GENERAL ATOMICS

1. NRC Approval Letter for the TRIGA Decommissioning Plan, dated August 12, 1999

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 12, 1999

Dr. Keith E. Asmussen, Director
Licensing, Safety and Nuclear Compliance
General Atomics
P.O. Box 85608
San Diego, California 92186-9784

SUBJECT: ISSUANCE OF AMENDMENT NO. 36 TO AMENDED FACILITY LICENSE NO. R-38 AND AMENDMENT NO. 45 TO FACILITY LICENSE NO. R-67 - GENERAL ATOMICS (TAC NOS. M98498 AND M98499)

Dear Dr. Asmussen:

The Commission has issued the enclosed Amendment No. 36 to Amended Facility License No. R-38 for the General Atomics TRIGA Mark I Research Reactor and Amendment No. 45 to Facility License No. R-67 for the General Atomics TRIGA Mark F Research Reactor. The amendment consists of changes to the facility license and technical specifications (TSs) in response to your application of April 18, 1997, as supplemented on November 20, 1998, and January 28 and 29, February 3, April 22, May 3 and 12, and June 15 (applicable to Mark I only), 16 (applicable to Mark F only) and 22, 1999.

The amendment approves the decommissioning plan for the General Atomics TRIGA Mark I and TRIGA Mark F Research Reactors. The amendment also authorizes the approved decommissioning plan to be included as a supplement to the Safety Analysis Report pursuant to 10 CFR 50.82(b)(5).

A copy of the safety evaluation supporting Amendment Nos. 36 and 45 is also enclosed.

Sincerely,

Alexander Adams, Jr.

Alexander Adams, Jr., Senior Project Manager
Events Assessment, Generic Communications and
Non-Power Reactors Branch
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-89 and
50-163

Enclosures:
1. Amendment Nos. 36 and 45
2. Safety Evaluation

cc w/enclosures: See next page

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GENERAL ATOMICS

DOCKET NO. 50-89

AMENDMENT TO AMENDED FACILITY LICENSE

Amendment No. 36
License No. R-38

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that

   A. The application for an amendment to Amended Facility License No. R-38 filed by General Atomics (the licensee) on April 18, 1997, as supplemented on November 20, 1998, and January 28 and 29, February 3, April 22, May 3 and 12, and June 15 and 22, 1999, conforms to the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the regulations of the Commission as stated in Chapter I of Title 10 of the *Code of Federal Regulations* (10 CFR);

   B. The facility will be decommissioned in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance that (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) such activities will be conducted in compliance with the regulations of the Commission;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. This amendment is issued in accordance with the regulations of the Commission as stated in 10 CFR Part 51, and all applicable requirements have been satisfied; and

   F. Prior notice of this amendment was not required by 10 CFR 2.105, and publication of notice for this amendment is not required by 10 CFR 2.106.

2. Accordingly, the license is amended by changes to the following paragraphs which are hereby amended to read as follows:

   1.C. The facility will be decommissioned in conformity with the application as amended, the provisions of the Act, and the rules and regulations of the Commission;

   1.D. There is reasonable assurance that the reactor facility can be decommissioned (i) without endangering the health and safety of the public and (ii) in compliance with the regulations in this chapter;

   1.E. General Atomics is technically and financially qualified to decommission the facility in accordance with the regulations in this chapter;

3. Accordingly, the license is amended by deleting paragraph 2.B.(2) in its entirety.

4. Accordingly, the license is amended to approve the decommissioning plan described in the licensee's application dated April 18, 1997, as supplemented on November 20, 1998, and January 28 and 29, February 3, April 22, May 3 and 12, and June 15 and 22, 1999, and authorizes inclusion of the decommissioning plan as a supplement to the Safety Analysis Report pursuant to 10 CFR 50.82(b)(5). Also, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Amended Facility License No. R-38 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 36, are hereby incorporated in the license. The licensee shall decommission the facility in accordance with the Technical Specifications.

5. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Ledyard B. Marsh, Chief
Events Assessment, Generic Communications and
Non-Power Reactors Branch
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A, Technical
Specifications Changes

Date of Issuance: August 12, 1999

ENCLOSURE TO LICENSE AMENDMENT NO. 36

AMENDED FACILITY LICENSE NO. R-38

DOCKET NO. 50-89

Replace the following pages of Appendix A, "Technical Specifications," with the enclosed pages. The revised pages are identified by amendment number.

Remove	Insert
cover page	cover page
table of contents	i
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TRIGA Reactors Facility

TECHNICAL SPECIFICATIONS
TRIGA MARK I REACTOR

Facility License No. R-38

AUGUST 12, 1999

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APPENDIX A

License No. R-38

TECHNICAL SPECIFICATIONS
FOR
THE TORREY PINES TRIGA MARK I REACTOR

DATE: JANUARY 21, 1972

1.0 DEFINITIONS

1.1 Facility

Facility means the building housing the disassembled reactor plus systems, components, etc. associated with possession and decommissioning of the reactor.

1.2 Facility Secured

Facility secured shall mean that condition where no decommissioning activities are in progress in the facility.

1.3 Facility Operation

Facility operation shall mean any condition wherein the facility is not secured.

1.4 Decommissioning Activities

Decommissioning activities shall mean those activities required to remove the facility safely from service and reduce residual radioactivity to a level that permits the release of the facility to unrestricted use and termination of the license. These activities will involve the handling, surveying, packaging, and dispositioning of radioactive materials in the facility.

1.5 Decommissioning Status

Decommissioning status means that the facility license has been amended to withdraw authorization to operate the reactor and authorizes possession only of the reactor, and that the fuel has been removed from the reactor and is maintained in a safe and secure manner in an approved location outside the Mark I facility.

1.6 Operable

A system or device shall be considered operable when it is capable of performing its intended functions in a normal manner.

1.7 Operating

Operating means a component or system is performing its intended function.

1.8 Abnormal Occurrence

An abnormal occurrence is an observed inadequacy in the implementation of either administrative or procedural controls, such that the inadequacy caused, or could have caused, the existence or development of a condition which resulted, or could have resulted, in a facility condition outside the requirements of the Technical Specifications.

1.9 Measured Value

The measured value is the magnitude of that variable as it appears on the output of a measuring channel.

1.10 Measuring Channel

A measuring channel is the combination of sensor, interconnecting cables or lines, amplifiers, and output device which is connected for the purpose of measuring the value of a variable.

1.11 Channel Calibration

A channel calibration consists of comparing a measured value from the measuring channel with a corresponding known value of the parameter so that the measuring channel output can be adjusted to respond, with acceptable accuracy, to known values of the measured variable.

2.0 SITE

A minimum radius of 120 feet from the approximate center of the TRIGA Reactor Facility building shall define the exclusion area.

3.0 REACTOR BUILDING

Until such time as all decommissioning activities have been completed in the facility, the following ventilation requirements shall apply:

3.1 Controls for the ventilating system shall be located in the facility.

3.2 The reactor room ventilation shall be separate from the rest of the building ventilation.

3.3 During decommissioning activities, tenting or an equivalent confinement barrier shall be placed around the work area and an associated air ventilation HEPA system shall be operating to ensure the work area is maintained at a negative pressure with respect to the surrounding areas and the exhaust pathway to the environment is controlled and monitored.

4.0 REACTOR POOL

4.1 Until drained as a part of decommissioning, the minimum level of the pool water shall be 12 inches below the reference point at the top of the pool. The reference point is marked on a straight edge affixed to the liner at the top of the pool. This point is located 9 1/2 inches from the top flange of the pool liner.

4.2 Until such time as the pool water is drained as a part of decommissioning, the pool water shall be sampled for conductivity at least weekly. The conductivity of the coolant shall not exceed 5 micro-mhos per centimeter averaged over any one calendar month.

5.0 REACTOR FUEL

No reactor fuel elements or control elements with fuel followers shall be located in, stored in, or moved into, the reactor core structure or reactor facility.

6.0 CONTROL AND SAFETY SYSTEMS

6.1 Control Systems

The poison section of the control rods which may remain in the facility contains borated graphite or stainless steel, B_4C powder or boron or its compounds in solid form, cadmium, or the rare earth poisons in solid form clad with the materials authorized for fuel-element cladding. The control rods may contain flux traps and may be equipped with non-fuel followers.

