



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**

REGION I  
2100 RENAISSANCE BOULEVARD, SUITE 100  
KING OF PRUSSIA, PENNSYLVANIA 19406-2713

August 3, 2021

Mr. David Rhoades  
Senior Vice President  
Exelon Generation Company, LLC  
President and Chief Nuclear Officer  
Exelon Nuclear  
4300 Winfield Road  
Warrenville, IL 60555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – INTEGRATED  
INSPECTION REPORT 05000333/2021002

Dear Mr. Rhoades:

On June 30, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at James A. FitzPatrick Nuclear Power Plant. On July 22, 2021, the NRC inspectors discussed the results of this inspection with Mr. Tim Peter, Site Vice President, and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Erin E. Carfang, Chief  
Projects Branch 1  
Division of Operating Reactor Safety

Docket No. 05000333  
License No. DPR-59

Enclosure:  
As stated

cc w/ encl: Distribution via LISTSERV®

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – INTEGRATED INSPECTION REPORT 05000333/2021002 DATED AUGUST 3, 2021

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**U.S. NUCLEAR REGULATORY COMMISSION  
Inspection Report**

Docket Number: 05000333

License Number: DPR-59

Report Number: 05000333/2021002

Enterprise Identifier: I-2021-002-0043

Licensee: Exelon Nuclear

Facility: James A. FitzPatrick Nuclear Power Plant

Location: Oswego, NY

Inspection Dates: April 01, 2021 to June 30, 2021

Inspectors: P. Boguszewski, Acting Senior Resident Inspector  
J. England, Resident Inspector  
H. Anagnostopoulos, Senior Health Physicist  
E. Dipaolo, Senior Reactor Inspector  
J. Dolecki, Resident Inspector  
M. Henrion, Health Physicist  
C. Hobbs, Reactor Inspector  
E. Miller, Senior Resident Inspector  
A. Patel, Senior Reactor Inspector  
B. Pinson, Reactor Inspector  
C. Swisher, Resident Inspector

Approved By: Erin E. Carfang, Chief  
Projects Branch 1  
Division of Operating Reactor Safety

Enclosure

## **SUMMARY**

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at James A. FitzPatrick Nuclear Power Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

### **List of Findings and Violations**

No findings or violations of more than minor significance were identified.

### **Additional Tracking Items**

None.

## PLANT STATUS

FitzPatrick started the inspection period at rated thermal power. On June 25, 2021 reactor power was lowered to 70 percent for routine testing and a rod sequence change. Power was restored to 100 percent on June 26, 2021. FitzPatrick remained at or near rated thermal power for the remainder of the inspection period.

## INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), resident and regional inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time, the resident inspectors performed periodic site visits each week, increasing the amount of time on site as local COVID-19 conditions permitted. As part of their onsite activities, resident inspectors conducted plant status activities as described in IMC 2515, Appendix D; observed risk significant activities; and completed on site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or a portion of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on site. The inspections documented below met the objectives and requirements for completion of the IP.

## REACTOR SAFETY

### 71111.01 - Adverse Weather Protection

#### Seasonal Extreme Weather Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated readiness for seasonal extreme weather conditions prior to the onset of seasonal hot temperatures for the switchyard and the 'A' and 'B' 125VDC battery rooms on June 2, 2021.

### 71111.04 - Equipment Alignment

#### Partial Walkdown Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) 'A' emergency service water on April 15, 2021
- (2) 4160-volt and 400-volt normal alternating current power distribution on April 26, 2021

- (3) 'A' and 'C' emergency diesel generator (EDG) during 'B' and 'D' EDG testing on May 24, 2021
- (4) High pressure coolant injection system following governor valve position indication repair on May 26, 2021
- (5) Primary containment venting on June 15, 2021
- (6) 'B' and 'D' residual heat removal (RHR) system during emergency Technical Specification amendment on June 21, 2021

Complete Walkdown Sample (IP Section 03.02) (1 Sample)

