

U.S. NUCLEAR REGULATORY COMMISSION

DRAFT REGULATORY GUIDE DG-1400

Proposed Revision 2 to Regulatory Guide 1.156
Preliminary Draft for ACRS Consideration
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QUALIFICATION OF CLASS 1E CONNECTION ASSEMBLIES FOR NUCLEAR POWER GENERATING STATIONS AND OTHER NUCLEAR FACILITIES

A. INTRODUCTION

Purpose

This regulatory guide (RG) describes an approach that is acceptable to the staff of the U.S. Nuclear Regulatory Commission (NRC) considers acceptable for complying with the Commission's regulations on the environmental qualification of connection assemblies and environmental seals in combination with cables or wires as assemblies for service in nuclear power plants and other nuclear facilities. It endorses the Institute of Electrical and Electronic Engineers (IEEE) standard 572-2019, "IEEE Standard for Qualification of Class 1E Connection Assemblies for Nuclear Power Generating Stations and Other Nuclear Facilities" (Ref. 1).

Applicability

This RG applies to nonreactor and reactor licensees subject to Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, "Domestic Licensing of Production and Utilization Facilities" (Ref. 2), 10 CFR Part 52, "Licenses, Certifications, and Approvals for Nuclear Power Plants" (Ref. 3). With respect to 10 CFR Part 50, this RG applies to holders of or an applicant for an operating license. With respect to 10 CFR Part 52, this RG applies to applicants and holders of combined licenses, standard design certifications, standard design approvals, and manufacturing licenses. This RG does not apply to nuclear power plants that have submitted license termination certifications as required by the 50.82(a)(1) and 52.110(a)(1). Though the guidance primarily reflects past reviews of large light-water nuclear power plant applications, this RG may also provide useful guidance to fuel cycle, spent fuel storage, waste disposal facilities, and non-power production or utilization licensees.

Applicable Regulations

- 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," provides regulations for licensing production and utilization facilities. 10 CFR Part 50 also requires, among other things, that structures, systems, and components (SSCs) that are important to safety in a nuclear power plant must be designed to accommodate the effects of environmental

This RG is being issued in draft form to involve the public in the development of regulatory guidance in this area. It has not received final staff review or approval and does not represent an NRC final staff position. Public comments are being solicited on this DG and its associated regulatory analysis. Comments should be accompanied by appropriate supporting data. Comments may be submitted through the Federal rulemaking Web site, <http://www.regulations.gov>, by searching for draft regulatory guide DG-1361. Alternatively, comments may be submitted to the Office of Administration, Mailstop: TWFN 7A-06M, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, ATTN: Program Management, Announcements and Editing Staff. Comments must be submitted by the date indicated in the *Federal Register* notice.

Electronic copies of this DG, previous versions of DGs and other recently issued guides are available through the NRC's public Web site under the Regulatory Guides document collection of the NRC Library at http://www.nrc.gov/reading-rm/doc-collections/reg_guides/. The DG is also available through the NRC's Agencywide Documents Access and Management System (ADAMS) at <http://www.nrc.gov/reading-rm/adams.html>, under Accession No. ML21288A562. The regulatory analysis may be found in ADAMS under Accession No. ML21288A561.

conditions (i.e., remain functional under postulated design-basis events (DBEs)). 10 CFR Part 50 also provides the regulatory requirements for licensing of non-power reactors

- 10 CFR 50.34, “Contents of applications; technical information,” requires evaluation of the design and performance of SSCs of the facility with the objective of assessing the risk to public health and safety resulting from operation of the facility and including determination of the margins of safety during normal operations and transient conditions anticipated during the life of the facility, and the adequacy of SSCs provided for the prevention of accidents and the mitigation of the consequences of accidents. Further, 10 CFR 50.34 requires a description and analysis of the SSCs of the facility, with emphasis upon performance requirements, the bases, with technical justification therefore, upon which such requirements have been established, and the evaluations required to show that safety functions will be accomplished.
- 10 CFR 50.49, “Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants,” the NRC requires that certain electric equipment important to safety be qualified for its application and specified performance. The regulation also identifies requirements for establishing environmental qualification methods and qualification parameters.
- Appendix B, “Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants,” Criterion III, “Design Control,” requires, in part, that test programs used to verify the adequacy of specific design features include suitable qualification testing of a prototype unit under the most adverse design conditions.
- 10 CFR Part 52, “Licenses, Certifications, and Approvals for Nuclear Power Plants,” requires that SSCs important to safety in a nuclear power plant be designed to accommodate the effects of environmental conditions and that design control measures, such as testing, be used to check the adequacy of the design.