6.2 Safety Systems

Until such time as decommissioning activities are completed in the facility, radiation monitoring systems shall be operating as follows:

a. An area radiation monitoring system capable of activating an evacuation alarm.

b. When the facility is not secured, a continuous monitoring system for airborne activity having a readout and audible alarm which can be heard in the facility.

These systems shall be calibrated annually and their setpoints verified monthly. For periods of time for maintenance or repair to the above systems, or during periods of other forced outages, the intent of this specification shall be satisfied if the installed system(s) is replaced with appropriate alternate or portable radiation monitoring system(s).

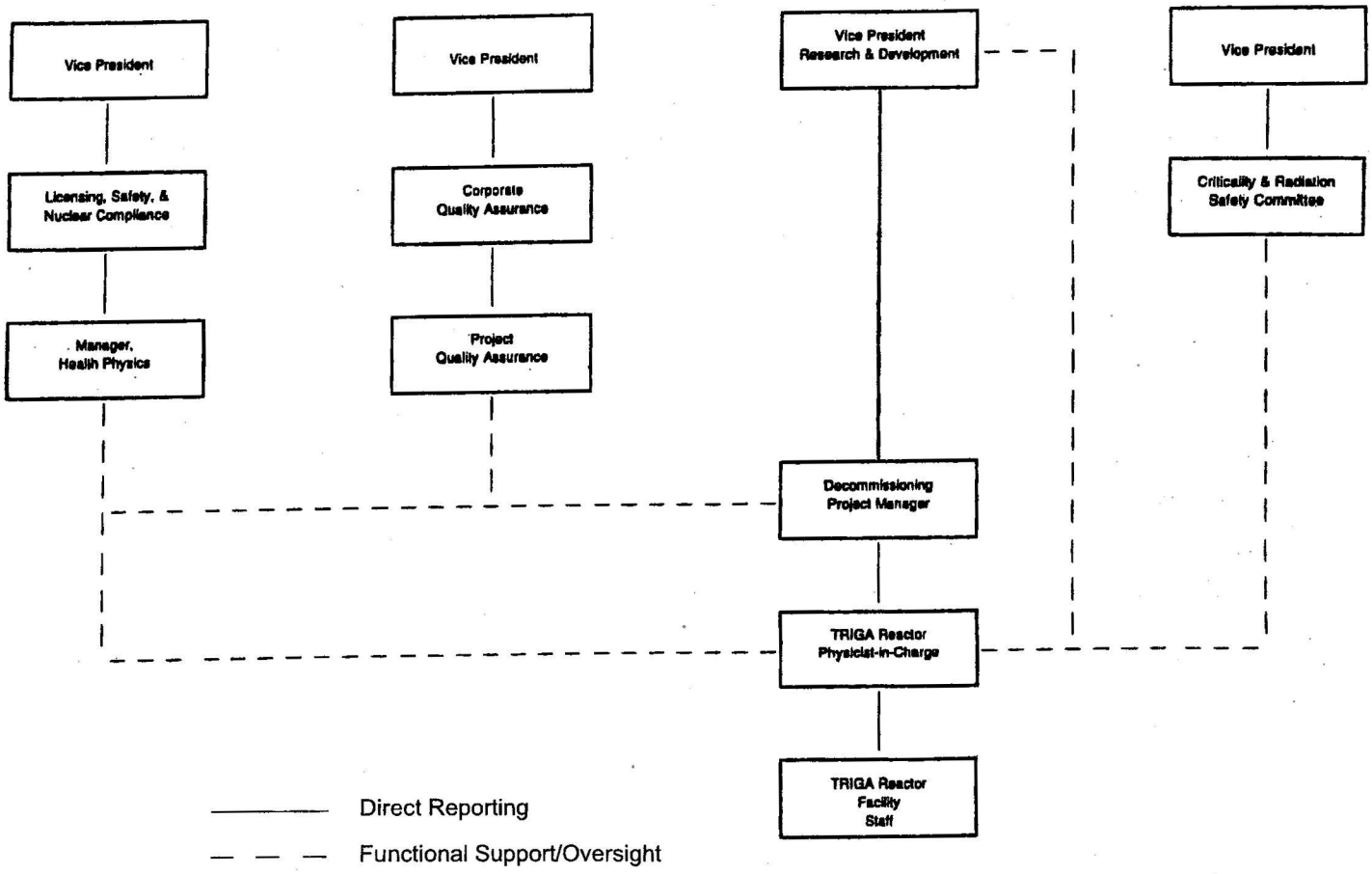
Following maintenance or modification of the radiation monitoring systems, the affected system shall be verified to be operable before returning it to routine operation.

