



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
REGION II  
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200  
ATLANTA, GEORGIA 30303-1200

July 27, 2021

Mr. Jim Barstow  
Vice President, Nuclear Regulatory Affairs & Support Services  
Tennessee Valley Authority  
1101 Market Street  
LP 4A-C  
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH, UNITS 1 AND 2 – INTEGRATED INSPECTION REPORT  
05000327/2021002 AND 05000328/2021002

Dear Mr. Barstow:

On June 30, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Sequoyah, Units 1 and 2. On July 20, 2021, the NRC inspectors discussed the results of this inspection with Mr. Tom Marshall and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

*/RA/*

Thomas A. Stephen, Chief  
Reactor Projects Branch #5  
Division of Reactor Projects

Docket Nos. 05000327 and 05000328  
License Nos. DPR-77 and DPR-79

Enclosure:  
As stated

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SUBJECT: SEQUOYAH, UNITS 1 AND 2 – INTEGRATED INSPECTION REPORT  
05000327/2021002 AND 05000328/2021002 DATED JULY 27, 2021

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DATE	07/21/2021	07/21/2021	07/21/2021		

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**U.S. NUCLEAR REGULATORY COMMISSION  
Inspection Report**

Docket Numbers: 05000327 and 05000328

License Numbers: DPR-77 and DPR-79

Report Numbers: 05000327/2021002 and 05000328/2021002

Enterprise Identifier: I-2021-002-0018

Licensee: Tennessee Valley Authority

Facility: Sequoyah, Units 1 and 2

Location: Soddy Daisy, TN 37379

Inspection Dates: April 01, 2021 to June 30, 2021

Inspectors: N. Childs, Resident Inspector  
S. Downey, Senior Reactor Inspector  
D. Hardage, Senior Resident Inspector

Approved By: Thomas A. Stephen, Chief  
Reactor Projects Branch #5  
Division of Reactor Projects

Enclosure

## **SUMMARY**

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Sequoyah, Units 1 and 2, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

### **List of Findings and Violations**

No findings or violations of more than minor significance were identified.

### **Additional Tracking Items**

None.

## PLANT STATUS

Unit 1 began the inspection period at rated thermal power (RTP). On April 5, 2021, the unit entered end of life coast down, and on April 10, 2021, the unit was shutdown from 95 percent RTP for refueling outage 1R24. On May 10, 2021, the reactor was started up and the unit returned to RTP on May 14, 2021. On May 24, 2021, the reactor tripped on power range nuclear instrument negative rate due to a failure of the Control Bank B Group 2 stationary gripper phase control card. The unit was returned to RTP on June 1, 2021 and remained at or near RTP for the remainder of the inspection period.

Unit 2 operated at or near RTP for the entire inspection period.

## INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards. Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), resident and regional inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time, the resident inspectors performed periodic site visits each week, increasing the amount of time on site as local COVID-19 conditions permitted. As part of their onsite activities, resident inspectors conducted plant status activities as described in IMC 2515, Appendix D; observed risk significant activities; and completed on site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or a portion of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on site. The inspections documented below met the objectives and requirements for completion of the IP.

## REACTOR SAFETY

### 71111.01 - Adverse Weather Protection

#### Seasonal Extreme Weather Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated readiness for seasonal extreme weather conditions prior to the onset of seasonal summer temperatures including review the material condition of the plant's offsite AC power systems and onsite alternate AC power systems including the switchyard and transformers on May 26, 2021.