- (1) Control rod drive system on April 8, 2021

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) Main control room, control room HVAC room, elevation 300', and relay room 286', fire area VII/CR-1 and RR-1, on April 1, 2021
- (2) Reactor building 272' elevation, fire area X/RB-1B and IX/RB-1A, on April 6, 2021
- (3) Site plan, fire area yard, on April 19, 2021
- (4) ISFSI pad and switch yard, 345-kilovolt and 115-kilovolt yards, fire area yard, on April 20, 2021
- (5) East electric fire bay, fire area/zone II/SW-2 on May 24, 2021
- (6) West electric fire bay, fire area/zone IC/SW-1 on May 24, 2021

71111.11A - Licensed Operator Requalification Program and Licensed Operator Performance

Requalification Examination Results (IP Section 03.03) (1 Sample)

- (1) The inspectors reviewed and evaluated the licensed operator requalification examination exam results for the annual operating exam and biennial written exam on June 29, 2021.

71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

- (1) The inspectors observed reactivity operations to lower power to 70 percent for a control rod sequence exchange, control rod scram time testing, and turbine valve testing on June 25 - June 26, 2021.

Licensed Operator Requalification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated operator performance in the simulator which included job performance measures to place the high pressure coolant injection system in pressure control operation, and reactor pressure vessel isolation on May 6, 2021

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (2 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) Core spray system on April 30, 2021
- (2) Structural monitoring walkdown of the torus room on May 19, 2021

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (7 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) Elevated risk (Action Green) during scheduled maintenance activities on 'C' EDG, week of April 19-22, 2021
- (2) Elevated risk (Action Green) during planned maintenance on 71T-3 115 transformer on May 11, 2021
- (3) Emergent risk associated with electro-hydraulic control turbine intercept valve movement on May 19, 2021
- (4) Unplanned loss of 'A' reactor water recirculation speed control circuit on June 4, 2021
- (5) 'B' RHR system during 'A' RHR planned maintenance on June 8, 2021
- (6) Emergency license amendment protected equipment during 'A' RHR pump maintenance on June 13, 2021
- (7) Fire risk protection due to 'A' RHR system unavailability exceeding 60 hours on June 14, 2021

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (3 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) Control room emergency ventilation air supply system due to degraded 'B' ventilation supply fan, 70FN-6B, on April 2, 2021
- (2) Emergency service water due to through-wall leakage on 'B' piping on April 13, 2021
- (3) Control room emergency ventilation air system due to a degraded relay associated with 70FN-6B on May 10, 2021

### 71111.18 - Plant Modifications

#### Severe Accident Management Guidelines (SAMG) Update (IP Section 03.03) (1 Sample)

- (1) Revision to Severe Accident Management Guidelines

### 71111.19 - Post-Maintenance Testing

#### Post-Maintenance Test Sample (IP Section 03.01) (10 Samples)

The inspectors evaluated the following post-maintenance test activities to verify system operability and functionality:

- (1) 'A' low pressure coolant injection inverter work on April 13, 2021
- (2) 76P-2, electric-driven fire pump following pump replacement on April 16, 2021
- (3) 'A' EDG following 'C' EDG scheduled maintenance activities on April 22, 2021
- (4) 'B' EDG and 'D' EDG Force Parallel Logic Relay (93TD9M-1EDGB13) on May 4, 2021
- (5) 'B' EDG following turbocharger replacement on May 6, 2021
- (6) High pressure coolant injection system following governor valve position indication repair on May 26, 2021
- (7) 'D' RHR service water pump breaker swap on June 2, 2021
- (8) 'A' RHR pump after failed Baker testing on June 8, 2021
- (9) 'A' RHR pump following motor replacement on June 17, 2021
- (10) Control rod 26-35 following scram solenoid pilot valve replacement on June 26, 2021

### 71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

#### Surveillance Tests (other) (IP Section 03.01) (1 Sample)

- (1) ST-9BB, EDG B and D Full Load Test and ESW Pump Operability Test, on April 2, 2021

#### Inservice Testing (IP Section 03.01) (4 Samples)

- (1) ST-3PB, Core Spray Loop B Quarterly Operability Test, on June 4, 2021
- (2) HPCI system review for Emergency License Amendment Review, on June 12, 2021
- (3) 'B' and 'D' residual heat removal system for Emergency License Amendment Review, on June 12, 2021
- (4) 'B' and 'D' emergency diesel generators for Emergency License Amendment Review, on June 12, 2021



## **RADIATION SAFETY**

### 71124.01 - Radiological Hazard Assessment and Exposure Controls

#### Radiological Hazard Assessment (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated how the licensee identifies the magnitude and extent of radiation levels and the concentrations and quantities of radioactive materials, and how the licensee assesses radiological hazards.