7.0 ADMINISTRATIVE REQUIREMENTS

7.1 Organization

The facility shall be under the supervision of the Physicist-in-Charge or his designated alternate, both of whom shall be licensed senior operators of the facility. The minimum qualification for the Physicist-in-Charge shall include a Bachelor's degree or equivalent academic education and three years experience in activities related to reactor operations. He shall be responsible to a Vice President whose duties include responsibility for research and development, or his designated

alternate, for safe possession of the reactor and its associated equipment and for limiting exposure of personnel and dispersal of radioactive material. The reactor shall be related to the Corporate structure as shown in the following chart:



7.2 Criticality and Radiation Safety Committee

a. There shall be a Criticality and Radiation Safety Committee (CRSC). The CRSC or a subcommittee thereof shall review activities of the facility to assure criticality and radiation safety. This Committee shall be composed of at least four members selected by the Cognizant Vice President, or a designee, considering their experience and education with regard to the various aspects of nuclear physics, chemistry, radiological health and statistics, as well as appropriate experience in other disciplines such as metallurgy and engineering. Subcommittees shall be appointed by the Chairman of the CRSC. The subcommittee assigned the responsibility for reviewing facility operations shall not have thereon person(s) who are directly involved with the facility.

b. The Criticality and Radiation Safety Committee shall be in accordance with a written charter including provisions for:

1. Holding meetings or audits at least annually; special meetings may be called by subcommittees of the CRSC or by the Chairman of the CRSC at times when such meetings are deemed appropriate.

2. Having a quorum when a majority of the members attend.

3. Preparing minutes or audit findings of the CRSC meetings by the Chairman or his designee. Evidence of approval of the participating members of the CRSC shall be obtained before distribution.

c. The Criticality and Radiation Safety Committee, or subcommittee, shall review and approve safety standards associated with possession of the Facility. The CRSC or a subcommittee thereof shall audit the Facility annually, but at intervals not to exceed fifteen (15) months.

d. The responsibilities of the Committee or designated subcommittee thereof include, but are not limited to, the following:

1. Review and approval of all proposed changes to the facility, procedures, and Technical Specifications;

2. Determination of whether a proposed change, test, or experiment would constitute an unreviewed safety question or a change in the Technical Specifications;

3. Review of Facility records;

4. Review of abnormal performance of plant equipment and other anomalies; and

5. Review of unusual or abnormal occurrences and incidents which are Reportable Occurrences under 10 CFR 20 and 10 CFR 50.

7.3 Written Procedures

Written instructions shall be in effect for the following items. The instructions shall be adequate to assure the safe possession of the reactor but shall not preclude the use of independent judgment and action should the situation require such:

a. Testing and calibration of instrumentation, controls, radiation monitors and radiation protection systems necessary to meet the requirements of the Technical Specifications.

b. Emergency and abnormal conditions including provisions for evacuation, reentry, recovery, and medical support.

c. Maintenance of systems necessary for maintaining the reactor in accordance with the license.

d. Actions to be taken to correct specific and foreseen potential malfunctions of systems or components, including responses to alarms and abnormal changes.

e. Decommissioning activities (consistent with an approved Decommissioning Plan).

Changes which alter the original intent of the above procedures shall be made only with the approval of the Criticality and Radiation Safety Committee (CRSC). Temporary changes to the procedures that do not change their original intent and do not constitute an unreviewed safety question (as defined in 10CFR50.59) may be made by the Physicist-in-Charge. All such temporary changes shall be documented and subsequently reviewed and approved by the CRSC.