Related Guidance

- RG 1.63, “Electric Penetration Assemblies in Containment Structures for Nuclear Power Plants,” (Ref. 4) provides guidance on the design, construction, testing, qualification, installation, and external circuit protection of electric penetration assemblies in containment structures of nuclear power plants.
- RG 1.89, “Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants,” (Ref. 5), provides guidance on environmental qualification of certain electric equipment important to safety for nuclear power plants.
- RG 1.100, “Seismic Qualification of Electrical and Active Mechanical Equipment and Functional Qualification of Active Mechanical Equipment for Nuclear Power Plants,” (Ref. 6), provides guidance on the seismic qualification of electrical and active mechanical equipment for nuclear power plants.
- RG 1.211, “Qualification of Safety-Related Cables and Field Splices for Nuclear Power Plants,” (Ref. 7) provides guidance on the qualification of safety-related cables and field splices for nuclear power plants.

- NUREG-1537, “Guidelines for Preparing and Reviewing Applications for the Licensing of Non-Power Reactors,” (Ref. 8) contains format and content guidance for non-power reactor applicants and licensees and contains a standard review plan and acceptance criteria.

Purpose of Regulatory Guides

The NRC issues RGs to describe methods that are acceptable to the staff for implementing specific parts of the agency’s regulations, to explain techniques that the staff uses in evaluating specific issues or postulated events, and to describe information that the staff needs in its review of applications for permits and licenses. Regulatory guides are not NRC regulations and compliance with them is not required. Methods and solutions that differ from those set forth in RGs are acceptable if supported by a basis for the issuance or continuance of a permit or license by the Commission.

Paperwork Reduction Act

This RG provides voluntary guidance for implementing the mandatory information collections in 10 CFR Parts 50 and 52 that are subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et. seq.). These information collections were approved by the Office of Management and Budget (OMB), approval numbers 3150-0011 and 3150-0151. Send comments regarding this information collection to the FOIA, Library, and Information Collections Branch (T6-A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer Office of Information and Regulatory Affairs (3150-0011 **3150-0011** and 3150-0151 **3150-0151**), Office of Management and Budget, Washington, DC20503

Public Protection Notification

The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.

B. DISCUSSION

Reason for Revision

Regulatory Guide (RG) 1.156, “Qualification of Class 1E Connection Assemblies for Nuclear Power Plants,” Revision 1 was issued in July 2011. Given that environmental qualification is a recent focus area this revision of the guide (Revision 2) updates the guidance to endorse, with clarification or exceptions, (IEEE) standard 572-2019, “IEEE Standard for Qualification of Class 1E Connection Assemblies for Nuclear Power Generating Stations and Other Nuclear Facilities”. This includes guidance used in conjunction with RG 1.89, “Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants”, as an acceptable method of demonstrating compliance with the NRC regulations pertaining to the environmental qualification of connectors, terminations, and environmental seals in combination with cables or wires as assemblies for service in nuclear power plants to ensure that the connection assemblies can perform their safety functions.

Background

IEEE Standard (Std.) 572-2019, “Qualification of Class 1E Connection Assemblies for Nuclear Power Generating Stations and Other Nuclear Facilities,” was prepared by Subcommittee 2 (Qualification) of the IEEE Nuclear Power Engineering Committee and was approved by the IEEE Standards Board on February 8, 2019. This standard describes basic procedures for qualifying connection assemblies (e.g., connectors, terminations, and environmental seals in combination with related cables or wires as assemblies). The qualification requirements in this standard, when followed, demonstrate and document the ability of the equipment to perform safety functions under applicable service conditions, including design-basis events.

Although the guidance primarily reflects past reviews of large light-water nuclear power plant applications, this RG may also provide useful guidance for nuclear facilities (e.g., fuel cycle, spent fuel storage, and waste disposal facilities) .

Consideration of International Standards

The International Atomic Energy Agency (IAEA) works with member states and other partners to promote the safe, secure, and peaceful use of nuclear technologies. The IAEA develops Safety Requirements and Safety Guides for protecting people and the environment from harmful effects of ionizing radiation. This system of safety fundamentals, safety requirements, safety guides, and other relevant reports, reflects an international perspective on what constitutes a high level of safety. To inform its development of this RG, the NRC considered IAEA Safety Requirements and Safety Guides pursuant to the Commission’s International Policy Statement (Ref. 9) and Management Directive and Handbook 6.6, “Regulatory Guides” (Ref. 10).

The NRC has a goal of harmonizing its regulatory guidance with documents issued by the IAEA to the extent practical. Although the NRC does not endorse IAEA Safety Report Series No. 3, “Equipment Qualification in Operational Nuclear Power Plants: Upgrading, Preserving, and Reviewing,” issued April 1998 (Ref. 11), RG 1.89 incorporates similar guidelines and is generally consistent with the basic safety principles provided in it. Specifically, IAEA Safety Report Series No. 3 provides guidance for assessing the status of equipment qualification in operational nuclear power plants and preserving their status. RG 1.89 incorporates similar EQ recommendations and is generally consistent with the basic safety principles in IAEA Safety Report Series No. 3.