#### Instructions to Workers (IP Section 03.02) (1 Sample)

The inspectors evaluated instructions to workers including radiation work permits used to access high radiation areas.

- (1) The inspectors reviewed the following:

##### Radiation Work Packages

- RWP JF-1-21-00302 TIP Area Activities
- RWP JF-1-21-00202 Radwaste Activities
- RWP JF-1-21-00209 Transportation of Radioactive Material

##### Electronic Alarming Dosimeter Alarms

- EAD Alarm on May 18, 2020
- EAD Alarm on September 19, 2020
- EAD Alarm on September 21, 2020

##### Labeling of Containers

- LSA trash bins
- Bagged laundry
- Bagged portable vacuums

#### Contamination and Radioactive Material Control (IP Section 03.03) (3 Samples)

The inspectors evaluated licensee processes for monitoring and controlling contamination and radioactive material. The inspectors:

- (1) Observed licensee survey of tools, filters, and meters for release from the radiologically controlled area
- (2) Observed workers exiting radiologically controlled area and using the small articles monitor at the main access point
- (3) Inspected radwaste packages that are stored and awaiting shipment

#### Radiological Hazards Control and Work Coverage (IP Section 03.04) (3 Samples)

The inspectors evaluated in-plant radiological conditions during facility walkdowns and observation of the following radiological work activities:

- (1) Replacement of 12P-1A RWCU pump in January 2021
- (2) Testing and maintenance of 'A' residual heat removal motor-operated valves

- (3) Replacement of 'A' feed pump cooling coils

High Radiation Area and Very High Radiation Area Controls (IP Section 03.05) (4 Samples)

The inspectors evaluated licensee controls of the following High Radiation Areas and Very High Radiation Areas:

- (1) Reactor building decontamination room
- (2) Reactor traversing in-core probe
- (3) Fuel pool cooling
- (4) Reactor water cleanup hold pump room

Radiation Worker Performance and Radiation Protection Technician Proficiency (IP Section 03.06) (1 Sample)

- (1) The inspectors evaluated radiation worker and radiation protection technician performance as it pertains to radiation protection requirements.

71124.03 - In-Plant Airborne Radioactivity Control and Mitigation

Permanent Ventilation Systems (IP Section 03.01) (2 Samples)

The inspectors evaluated the configuration of the following permanently installed ventilation systems:

- (1) Reactor building ventilation system
- (2) Control room ventilation system

Temporary Ventilation Systems (IP Section 03.02) (2 Samples)

The inspectors evaluated the configuration of the following temporary ventilation systems:

- (1) Decontamination booth in radwaste
- (2) Reactor building decontamination room

Use of Respiratory Protection Devices (IP Section 03.03) (1 Sample)

- (1) The inspectors evaluated the licensee's use of respiratory protection devices.

Self-Contained Breathing Apparatus for Emergency Use (IP Section 03.04) (1 Sample)

- (1) The inspectors evaluated the licensee's use and maintenance of self-contained breathing apparatuses.

**OTHER ACTIVITIES – BASELINE**

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

BI01: Reactor Coolant System (RCS) Specific Activity Sample (IP Section 02.10) (1 Sample)

- (1) January 1, 2020 through December 31, 2020

BI02: RCS Leak Rate Sample (IP Section 02.11) (1 Sample)

- (1) April 1, 2020 through March 31, 2021

71152 - Problem Identification and Resolution

Semiannual Trend Review (IP Section 02.02) (1 Sample)

- (1) The inspectors reviewed the licensee's corrective action program for potential adverse trends in that might be indicative of a more significant safety issue.