7.4 Action to be Taken in the Event of an Abnormal Occurrence

In the event of an abnormal occurrence, as defined in Section 1.8 of the Specifications, the following action shall be taken:

a. The Physicist-in-Charge or designated alternate shall be notified and corrective action taken with respect to the operation involved.

b. The Physicist-in-Charge or other person notified under Section 7.4a shall notify the Chairman of the CRSC and the Vice President, or his designated alternate, identified in Section 7.1.

c. A report shall be made to the CRSC which shall include an analysis of the cause of the occurrence, efficacy of corrective action, and recommendations for measures to prevent or reduce the probability of recurrence.

d. A report shall be made to the NRC in accordance with Section 7.6 of these Specifications.

7.5 Facility Records

In addition to the requirements of applicable regulations, and in no way substituting therefore, records and logs shall be prepared of at least the following items and retained for a period of at least five (5) years:

a. Normal reactor operation and facility status;

b. Principal maintenance activities;

c. Abnormal or reportable occurrences;

d. Equipment and component surveillance activities required by the Technical Specifications;

e. Gaseous and liquid radioactive effluents released to the environs;

f. Off-site environmental monitoring surveys;

g. Fuel inventories and transfers;

h. Facility radiation and contamination surveys;

i. Experiments performed with the reactor or in support of decommissioning;

j. Radiation exposures for all personnel;

k. Updated, corrected, and as-built drawings of the facility (these shall be retained for the lifetime of the facility).

l. Decommissioning/dismantlement activities.

7.6 Reporting Requirements

In addition to the requirements of applicable regulations, and in no way substituting therefor, reports shall be made to the NRC as follows:

a. A report within 24 hours by telephone or facsimile to the NRC Operations Center:

1. Any accidental release of radioactivity above permissible limits in unrestricted areas, whether or not the release resulted in property damage, personal injury, or exposure;

2. Any abnormal occurrences as defined in Section 1.8 of these Specifications.

b. A report within 10 days in writing to the Document Control Desk, USNRC , Washington, D.C. 20555:

1. Any accidental release of radioactivity above permissible limits in unrestricted areas, whether or not the release resulted in property damage, personal injury, or exposure; the written report (and, to the extent possible, the preliminary telephone or facsimile report) shall describe, analyze and evaluate safety implications, and outline the corrective measures taken or planned to prevent reoccurrence of the event.

2. Any abnormal occurrence as defined in Section 1.8 of these Specifications.

c. A report within 30 days in writing to the Document Control Desk, USNRC , Washington, D.C. 20555:

1. Any significant changes in facility organization or personnel for which minimum qualifications are specified in these specifications.

d. Commencing in 2000, a routine report in writing to the Document Control Desk, U.S. Nuclear Regulatory Commission by April 15 of each year, providing the following information:

1. A brief narrative summary of: (1) actions taken for decommissioning of the facility, (2) current facility status, (3) any changes to the decommissioning schedule, and (4) a summary of radioactive material shipments.

2. Discussion of the major maintenance operations performed during the period, including the effects, if any, on the safe possession of the reactor, and the reasons for any corrective maintenance required.

3. A summary of each change, if such changes have been made, to the facility, or procedures, tests, and experiments carried out under the conditions of Section 50.59 of CFR Part 50.

4. A summary of the nature and amount of radioactive effluents released or discharged to the environs beyond the effective control of the licensee as measured at or prior to the point of such release or discharge.

5. A description of any environmental surveys performed outside the facility.

6. A summary of radiation exposures received by facility personnel and visitors, including the dates and time of significant exposure, and a brief summary of the results of radiation and contamination surveys performed within the facility.