Documents Discussed in Staff Regulatory Guidance

This RG endorses, in part, the use of one or more codes or standards developed by external organizations, and other third-party guidance documents. These codes, standards and third-party guidance documents may contain references to other codes, standards or third-party guidance documents (“secondary references”). If a secondary reference has itself been incorporated by reference into NRC regulations as a requirement, then licensees and applicants must comply with that standard as set forth in the regulation. If the secondary reference has been endorsed in a RG as an acceptable approach for meeting an NRC requirement, then the standard constitutes a method acceptable to the NRC staff for meeting that regulatory requirement as described in the specific RG. If the secondary reference has neither been incorporated by reference into NRC regulations nor endorsed in a RG, then the secondary reference is neither a legally-binding requirement nor a “generic” NRC approved acceptable approach for meeting an NRC requirement. However, licensees and applicants may consider and use the information in the secondary reference, if appropriately justified, consistent with current regulatory practice, and consistent with applicable NRC requirements.

C. STAFF REGULATORY GUIDANCE

The NRC staff considers conformance with the requirements of IEEE Standard 572-2019 an acceptable method for use in satisfying the Commission's regulations with respect to environmental qualification of connection assemblies and environmental seals in combination with cables or wires as assemblies for service in nuclear power plants and other nuclear facilities.

D. IMPLEMENTATION

The NRC staff may use this RG as a reference in its regulatory processes, such as licensing, inspection, or enforcement. However, the NRC staff does not intend to use the guidance in this RG to support NRC staff actions in a manner that would constitute backfitting as that term is defined in 10 CFR 50.109, "Backfitting," and as described in NRC Management Directive 8.4, "Management of Backfitting, Forward Fitting, Issue Finality, and Information Requests," (Ref. 12), nor does the NRC staff intend to use the guidance to affect the issue finality of an approval under 10 CFR Part 52, "Licenses, Certifications, and Approvals for Nuclear Power Plants." The staff also does not intend to use the guidance to support NRC staff actions in a manner that constitutes forward fitting as that term is defined and described in Management Directive 8.4.

REFERENCES¹

1. IEEE Std. 572-2019, “Qualification of Class 1E Connection Assemblies for Nuclear Power Generating Stations,” Institute of Electrical and Electronics Engineers, Piscataway, NJ, February 2019.²
2. U.S. Code of Federal Regulation (CFR), “Domestic Licensing of Production and Utilization Facilities,” Part 50, Chapter 1, Title 10, “Energy.”
3. CFR, “Licenses, Certifications, and Approvals for Nuclear Power Plants,” Part 52, Chapter 1, Title 10, “Energy.”
4. NRC, Regulatory Guide (RG) 1.63, “Electric Penetration Assemblies in Containment Structures for Nuclear Power Plants,” U. S. Nuclear Regulatory Commission, Washington, DC.
5. NRC, RG 1.89, “Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants,” U. S. Nuclear Regulatory Commission, Washington, DC.
6. NRC, RG 1.100, “Seismic Qualification of Electrical and Active Mechanical Equipment and Functional Qualification of Active Mechanical Equipment for Nuclear Power Plants,” U.S. Nuclear Regulatory Commission, Washington, DC.
7. NRC, RG 1.211, “Qualification of Safety-Related Cables and Field Splices for Nuclear Power Plants,” U.S. Nuclear Regulatory Commission, Washington, DC.
8. NRC, NUREG-1537, “Guidelines for Preparing and Reviewing Applications for the Licensing of Non-Power Reactors.
9. NRC, “Nuclear Regulatory Commission International Policy Statement,” Federal Register, Vol. 79, No. 132, July 10, 2014, pp. 39415-39418.
10. NRC, Management Directive (MD) 6.6, “Regulatory Guides,” Washington, DC, May 2, 2016 (ADAMS Accession No. ML18073A170).
11. IAEA Safety Report Series No. 3, “Equipment Qualification in Operational Nuclear Power Plants: Upgrading, Preserving, and Reviewing,” issued April 1998²
12. NRC, “Management of Backfitting, Forward Fitting, Issue Finality, and Information Requests,” Management Directive 8.4 (ADAMS Accession No. ML18093B087).

¹ Publicly available NRC-published documents are available electronically through the NRC Library on the NRC’s public Web site at: <http://www.nrc.gov/reading-rm/doc-collections/>. The documents can also be viewed online or printed for a fee in the NRC’s Public Document Room (PDR) at 11555 Rockville Pike, Rockville, MD; the mailing address is USNRC PDR, Washington, DC 20555; telephone 301-415-4737 or (800) 397-4209; fax (301) 415-3548; and e-mail PDR.Resource@nrc.gov.

² Copies of Institute of Electrical and Electronics Engineers (IEEE) documents may be purchased from the Institute of Electrical and Electronics Engineers Service Center, 445 Hoes Lane, PO Box 1331, Piscataway, NJ 08855 or through the IEEE’s public Web site at http://www.ieee.org/publications_standards/index.html

² Copies of International Atomic Energy Agency (IAEA) documents may be obtained through its Web site: www.iaea.org/ or by writing the International Atomic Energy Agency P.O. Box 100 Wagramer Strasse 5, A-1400 Vienna, Austria. Telephone (+431) 2600-0, Fax (+431) 2600-7, or E-Mail at Official.Mail@IAEA.Org