Annual Follow-up of Selected Issues (IP Section 02.03) (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issue:

- (1) IR 4267056 - Spent Fuel Pool Thermal Dispersion Criteria Not Maintained

71153 - Follow Up of Events and Notices of Enforcement Discretion

Personnel Performance (IP Section 03.03) (1 Sample)

- (1) Half scram following blown fuse on reactor protection system channel A1 during response time testing of master trip unit 05MTU-212A on May 18, 2021

**INSPECTION RESULTS**

Minor Violation	71152
Minor Violation: On July 25, 2019, Exelon Nuclear Oversight conducted an audit at FitzPatrick and discovered that fuel assemblies discharged into the spent fuel pool (SFP) at the end of the last refueling outage (J1R23), which ended in October 2018, had not conformed to the B.5.b thermal dispersion pattern committed to by FitzPatrick. Specifically, four fuel bundles discharged into the northeast corner of the SFP did not meet the separation criteria specified in NF-AB-309, "BWR Special Nuclear Material and Core Component Move Sheet Development," Attachment 8, which describes a Knight's Move, 6-cell pattern, similar to the way the knight piece moves in a game of chess. In this pattern, recently discharged fuel bundles that have been out of the core for less than 275 days do not share an adjacent location in the SFP with another fuel bundle that has also been discharged from the reactor core for less than 275 days. This arrangement of recently discharged fuel bundles in the SFP is a mitigation strategy to cope with the requirements of Section B.5.b of Commission Order EA-02-026 for protection against loss of large areas of the plant due to fire or explosion, and is codified in Title 10 of the <i>Code of Federal Regulations</i> (10 CFR) 50.155 (b)(2), "Mitigation of Beyond-Design-Basis Events."	
An Extent of Condition analysis was conducted in IR 04267056, and it was determined that the software program utilized for planning and executing fuel moves in the SFP had not correctly modeled the dimensions between racks in the SFP in the northeast corner of the	

pool. This was a latent error that had existed since FitzPatrick first began using the ShuffleWorks software for planning fuel moves in the 2004 through 2006 timeframe. On August 23, 2007, FitzPatrick adopted a License Amendment committing to the offloading of fuel assemblies into the Knight's Move B.5.b dispersal pattern in the SFP. Exelon determined that the four fuel assemblies identified on July 25, 2019, had already been in the SFP for greater than 275 days, therefore, it was not necessary to move the fuel into a new location in the pool. Additional corrective actions taken by Exelon included a review of FitzPatrick refueling procedures, an update to the ShuffleWorks fuel movement software, and fleet actions to have each Exelon plant perform a check to determine if the fuel movement software at that facility modeled the actual physical rack dimensions in the SFP at that facility. For the review of FitzPatrick refueling procedures, Exelon determined that adequate steps already existed in OS-66.001, "Management of Refueling Activities," and NF-AB-309, "BWR Special Nuclear Material and Core Component Move Sheet Development" to describe the B.5.b fuel thermal dispersion pattern, and to incorporate this strategy into reactor core offloads during refueling outages. Therefore, Exelon determined that no procedural changes were necessary. The ShuffleWorks program at FitzPatrick was modified to place a constraint violation on the locations in the SFP that do not meet the B.5.b thermal dispersion distance separation requirements, so that a warning box appears if a fuel assembly is placed in one of these locations during fuel movement planning operations. The inspectors conducted interviews with reactor engineers, and reviewed the corrective actions taken by Exelon in response to the B.5.b thermal dispersion issue identified on July 25, 2019, and determined they were adequate.

Corrective Action Reference: IR 04267056

Screening: The inspectors determined the performance deficiency was minor. The inspectors determined that the failure to comply with the B.5.b thermal dispersion pattern described in NF-AB-309, Attachment 8 was a performance deficiency. Specifically, NF-AB-309, "BWR Special Nuclear Material and Core Component Move Sheet Development," Revision 3, Step 4.4.9 requires that fuel move sheets are generated using approved and controlled move planning software, that ensures the required prerequisites and recommended constraints are followed, and that the move planning software is up-to-date. The inspectors determined that the performance deficiency was minor due to the low number of fuel assemblies impacted, and that the small dimensional discrepancy between assemblies would not have a significant effect on the heat removal capability of the SFP during a postulated Beyond Design Basis scenario.