### 71111.04 - Equipment Alignment

#### Partial Walkdown Sample (IP Section 03.01) (4 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 1 residual heat removal system (RHRS) with both trains in service for cooldown in Mode 5 with reactor coolant system (RCS) level below the flange on April 13, 2021
- (2) Unit 1 RHRS with both aligned for low pressure safety injection in Mode 1 following realignment from 1R24 on May 23, 2021
- (3) Unit 1 auxiliary feedwater (AFW) system following replacement of 1B AFW pump on June 11, 2021
- (4) Unit 1 emergency core cooling system pumps and 1A emergency diesel generator (EDG) with 1B-B EDG inoperable due to failure to start on June 28, 2021

#### 71111.05 - Fire Protection

##### Fire Area Walkdown and Inspection Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) Unit 1 Reactor Building, Lower Containment, Elevation 679 on May 3, 2021
- (2) Unit 1 Reactor Building, Upper Containment, Elevation 734 on May 5, 2021
- (3) Unit 2 Auxiliary Building (Non-RCA), Elevation 734 on June 10, 2021
- (4) Unit 1 and Unit 2 Auxiliary Building, Elevation 653 on June 24, 2021
- (5) Unit 1 and Unit 2 Auxiliary Instrument Rooms, Control Building Elevation 685 on June 28, 2021

##### Fire Brigade Drill Performance Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated the onsite fire brigade performance in response to a fire in 1B AFW pump on May 27, 2021.

#### 71111.08P - Inservice Inspection Activities (PWR)

##### PWR Inservice Inspection Activities Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated pressurized water reactor non-destructive testing by reviewing the following examinations from May 10 to May 14, 2021:
  1. Liquid Penetrant Examination
    - a. RHRHXH-18-A-2-IA, Integral attachment to Residual Heat Removal Heat Exchanger 1A, ASME Class 2
    - b. Weld 0-ER-1715A, Pipe to elbow weld, ASME Class 3. This included a review of the associated welding activities.
    - c. Weld 1-RX-2134A, Transition piece to elbow weld, ASME Class 1. This included a review of the associated welding activities.
  2. Magnetic Particle Examination
    - a. Weld RFDW-2, Steam Generator 2 Feedwater Nozzle to Shell weld, ASME Class 2
  3. Radiographic Examination
    - a. Weld SIF-196, Pipe to valve Weld, ASME Class 1

4. Ultrasonic Examination
  - a. Weld AFWF-150, Pipe to reducer weld, ASME Class 2
  - b. Weld RFDW-2, Steam Generator 2 Feedwater Nozzle to Shell weld, ASME Class 2
  - c. Weld RCS-032, Pipe to elbow weld, ASME Class 1
  - d. Weld SI-1693, socket weld, ASME Class 1
  - e. Weld SIF-196, Pipe to valve weld, ASME Class 1
5. Visual Examination
  - a. Bare metal visual (remote) examination of Reactor Pressure Vessel Closure Head and penetrations, ASME Class 1

The Inspectors also evaluated the licensee's boric acid corrosion control program performance.

#### 71111.11Q - Licensed Operator Regualification Program and Licensed Operator Performance

##### Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

- (1) The inspectors observed and evaluated licensed operator performance in the main control room during shutdown activities for U1R24 refueling outage on April 9 - 10, 2021.

##### Licensed Operator Regualification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated a simulator examination scenario on June 14, 2021.

#### 71111.12 - Maintenance Effectiveness

##### Maintenance Effectiveness (IP Section 03.01) (2 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) Failure of 1-FCV-67-344, essential raw cooling water (ERCW) to 1B-B pipe chase cooler, to open during surveillance testing on November 23, 2020
- (2) Auxiliary building secondary containment enclosure isolation damper 2-FCO-30-272 failed to close on auxiliary building isolation actuation on February 19, 2021

#### 71111.13 - Maintenance Risk Assessments and Emergent Work Control

##### Risk Assessment and Management Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) Unit 1, Yellow shutdown risk week of April 11 – 17, 2021, while RCS level was below Reactor (Rx) flange level and during Rx core offload, including review of defense in depth protected equipment for U1R24.
- (2) Unit 1, Yellow shutdown risk week of April 18 – 23, 2021, during reactor core offload concurrent with maintenance on the 'A' RHR train, including review of defense in depth protected equipment for U1R24.
- (3) Unit 1, Yellow shutdown risk week of May 2-8, 2021, while RCS level was below Rx flange level and RCS heat up from Mode 5 to normal operating temperature including review of defense in depth protected equipment for U1R24.
- (4) Unit 1 and 2, week of May 23-29, 2021, including a reactor trip on Unit 1 and Yellow shutdown risk following a fire in the 1B AFW pump, including protection equipment reviews for emergent maintenance and a conservative operations alert.
- (5) Unit 1 and 2, week of June 6-12, 2021, including protection equipment reviews for planned maintenance on the 'B' auxiliary air compressor and emergent maintenance on 'D' air compressor.
- (6) Unit 1 and 2, week of June 27-30, 2021, including planned replacement of 1B start bus and emergent maintenance on 1B EDG, including protection equipment reviews for the emergent maintenance and a conservative operations alert.

