR Part 70, "Domestic Licensing of Special Nuclear Material," to possess at any one time up to: a) 2500 kilograms of contained uranium-235 in fuel rods, b) 10.32 grams of uranium-235 as contained in fission counters, c) 150 kilograms of plutonium contained in PuO<sub>2</sub>-UO<sub>2</sub> fuel rods, and d) 5 curies of plutonium encapsulated as a plutonium-beryllium neutron source,
      - (a) Deleted
      - (b) Deleted
      - (c) Deleted
      - (d) Deleted
      - (e) Deleted
      - (f) Deleted
      - (g) Deleted
    - B.(3) Pursuant to the Act and 10 CFR Part 30, "Rules of General Applicability to Domestic Licensing of Byproduct Material," to receive, possess, and use at any one time up to 7000 curies of antimony-beryllium in the form of neutron sources, 3.7 curies of cobalt-60 as sealed sources, 45 curies of cesium-137 as sealed sources, 10 microcuries of miscellaneous alpha-emitting material as sealed sources, and up to 500 millicuries per nuclide of any byproduct material between atomic numbers 1 and 83, inclusive, without restriction as to chemical and physical form;
    - B.(4) Pursuant to the Act and 10 CFR Part 40, "Domestic Licensing of Source Material," to possess at any one time up to 500 kilograms of depleted uranium dioxide contained in the facility's fuel assembly;

B.(5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

C.(1) Reactor Operation

The reactor is not licensed for power operation. Fuel shall not be placed in the reactor vessel.

C.(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. XXX, are hereby incorporated in the license. HDI shall maintain the facility in accordance with the Technical Specifications.

C.(3) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the physical security, guard training and qualification, and safeguards contingency plans approved by the Commission and all amendments and revisions to such plans made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p), as modified by NRG-approved exemptions. The plan, which contains safeguards information protected under 10 CFR 73.21, is entitled: "Big Rock Point ISFSI Security Plan," as submitted on July 31, 2001, and modified by letter dated March 6, 2002. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

C.(4) License Termination Plan

The License Termination Plan (LTP) dated April 1, 2003, as supplemented by LTP, Rev. 1, dated July 1, 2004, is approved by NRC License Amendment No. 126.

In addition to those criteria specified in 10 CFR 50.59, 10 CFR 50.82(a)(6), and 10 CFR 50.82(a)(7), changes to the approved LTP shall require NRC approval prior to being implemented if the change:

(a) Increases in radionuclide-specific derived concentration guideline levels of area factors (discussed in Chapter 6 of the LTP);

# BIG ROCK POINT DEFUELED TECHNICAL SPECIFICATIONS

## **6.0 ADMINISTRATIVE CONTROLS**

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### **6.1 RESPONSIBILITY AND AUTHORITY**

#### **6.1.1 SENIOR NUCLEAR OFFICER**

The Senior Nuclear Officer shall be the HDI President and shall be responsible for the overall operation, maintenance and decommissioning of the Big Rock Point nuclear power plant.

#### **6.1.2 SITE GENERAL MANAGER**

The site general manager shall be responsible for overall facility operation, maintenance and decommissioning and for periods of absence shall delegate in writing the succession to this responsibility. Unless otherwise specified, the site general manager's delegate has authority to perform all actions and grant approvals assigned by these specifications to the site general manager. The site general manager may delegate specific tasks to other individuals who may perform those tasks whether the site general manager is absent or present at the site.

### **6.2 ORGANIZATION**

#### **6.2.1 REPORTING RELATIONSHIPS**

Onsite organization and corporate reporting relationship shall be established as described in the Quality Program Description for Big Rock Point.

#### **6.2.2 FACILITY ORGANIZATION**

The site general manager or his designate shall verify that required security staffing and Dry Fuel Storage Technical Specification surveillance(s) have been met.

# BIG ROCK POINT DEFUELED TECHNICAL SPECIFICATIONS

## **6.0 ADMINISTRATIVE CONTROLS**

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### **6.3 STAFF QUALIFICATIONS**

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Quality Assurance Program Manual (QAPM).

### **6.4 REVIEW AND AUDIT**

Requirements for onsite and offsite reviews and audits are described in the Quality Program Description for Big Rock Point.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

HOLTEC DECOMMISSIONING INTERNATIONAL, LLC

HOLTEC PALISADES, LLC

DOCKET NO. 50-255

PALISADES NUCLEAR PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. XXX  
License No. DPR-20

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by Entergy Nuclear Operations, Inc., on behalf of itself; Entergy Nuclear Palisades, LLC; Holtec International; and Holtec Decommissioning International, LLC, dated December 23, 2020, as supplemented by letters dated December 23, 2020, December 23, 2020, and October 29, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, Renewed Facility Operating License No. DPR-20 is amended as indicated in the attachment to this license amendment.
3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from that date.