Enforcement: FitzPatrick License Condition 2R.(b)(7), "Mitigation Strategy License Condition," requires in part that SFP mitigation measures are developed and maintained to address large fires and explosions. Contrary to the above, from August 23, 2007, the date the License Condition was implemented, until July 25, 2019, fuel assemblies loaded into the northeast corner of the SFP from the reactor core in every other refueling outage, were not in compliance with the B.5.b thermal dispersion separation requirements specified in NF-AB-309, "BWR Special Nuclear Material and Core Component Move Sheet Development." Following identification of the issue, Exelon took corrective actions to restore compliance. This failure to comply with License Condition 2R.(b)(7) constitutes a minor violation that is not subject to enforcement action in accordance with the NRC's Enforcement Policy.

Observation: Semi-Annual Trend	71152
<p>The inspectors evaluated a sample of issues and events that occurred over the first and second quarters of 2021. The evaluation did not reveal any new trends that could indicate a more significant safety issue. The inspectors determined that, in most cases, the issues were appropriately evaluated by Exelon staff for potential trends at a low threshold, and resolved within the scope of the corrective action program. The inspectors identified a trend associated with additional equipment failure that resulted in additional monitoring and unplanned maintenance, which included the 'A' reactor water cleanup pump seal failure, an immersion heater failure associated with the 'C' emergency diesel generator, unexpected movement of the turbine intercept valves, and failure of the 'A' residual heat removal pump motor. The inspectors also identified a trend with increasing failure of emergency battery lights.</p> <p>Based on the overall results of the semi-annual trend review, the inspectors determined that Exelon had generally identified adverse trends at FitzPatrick before they could become more significant safety problems. The inspectors independently evaluated the deficiencies noted above for significance in accordance with the guidance in IMC 0612, Appendix B, "Issue Screening," and Appendix E, "Examples of Minor Issues," and determined them to be minor.</p>	

### **EXIT MEETINGS AND DEBRIEFS**

The inspectors verified no proprietary information was retained or documented in this report.

- On July 22, 2021, the inspectors presented the integrated inspection results to Mr. Tim Peter, Site Vice President, and other members of the licensee staff.
- On June 10, 2021, the inspectors presented the radiological hazard assessment and exposure controls inspection results to Mr. Mohamed Khan, Director, Site Engineering, and other members of the licensee staff.
- On June 10, 2021, the inspectors presented the spent fuel pool inspection results to Mr. Philip Nichols, Reactor Engineering Manager, and other members of the licensee staff.

### **THIRD PARTY REVIEWS**

Inspectors reviewed Institute of Nuclear Power Operations reports that were issued during the inspection period for the evaluation conducted in February 2021.

## DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date	
71111.01	Procedures	WC-JF-107-1000	Seasonal Readiness T&RM for JAF	3	
	Work Orders	05042019			
71111.04	Drawings	FE-1B	Main One line Diagram Sheet 2 Station Service Transformers	19	
		FM-20A	Flow Diagram Residual Heat Removal System 10	73	
		FM-27A	Flow Diagram Control Rod Drive System 03	39	
		FM-46A	Flow Diagram Service Water System 46	94	
		FM-46B	Flow Diagram Emergency Service Water System 46 & 15	57	
		ISI-FM-27B	ISI Drawing	13	
	Procedures	OP-21	Emergency Service Water (ESW)	41	
		OP-22	Diesel Generator Emergency Power	71	
		OP-25	Control Rod Drive Hydraulic System	93	
		OP-46A	4160V and 600V Normal AC Power Distribution	71	
		TSG-9	Primary Containment Venting Without AC Power	5	
71111.05	Corrective Action Documents Resulting from Inspection	04414983			
		04417967			
		04417970			
		04417972			
		04417989			
	Fire Plans			Site Plan	5
		FPP-3.56		Portable Fire Extinguisher Inspection Procedure	6
		PFP-OUT39		ISFSI Pad Location, Elevation 272' Fire Area/Zone/Yard	3
		PFP-PWR12		Relay Room/Elev. 286' Fire Area/Zone Vii/RR-1	5
		PFP-PWR13		Main Control Room and Control Room HVAC Equipment Rooms/Elev. 300' Fire Area/Zone VII/CR-1	8
		PFP-PWR20		Reactor Building - East/Elev. 272' Fire Area/Zone IX/RB-1A	05
		PFP-PWR21		Reactor Building - West/Elev. 272' Fire Area/Zone X/RB-1B	05
		PFP-PWR29		Switchgear Room East, Elevation 272', Fire Area 2/Fire Zone SW-2	4