#### 71111.15 - Operability Determinations and Functionality Assessments

##### Operability Determination or Functionality Assessment (IP Section 03.01) (6 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) 1B Centrifugal Charging Pump Gear Oil Cooler has lower than expect flow on May 5, 2021 (CR 1691745)
- (2) ERCW leak on internal coils of the 2B 669 pipe chase cooler on May 12, 2021 (CR 1693400)
- (3) 1B motor driven auxiliary feedwater (MDAFW) pump differential pressure below acceptance criteria on May 29, 2021 (CR 1697633)
- (4) Prompt determination of operability request for 1-FCV-68-333 power operated relief valve block valve on April 23, 2021 (CR 1688783)
- (5) Nuts found loose on flange of containment vacuum relief valve 1-VLV-30-47 on April 30, 2021 (CR 1688414)
- (6) Conduit for power range nuclear instrument N41 detector filled with borated water on May 4, 2021 (CR 1690093)

#### 71111.19 - Post-Maintenance Testing

##### Post-Maintenance Test Sample (IP Section 03.01) (8 Samples)

The inspectors evaluated the following post-maintenance test activities to verify system operability and functionality:

- (1) Work Order (WO) 121287367, ERCW header 1A isolation valve 1-BCTD-3-116A clean square D contactor on April 7, 2021
- (2) WO 121157462, Repack outboard end of Unit 1 turbine driven auxiliary feedwater (TDAFW) pump due to cocked gland on April 9, 2021



- (3) WO 120868165, 1-FCV-74-3, '1A' RHR pump suction valve failed seat leak test, rework valve on April 29, 2021
- (4) WO 122073016, Nuts found loose on bolting surface of 1-VLV-30-46, Containment Vacuum Relief Valve on May 6, 2021
- (5) WO 117818333, Replace unit 1 reactor coolant pump #2 seal on May 8, 2021
- (6) WO 119467102, 1-VLV-1-516 remove valve for testing offsite and replace gaskets on May 8, 2021
- (7) WO 122074616, Replace 1-VLV-063-0559, safety injection loop 2 hot leg check valve, on May 8, 2021
- (8) WO 122153643, Replace 1B-B AFW pump on May 29, 2021

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated U1R24 outage activities from April 10 to May 11, 2021.

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Surveillance Tests (other) (IP Section 03.01) (3 Samples)

- (1) 1-SI-SXP-003-202.A, MDAFW 1A-A Comprehensive Performance Test on April 7, 2021
- (2) 1-SI-OPS-088-001.0, Phase A Isolation Test on April 11, 2021
- (3) 1-SI-SXP-003-202.S, TDAFW Comprehensive Performance Test on April 9, 2021

Inservice Testing (IP Section 03.01) (2 Samples)

- (1) 0-SI-SXV-001-859.0, Testing and Setting of Main Steam Safety Valves on April 7, 2021
- (2) 1-SI-SXP-003-202.B, MDAFW 1B-B Comprehensive Test on May 31, 2021

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) 0-SI-SLT-043-258.1, Containment Isolation Valve Local Leak Rate Test Post Accident Sampling on April 19, 2021

Ice Condenser Testing (IP Section 03.01) (1 Sample)

- (1) 0-SI-MIN-061-109.0, Ice Condenser Intermediate and Lower Inlet Doors and Vent Curtains on May 2, 2021

**OTHER ACTIVITIES – BASELINE**

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

IE01: Unplanned Scrams per 7000 Critical Hours Sample (IP Section 02.01) (2 Samples)