FOR THE NUCLEAR REGULATORY COMMISSION

Andrea D. Veil, Director  
Office of Nuclear Reactor Regulation

Attachments:  
Changes to Renewed Facility  
Operating License No. DPR-20  
and Technical Specifications

Date of Issuance:

DRAFT

ATTACHMENT TO LICENSE AMENDMENT NO. XXX

PALISADES NUCLEAR PLANT

RENEWED FACILITY OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Replace the following pages of Renewed Facility Operating License No. DPR-20 and Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating License No. DPR-20

Remove Pages

-1- through -3-  
-5a-  
-6-  
-7-

Insert Pages

-1- through -3-  
-5a-  
-6-  
-7-

Appendix A, Technical Specifications

Remove Pages

4.0-1

Insert Pages

4.0-1

HOLTEC PALISADES, LLC

HOLTEC DECOMMISSIONING INTERNATIONAL, LLC

DOCKET NO. 50-255

PALISADES NUCLEAR PLANT

RENEWED FACILITY OPERATING LICENSE

Renewed License No. DPR-20

1. The Nuclear Regulatory Commission (NRC or the Commission) having previously made the findings set forth in Operating License No. DPR-20, dated February 21, 1991, has now found that:
  - A. The application for Renewed Operating License No. DPR-20 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Palisades Plant (the facility) has been completed in conformity with Provisional Construction Permit No. CPPR-25 and the application, as amended, the provisions of the Act, and the regulations of the Commission, and has been operating under facility operating license since February 21, 1991;
  - C. Actions have been identified and have been or will be taken with respect to:
    - (1) managing the effects of aging on the functionality of structures and components that have been identified to require review under 10 CFR 54.21(a)(1) during the period of extended operation, and
    - (2) time-limited aging analyses that have been identified to require review under 10 CFR 54.21(c), such that there is reasonable assurance that the activities authorized by this renewed operating license will continue to be conducted in accordance with the current licensing basis, as defined in 10 CFR 54.3 for the facility, and that any changes made to the facility's current licensing basis in order to comply with 10 CFR 54.29(a) are in accordance with the Act and the Commission's regulations;

- D. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - E. There is reasonable assurance: (i) that the activities authorized by this renewed operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - F. Holtec Palisades, LLC (Holtec Palisades) is financially qualified and Holtec Decommissioning International, LLC (HDI) is financially and technically qualified to engage in the activities authorized by this renewed operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;
  - G. Holtec Palisades and HDI have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements" of the Commission's regulations;
  - H. The issuance of this renewed operating license will not be inimical to the common defense and security or to the health and safety of the public;
  - I. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of this renewed Facility Operating License No. DPR-20, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 51 (formerly Appendix D to Part 50), of the Commission's regulations and all applicable requirements have been satisfied; and
  - J. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this renewed operating license will be in accordance with 10 CFR Parts 30, 40, and 70.
2. Operating License No. DPR-20, dated February 21, 1991, as amended, was superseded in its entirety by Renewed Facility Operating License No. DPR-20, is hereby issued to Holtec Palisades and HDI as follows:
- A. This renewed license applies to the Palisades Plant, a pressurized light water moderated and cooled reactor and electrical generating equipment (the facility). The facility is located in Van Buren County, Michigan, and is described in the Palisades Plant Updated Final Safety Analysis Report, as supplemented and amended, and in the Palisades Plant Environmental Report, as supplemented and amended.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:

- (1) Pursuant to Section 104b of the Act, as amended, and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," (a) Holtec Palisades to possess and use, and (b) HDI to possess, use and operate, the facility as a utilization facility at the designated location in Van Buren County, Michigan, in accordance with the procedures and limitation set forth in this license;
  - (2) HDI, pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use source and special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;
  - (3) HDI, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use byproduct, source, and special nuclear material as sealed sources for reactor startup, reactor instrumentation, radiation monitoring equipment calibration, and fission detectors in amounts as required;
  - (4) HDI, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material for sample analysis or instrument calibration, or associated with radioactive apparatus or components; and
  - (5) HDI, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operations of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act; to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) HDI is authorized to operate the facility at steady-state reactor core power levels not in excess of 2565.4 Megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.
  - (2) The Technical Specifications contained in Appendix A, as revised through Amendment No. XXX, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. HDI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
  - (3) Fire Protection  
  
HDI shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the license amendment request dated December 12, 2012, November 1, 2017, November 1, 2018, and March 8, 2019, as supplemented by letters dated February 21, 2013, September 30, 2013, October 24, 2013,

margins are maintained when changes are made to the fire protection program.