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		PFP-PWR51	Switch Yard, 345 and 115 kV Yards, Elevation 272'0" Fire Area Yard/Fire Zone Yard	0
		PFP-PWR30	Switchgear Room West Elevation 272', Fire Area/Fire Zone IC/SW-1	2
	Work Orders	05099562		
71111.11Q	Procedures	OP-AB-300-1003	BWR Reactivity Maneuver Guidance	18
		RAP-7.4.01	Control Rod Scram Time Evaluation	31
		ST-1L	Main Turbine Control Valve Instrument Channel and Valve Operability Check	37
71111.12	Corrective Action Documents	04404301		
		04390733		
		04394058		
		04404883		
		04404967		
		04412087		
		04416039		
	Corrective Action Documents Resulting from Inspection	04424860		
		04424863		
		04424885		
		04424886		
		04424889		
		04424891		
		04424960		
		04424973		
	Engineering Evaluations	JAF-RPT-07-00006	Maintenance Rule Structural Monitoring Report	006
		JAF-RPT-07-00006	Maintenance Rule Structural Monitoring Report	005
Miscellaneous	Maintenance Rule System Basis Document, System 14	Core Spray System		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Procedures	ST-9BB	EDG B and D Full Load Test and ESW Pump Operability Test	Revision 19
71111.13	Corrective Action Documents	04422793		
		04424128		
		04427658		
	Corrective Action Documents Resulting from Inspection	04423185		
	Drawings	1.11-466	Load Control Unit	C
		1.11-472	Intercept: Valve Pre-Amp	
		1.11-545	Schematic Wiring Diagram Valve Test Logic IV-1 & IV-3	
		1.11-588	Schematic Wiring Diagram Intercept Valve No. 1 Position Unit	B
		FM-31A	Flow Diagram Extraction Steam System 31	25
		FM-31B	Flow Diagram Moisture Separator Reheater Drain System 31	52
		S94-003.cdr	EHC Hydraulics	0
	Miscellaneous	James A. FitzPatrick Technical Support Guidelines, TSGS		1
	Procedures	AP-16.14	Hazard Barrier Controls	8
		EOP-PC	Hot Primary Containment Control	0
		EP-6	Post Accident Containment Venting and Gas Control	012
		JF-CRM-004	Development of Risk Management Actions for the Inclusion of Fire Insights into James A. FitzPatrick Configuration Risk Management Program	1
		JF-PRA-005.15	JAF PRA Emergency Diesel Generators System Notebook	0
MP-093.05		EDG Power Assembly Maintenance	14	
MP-093.11		EDG System Mechanical PM	55	