8.0 FACILITY OPERATIONS

The following activities shall be conducted in the facility:

a) Maintenance, monitoring, control of radioactive material, protection of personnel, and security activities as defined in approved procedures (see Section 7.3).

b) Decommissioning in accordance with the facility license and consistent with an approved Decommissioning Plan.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 29, 1999

Dr. Keith E. Asmussen, Director
Licensing, Safety and Nuclear Compliance
General Atomics
P.O. Box 85608
San Diego, California 92186-9784

SUBJECT: ENVIRONMENTAL ASSESSMENT RELATED TO ISSUANCE OF AMENDMENT NO. 36 TO AMENDED FACILITY LICENSE NO. R-38 AND AMENDMENT NO. 45 TO FACILITY LICENSE NO. R-67 - GENERAL ATOMICS (TAC NOS. M98498 AND M98499)

Dear Dr. Asmussen:

Enclosed is a copy of the Environmental Assessment and Finding of No Significant Impact related to your application for amendment to Amended Facility License No. R-38 for the General Atomics TRIGA Mark I Research Reactor and Facility License No. R-67 for the General Atomics Mark F Research Reactor dated April 18, 1997, as supplemented on November 20, 1998, and January 28 and 29, February 3, April 22, May 3 and 12, and June 15, 16, and 22, 1999. The proposed amendment would approve the decommissioning plan for the General Atomics TRIGA Mark I and Mark F Research Reactors.

The assessment is being forwarded to the Office of the Federal Register for publication.

Sincerely,

Alexander Adams, Jr., Senior Project Manager
Events Assessment, Generic Communications and
Non-Power Reactors Branch
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-89 and
50-163

Enclosure: Environmental Assessment

cc w/enclosure:
See next page

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7590-01-P

UNITED STATES NUCLEAR REGULATORY COMMISSION

GENERAL ATOMICS

DOCKET NOS. 50-89 AND 50-163

GENERAL ATOMICS TRIGA MARK I AND MARK F RESEARCH REACTORS

ENVIRONMENTAL ASSESSMENT AND FINDING OF NO SIGNIFICANT IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of a license amendment to Amended Facility License No. R-38 and Facility License No. R-67, issued to General Atomics (GA or the licensee), for decommissioning of the GA TRIGA Mark I and TRIGA Mark F Research Reactors, located at General Atomics in San Diego, San Diego county, California.

ENVIRONMENTAL ASSESSMENT

Identification of Proposed Action:

The proposed action would approve the licensee's decommissioning plan. GA submitted their decommissioning plan in accordance with 10 CFR 50.82(b) for the GA TRIGA Mark I and TRIGA Mark F Research Reactors which occupy parts of the TRIGA Reactor Facility within GA's Torrey Mesa site. The TRIGA Mark I license was amended on October 29, 1997, and the TRIGA Mark F license was amended on March 22, 1995, to remove authority to operate the reactors. Fuel from both reactors have been placed in the TRIGA Mark F fuel storage canal which is in the same pool as the TRIGA Mark F reactor. The proposed decommissioning plan would authorize immediate dismantlement of the TRIGA Mark I Research Reactor. To protect the stored fuel from potential damage due to

decommissioning activities, only limited dismantlement of the TRIGA Mark F Research Reactor will occur with fuel in the TRIGA Mark F fuel storage canal. This would be followed by a period of fuel storage. After fuel is removed from the TRIGA Mark F fuel storage canal, dismantling will be completed on the TRIGA Mark F Research Reactor. The soonest that the Department of Energy can accept fuel from GA is 2003. Domestic spent nuclear fuel receipts at the Idaho National Engineering and Environmental Laboratory have been severely constrained because of a settlement agreement of a lawsuit concerning spent nuclear fuel and nuclear waste. The site will be decontaminated to meet unrestricted release criteria. After the Commission verifies that the release criteria have been met, the reactor license will be terminated.

The licensee will continue with their health physics program, and approved emergency and security plan during the decommissioning and their operator requalification plan until fuel is removed from the facility.

A "Notice of Application for Decommissioning Amendment" was published in the FEDERAL REGISTER on December 11, 1997 (62 FR 65288), in accordance with the requirements of 10 CFR 50.82(b)(5).

The proposed action is in accordance with the licensee's application for amendment dated April 18, 1997, as supplemented on November 20, 1998, and January 28 and 29, February 3, April 22, May 3 and 12, and June 15, 16, and 22, 1999.