(c) Transition License Conditions

1. Before achieving full compliance with 10 CFR 50.48(c), as specified by 2, below, risk-informed changes to the licensee's fire protection program may not be made without prior NRC review and approval unless the change has been demonstrated to have no more than a minimal risk impact, as described in 2. above.
  2. The licensee shall implement the modifications to its facility, as described in Table S-2, "Plant Modifications Committed," of Entergy Nuclear Operations, Inc. (ENO) letter PNP 2019-028 dated May 28, 2019, to complete the transition to full compliance with 10 CFR 50.48(c) before the end of the refueling outage following the fourth full operating cycle after NRC approval. The licensee shall maintain appropriate compensatory measures in place until completion of these modifications.
  3. The licensee shall implement the items listed in Table S-3, "Implementation Items," of ENO letter PNP 2014-097 dated November 4, 2014, within six months after NRC approval, or six months after a refueling outage if in progress at the time of approval with the exception of Implementation Items 3 and 8 which will be completed once the related modifications are installed and validated in the PRA model.
- (4) The following requirements shall apply to control rod drive CRD-13 during cycle 25:
- (a) Performance of Technical Specifications Surveillance Requirement SR 3.1.4.3 is not required for CRD-13 until the next entry into Mode 3.
  - (b) Seal leakage on CRD-13 shall be repaired prior to entering Mode 2, following the next Mode 3 entry.
  - (c) The reactor shall be shut down if CRD-13 seal leakage exceeds two gallons per minute.
- (5) [deleted]

- D. The facility has been granted certain exemptions from Appendix J to 10 CFR Part 50, "Primary Reactor Containment Leakage Testing for Water Cooled Power Reactors." This section contains leakage test requirements, schedules and acceptance criteria for tests of the leak-tight integrity of the primary reactor containment and systems and components which penetrate the containment. These exemptions were granted in a letter dated December 6, 1989.

These exemptions granted pursuant to 10 CFR 50.12, are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

- E. HDI shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Palisades Nuclear Plant Physical Security Plan."

HDI shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The Palisades CSP was approved by License Amendment No. 243 as supplemented by changes approved by License Amendment Nos. 248, 253, 259, and 264.

- F. [deleted]

- G. Holtec Palisades and HDI shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.



- H. The Updated Safety Analysis Report supplement, as revised, submitted pursuant to 10 CFR 54.21(d), shall be included in the next scheduled update to the Updated Safety Analysis Report required by 10 CFR 50.71(e)(4) following the issuance of this renewed operating license. Until that update is complete, HDI may make changes to the programs and activities described in the supplement without prior Commission approval, provided that HDI evaluates such changes pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.
- I. The Updated Safety Analysis Report supplement, as revised, describes certain future activities to be completed prior to the period of extended operation. ENO shall complete these activities no later than March 24, 2011, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.
- J. All capsules in the reactor vessel that are removed and tested must meet the test procedures and reporting requirements of American Society for Testing and Materials (ASTM) E 185-82 to the extent practicable for the configuration of the specimens in the capsule. Any changes to the capsule withdrawal schedule, including spare capsules, must be approved by the NRC prior to implementation. All capsules placed in storage must be maintained for future insertion. Any changes to storage requirements must be approved by the NRC, as required by 10 CFR Part 50, Appendix H.
- K. This license is effective as of the date of issuance and shall expire at midnight on March 24, 2031.

FOR THE NUCLEAR REGULATORY COMMISSION

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J. E. Dyer, Director  
Office of Nuclear Reactor Regulation

Attachments:

- 1. Appendix A - Technical Specifications
- 2. Appendix B - Environmental Protection Plan

Date of Issuance: January 17, 2007

## 4.0 DESIGN FEATURES

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### 4.1 Site Location

The Palisades Nuclear Plant is located on property owned by Holtec Palisades, LLC on the eastern shore of Lake Michigan approximately four and one-half miles south of the southern city limits of South Haven, Michigan. The minimum distance to the boundary of the exclusion area as defined in 10 CFR 100.3 shall be 677 meters.

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### 4.2 Reactor Core

#### 4.2.1 Fuel Assemblies

The reactor core shall contain 204 fuel assemblies. Each assembly shall consist of a matrix of zircaloy-4 or M5 clad fuel rods with an initial composition of depleted, natural, or slightly enriched uranium dioxide (UO<sub>2</sub>) as fuel material. Limited substitutions of zirconium alloy or stainless steel filler rods for fuel rods, in accordance with approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with applicable NRC staff approved codes and methods and shown by tests or analyses to comply with all fuel safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core regions. A core plug or plugs may be used to replace one or more fuel assemblies subject to the analysis of the resulting power distribution. Poison may be placed in the fuel bundles for long-term reactivity control.

#### 4.2.2 Control Rod Assemblies

The reactor core shall contain 45 control rods. Four of these control rods may consist of part-length absorbers. The control material shall be silver-indium-cadmium, as approved by the NRC.

### 4.3 Fuel Storage

#### 4.3.1 Criticality

4.3.1.1 The Region I (See Figure B 3.7.16-1) Carborundum equipped fuel storage racks incorporating Regions 1A, 1B, 1C, 1D, and 1E are designed and shall be maintained with:

- a. New or irradiated fuel assemblies having a maximum nominal planar average U-235 enrichment of 4.54 weight percent;