Inspection Procedure	Type	Designation	Description or Title	Revision or Date	
		OP-AA-108-111	Adverse Condition Monitoring and Contingency Plan	15	
		OP-AA-108-117	Protected Equipment Program	006	
		OP-AA-108-117	Protected Equipment Program	6	
		OP-AA-201-012-1001	Operations On-Line Fire Risk Management	4	
		Severe Accident Operating Guideline (SAOG)-2	RPV, Containment, and Radioactive Release Control (1-4)	5	
		TSG-9	Primary Containment Venting without AC Power	5	
71111.15	Work Orders	05158736			
		Corrective Action Documents	04412368		
			04412368		
			04416041		
			04416224		
			04416742		
	04416743				
	Procedures	ESK-6FM	HVAC Emergency Air Supply FN-6A and 6B Control Room	12	
ISP-85		Control Room Ventilation Temperature and Differential Pressure Instrument Calibration	26		
71111.18	Miscellaneous		Reactor Oversight Process Changes to Address Severe Accident Management Guidelines	February 23, 2016 (ML16032A029)	
		JAFP-15-0145	Commitment to Maintain Severe Accident Management Guidelines	December 15, 2015	
		NEI 14-01	Emergency Response Procedures and Guidelines for Beyond Design Basis Events and Severe Accidents		
		TSGS	James A. FitzPatrick Nuclear Power Plant Technical Support Guidelines	1	
	Procedures	SAOG-1	RPV and Primary Containment Injection Control (1-4)	1	
		SAOG-2	RPV, Containment, and Radioactivity Release Control (1-4)	5	
		SAOG-3	RPV Controls	5	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		SAOG-4	Containment and Radioactivity Release Control (5)	0
71111.19	Corrective Action Documents	04418483		
	Miscellaneous	ST-76B	Electric Fire Pump 76P-2 Operational Check	March 18, 2021
		ST-76J24	Electric Fire Pump 76P-2 Performance Test	March 18, 2021
	Procedures	MA-AA-723-330	Electrical Testing of AC Motors using Baker Instrument Advanced Winding Analyzer	6
		MP-054.05	4.16 kV Magne-Blast Breaker Swap/Replacing	03
		ST-16GA	A LPCI MOV Independent Power Supply Test	005
		ST-2AL	RHR Loop A Quarterly Operability Test (IST)*	042
		ST-9BA	EDG A and C Full Load Test and ESW Pump Operability Test	20
	Work Orders	04832790		
		04879904-47		
		04932365		
		05125609		
		05129031		
		05135425		
		4915588-02		
5026005				
71111.20	Miscellaneous	NFPA 20-1970	Standard for the Installation of Centrifugal Fire Pumps	1970
71111.22	Corrective Action Documents	04372229		
		04377749		
	Procedures	ST-3PB	Core Spray Loop B Quarterly Operability Test	31
		ST-43I	Remote Shutdown Instrument Check	
		ST-9BB	EDG B and D Full Load Test and ESW Pump Operability Test	19
		ST-9CB	EDG B and D Load Sequencing Test and 4kV Emergency Power System Voltage Relays Instrument Functional Test	6
		ST-9QB	EDG B and D Full Load Test (8 Hour Run)	0
	Work Orders	4684948		
4886078				
4973154				

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		5047948		
		5100611		
		5123482		
		5125380		
		5130230-01		
		5141387		
		5148131		
		51568506		
71151	Procedures	ST-40D	Daily Surveillance and Channel Check	124
71152	Calculations	JAF-RPT-08-00013	GNF2 Fuel Design Cycle Independent Analysis for Entergy FitzPatrick	1
	Corrective Action Documents	04155271		
		04174769		
		04267056		
		04277344		
		04355560		
	Corrective Action Documents Resulting from Inspection	04428127		
	Drawings	Drawing No. 3.52-73	Pool Layout - Spent Fuel Pool Storage Racks	1
		UFSAR Figure No. 9.3-1	Arrangement of Aluminum High Density Racks	4
		UFSAR Figure No. 9.3-6	Arrangement of Spent Fuel Racks with Stainless Steel High Density Racks	2
	Miscellaneous	DPR-59	FitzPatrick Renewed Facility Operating License No. DPR-59	
NRC Information Notice 2014-09		Spent Fuel Storage or Transportation System Misloading		
NRC Information Notice 2014-14		Potential Safety Enhancements to Spent Fuel Pool Storage		
UFSAR		FitzPatrick Updated Final Safety Analysis Report	7	
Procedures	NF-AA-309	Special Nuclear Material and Core Component Move	10	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Sheet Development	
		NF-AA-310	Special Nuclear Material and Core Component Movement	23
		NF-AB-110-1170	Generation of Fuel Moves Using ShuffleWorks	13
		NF-AB-309	BWR Special Nuclear Material and Core Component Move Sheet Development	3
		OSP-66.001	Management of Refueling Activities	15
71153	Corrective Action Documents	04424276		
	Procedures	ISP-107A	RPS Drywell Pressure Instrument Response Time Test (ATTS)	6
	Work Orders	82603706		