

UNITED STATES NUCLEAR REGULATORY COMMISSION

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November 3, 2021

MEMORANDUM TO: Dennis C. Morey, Chief

Licensing Processes Branch

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

FROM: Joseph J. Holonich, Senior Project Manager /RA/

Licensing Processes Branch

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF OCTOBER 12, 2021, MEETING BETWEEN THE

U.S. NUCLEAR REGULATORY COMMISSION STAFF AND THE ELECTRIC POWER RESEARCH INSTITUTE TO DISCUSS MRP-231,

REVISION 4 (EPID: L-2020-PPM-0002)

On October 12, 2021, the U.S. Nuclear Regulatory Commission (NRC) staff met with representatives from the Electric Power Research Institute (EPRI). The purpose of the meeting was to discuss the "Materials Reliability Program: Aging Management Strategies for B&W Pressurized Water Reactor Internals (MRP-231, Rev. 4)." The information related to the meeting including the meeting attendee list can be found in Agencywide Documents Access and Management System Package Accession No. ML21285A021.

The meeting was bifurcated in two parts. The first part was open to the public. In that part, the EPRI representatives provided a general overview of MRP-231, Revision 4. In the closed portion of the meeting (second part), the EPRI representatives discussed in detail the information contained in MRP-231, Revision 4. A nonproprietary version of the presentation was not provided and thus is not available for stakeholders. A few questions were asked by the NRC staff to help gain a better understanding of the analyses.

During the meeting, EPRI representatives discussed additional guidance documents. The NRC staff requested that EPRI provide those additional guidance documents and the EPRI representatives agreed. Specific documents include:

CSB-21-04 dated July 21, 2021, "Post Stress Relief Weld Repairs in the B&W Core Barrel and Impact on SCC Susceptibility"

MRP-2021-005 dated September 10, 2021, "NEI 'Needed' Interim Guidance regarding MRP-227-A and MRP-227, Revision 1-A B&W Core Barrel Cylinder Top Flange Circumferential Weld Heat-Affected Zone Inspection"

Docket No. 99902021

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While closing the meeting, the NRC staff stated that this meeting along with several other informational meetings on different reports will help provide the NRC staff with a technical base for the review of MRP-227, Revision 2, "Pressurized Water Reactor (PWR) Internals Inspection and Evaluation Guidelines," when it is submitted.

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REVISION 4 (L-2020-PPM-0002) DATED NOVEMBER 3, 2021

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