The Need for the Proposed Action

The proposed action is needed because of GA's decision to cease reactor operations permanently at the Torrey Mesa site. As specified in 10 CFR 50.82, any licensee may apply to the NRC for authority to surrender a license voluntarily and to

decommission the affected facility. Once the licensee permanently ceases operation, 10 CFR 50.82(b)(1) requires the licensee to make application for license termination within two years following permanent cessation of operations, and in no case later than one year prior to expiration of the operating license. GA is planning to use the area that would be released for unrestricted use for other purposes.

Environmental Impact of the Proposed Action:

The Commission has completed its evaluation of the proposed action and concludes that the radiological effects of decommissioning the TRIGA Mark I and Mark F Research Reactors will be minimal. The licensee will continue with their health physics program, and approved emergency and security plans. Until fuel is removed from the site, the licensee will also continue to meet the requirements of their operator requalification plan.

All proposed operations in connection with decommissioning and decontaminating of the GA reactors will be carefully planned and controlled, all contaminated components will be removed, packaged, and shipped offsite in accordance with the regulations, and radiological control procedures will be in place and implemented to ensure that releases of radioactive wastes from the facility are within the limits of 10 CFR Part 20 and are as low as reasonably achievable (ALARA).

All decontamination will be performed by trained personnel in accordance with previously reviewed procedures and will be overseen by experienced health physics staff. No new postulated accidents have been identified during decommissioning activities or storage of the reactor fuel that would have greater radiological impact than previously evaluated accidents. The GA staff has calculated that the total dose to workers for the

decommissioning project will be about 20 person-rem over the period 1999 to 2004 (assuming fuel is removed from the facility in 2003). The GA staff estimates that the dose to members of the public from decommissioning activities will be negligible. These doses are consistent with those given in NUREG-0586, "Final Generic Environmental Impact Statement on Decommissioning of Nuclear Facilities," for the reference research reactor.

While on site, fuel will be stored in approved storage locations under the restrictions of the facility license. The license will continue to maintain systems necessary for safe storage of the fuel.

The proposed action will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not involve any historic sites. Hazardous materials such as lead and asbestos will be handled and disposed of in accordance with all applicable regulations and, therefore, will not result in any significant release of non-radiological plant effluents and has no other environmental impact. Therefore, there are no significant non-radiological environmental impacts associated with the proposed action.

Accordingly, the Commission concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action:

The alternatives to the proposed action for the GA TRIGA research reactors are SAFSTOR, ENTOMB and no action. ENTOMB is the alternative in which radioactive contaminates are encased in a structurally long-lived material, such as concrete, the entombed structure is appropriately maintained and continued surveillance is carried out until the radioactivity decays to a level permitting release of the property for unrestricted use. SAFSTOR is the alternative in which the facility is placed and maintained in a condition that allows the facility to be safely stored and subsequently decontaminated to levels that permit release for unrestricted use.

The ENTOMB alterative could not be put into place until the fuel was removed from the facility and would require the facility to remain on site for an extended period of time. Likewise, the SAFSTOR alternative would require continued surveillance for an extended period of time. However, GA wants to use the space that will become available for other purposes and wants to enter into the decommissioning activities as soon as possible. The alternative of not decommissioning reactors was rejected in NUREG-0586. The no action alternative would leave the facility in its present configuration. Denial of the application would result in no significant change in current environmental impacts.

The environmental impacts of the proposed action and the alternative actions are similar.

Alternative Use of Resources:

The action does not involve the use of resources different from previously committed for construction and operation of the GA TRIGA reactors.

Agencies and Persons Consulted:

In accordance with its stated policy, on July 20, 1999, the staff consulted with the State of California official, R. Lupo of the Radiologic Health Branch of the California Department of Health Services regarding the environmental impact of the proposed action. The state official had no comments.

FINDING OF NO SIGNIFICANT IMPACT

On the basis of the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to this proposed action, see the licensee's letter dated April 18, 1997, as supplemented by letter dated November 20, 1998, and January 28 and 29, February 3, April 22, May 3 and 12, and June 15, 16, and 22, 1999. These documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, D.C. 20003-1527.

Dated at Rockville, Maryland, this 29th day of July 1999.

FOR THE NUCLEAR REGULATORY COMMISSION

L B Marsh

Ledyard B. Marsh, Chief
Events Assessment, Generic Communications and
Non-Power Reactors Branch
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation