

**POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 (PBN)  
SUBSEQUENT LICENSE RENEWAL APPLICATION (SLRA)  
DRAFT REQUEST FOR ADDITIONAL INFORMATION (RAI)  
SAFETY - SET 4**

Section 3.3.2.2.8, “Cracking Due to Stress Corrosion Cracking in Aluminum Alloys”

DRAI 3.3.2.2.8-1

Regulatory Basis

10 CFR 54.21(a)(3) requires an applicant to demonstrate that the effects of aging for structures and components will be adequately managed so that the intended function(s) will be maintained consistent with the current licensing basis for the period of extended operation. One of the findings that the NRC staff must make to issue a renewed license (10 CFR 54.29(a)) is that actions have been identified and have been or will be taken with respect to managing the effects of aging during the period of extended operation on the functionality of structures and components that have been identified to require review under 10 CFR 54.21, such that there is reasonable assurance that the activities authorized by the renewed license will continue to be conducted in accordance with the current licensing basis. In order to complete its review and enable it to make a finding under 10 CFR 54.29(a), the staff requires additional information in regard to the matters described below.

Background

During the review of SLRA Table 3.3-1, “Summary of Aging Management Evaluations for the Auxiliary Systems,” the staff determined that Table 3.3-1 omits Item Number 3.3-1, 233 for cracking due to stress corrosion cracking for insulated aluminum piping, piping components, and tanks in air or condensation. Item Number 3.3-1, 233 requires further evaluation according to SLR-SRP Section 3.3.2.2.8.

Issue

Because Item Number 3.3-1, 233 was omitted from Table 3.3-1, the SLRA does not include a discussion of aging management for this item.

NRC Request

Please provide the SLRA Table 3.3-1 entry and disposition for Item Number 3.3-1, 233.