- (1) Unit 1 (April 2020 – March 2021)
- (2) Unit 2 (April 2020 – March 2021)

IE03: Unplanned Power Changes per 7000 Critical Hours Sample (IP Section 02.02) (2 Samples)

- (1) Unit 1 (April 2020 – March 2021)
- (2) Unit 2 (April 2020 – March 2021)

IE04: Unplanned Scrams with Complications (USwC) Sample (IP Section 02.03) (2 Samples)

- (1) Unit 1 (April 2020 – March 2021)
- (2) Unit 2 (April 2020 – March 2021)

71152 - Problem Identification and Resolution

Semiannual Trend Review (IP Section 02.02) (1 Sample)

- (1) The inspectors reviewed the licensee's corrective action program for potential adverse trends related to ice condenser ice baskets being frozen in place which might be indicative of a more significant safety issue.

Annual Follow-up of Selected Issues (IP Section 02.03) (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) Unit 1 Reactor Vessel Head Vent Degradation

71153 - Follow Up of Events and Notices of Enforcement Discretion

Event Follow up (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated licensee performance in response to a fire in 1B-B AFW pump on May 27, 2021.

Personnel Performance (IP Section 03.03) (1 Sample)

- (1) The inspectors evaluated licensee performance in response to Unit 1 reactor trip on May 24, 2021.

**INSPECTION RESULTS**

No findings were identified.

**EXIT MEETINGS AND DEBRIEFS**

The inspectors verified no proprietary information was retained or documented in this report.

- On July 20, 2021, the inspectors presented the integrated inspection results to Mr. Tom Marshall and other members of the licensee staff.
- On May 13, 2021, the inspectors presented the ISI Exit Meeting inspection results to Chris Reneau, Plant Manager and other members of the licensee staff.

## DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.01	Procedures	0-PI-OPS-000-006.1	Summer Operation	Revision 9
		AOP-P.07	Degraded Grid or Abnormal Voltage Conditions	Revision 17
		NPG-SPP-07.1.6	On Line Work Control Power Systems Alerts/Offsite Power	Revision 8
71111.04	Miscellaneous	0-PE-2021 / 1-82-0006	Protected Equipment/Clearance Section for 1B-B D/G Inoperable	06/28/2021
	Procedures	0-SO-74-1	Residual Heat Removal System	Revision 109
		1-SO-3-2	Auxiliary Feedwater System	Revision 61
71111.05	Corrective Action Documents	CR 1697239	Report of sparks coming from 1B-B MDAFW pump	05/27/2021
		CR 1697332	Fire Emergency Response: 1B MDAFW pump	05/27/2021
		CR 1700545	Temporary Cat5e Cable routed within the 1-foot boundary of safety related cable trays and conduit	06/11/2021
	Fire Plans	AUX-0-653-00	Pre-Fire Plan Auxiliary Building EI 653	Revision 9
		AUX-0-734-01	Pre-Fire Plan Auxiliary Building EI 734 Unit 1 Side	Revision 10
		AUX-0-734-02	Pre-Fire Plan Auxiliary Building EI 734 Unit 2 Side	Revision 9
		CON-0-685-00	Pre-Fire Plan, Control Building EI. 685	Revision 6
		RXB-0-679-02	Pre-Fire Plan for Reactor Building (Elevation 679)	Revision 3
	RXB-0-734-02	Pre-Fire Plan for Reactor Building (Elevation 734) and Annulus (Elevation 740, 759, and 778)	Revision 3	
	Miscellaneous	Standing Order: SO-21-017	ACMP Fire Watch Camera Monitoring Plan	05/12/2021
71111.08P	Corrective Action Documents	CRs (by number)	1463750, 1564272, 1580350, 1584849, 1608941, 1685545, 1686073, 1689057, 1689365, 1693829	
	NDE Reports	R-0478	Ultrasonic Examination Report for RCS-032	04/13/2021
		R-0506	Ultrasonic Examination Report for SI-1693	04/20/2021
		R-0513	Magnetic Particle Examination Report for RFDW-2	04/21/2021
		R-0519	Visual Examination Report for Reactor Pressure Vessel Closure Head	04/26/2021
		R-0524	Ultrasonic Examination Report for RFDW-2	04/26/2021
	Procedures	N-GP-18	Ultrasonic Testing Supplements	Revision 0023

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		N-MT-6	Magnetic Particle Examination for ASME and ANSI Code Components and Welds	Revision 0037
		N-UT-19	Ultrasonic Examination of Welds in Vessels Greater than 2-inches in Well Thickness	Revision 0020
		N-UT-64	Generic Procedure for the Ultrasonic Examination of Austenitic Pipe Welds	Revision 0018
		N-UT-85	Manual Ultrasonic Examination for the Detection of Thermal Fatigue in Piping and Components within the Material Reliability Programs	
		NPG-SPP-09.7.4	Boric Acid Corrosion Control Program	Revision 0006
	Work Orders	Work Orders (by number)	120593669, 121132022	
71111.11Q	Miscellaneous	SEG #OPL273E2130	LOR Cycle 21-03 Simulator Exam	06/14/2021
	Procedures	1-SI-OPS-082-026.B	Loss of Offsite Power with Safety Injection D/G 1B-B Test	Revision 56
		1-SI-OPS-088-001.0	Phase A Isolation Test	Revision 24
71111.12	Corrective Action Documents	CR 1654164	Failed valve stroke for 1-FCV-67-344	11/23/2020
		CR 1660457	Replace packing gland/packing gland follower, gland stud, gland stud nut and repack valve	12/23/2020
		CR 1661050	Failed PMT valve stroke for 1-FCV-67-344	12/29/2021
		CR 1663703	1-FSV-67-344-B basic cause determination	01/12/2021
		CR 1672837	2-FCO-30-272 did not go closed with a valid auxiliary building isolation (ABI) signal present	02/19/2021
		CR 1684582	Maintenance rule basic cause documentation of 2-FCO-30-272	04/07/2021
	Miscellaneous	CDE 3125	Cause determination evaluation for 1-FCV-67-344 stroke failure	03/17/2021
		CDE 3132	2-FCO-30-272 failed to close on ABI actuation	Revision 1
	Procedures	NPG-SPP-03.4	Maintenance Rule Performance Indicator Monitoring , Trending and Reporting - 10CFR50.65	Revision 4
TI-4, Attachment		Heating, Ventilation, and Air Conditioning - System 030	Revision 31	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		#7		
	Work Orders	121941721; 121990076; 122050297		
71111.13	Corrective Action Documents	CR 1698157	Risk Classification of Works Orders during Forced Outage	06/02/2021
	Corrective Action Documents Resulting from Inspection	CR 1704128	Error in online EOOS, 1B EDG not included after failure	06/28/2021
	Procedures	0-GO-16	System Operability Checklists	Revision 28
71111.15	Corrective Action Documents	CR 1686772	Need Engineering review for 1-FCV-68-333 MOV Test Data. Test Values exceeded acceptance criteria.	04/15/2021
		CR 1690524	No ERCW flow indicated on 1B CCP	04/30/2021
		CR 1691051	1B-B CCP speed increaser gearing oil cooler has insufficient cooling flow	05/03/2021
		CR 1691423	Document critical thinking related to CR 1690093 (Insulation resistance of N41 cable met acceptance criteria).	05/04/2021
	Engineering Evaluations	EWR 21-DEM-003-075	1B-B MDAFW Pump Replacement total developed head requirement	05/29/2021
		EWR-21-DEM-067-065	Hydraulic Flow Values Associated with 1B-B Cent. Charging Pump Gear Oil Cooler	05/05/2021
	Operability Evaluations		Prompt Determination of Operability for PER 732852 (2B Pipe Chase Cooler Leak)	05/31/2013
			Prompt Determination of Operability for CR 1611076 (B-B EGTS Cooler Leak)	05/29/2020
71111.19	Corrective Action Documents	CR 1697633	1B MDAFW Pump DP below acceptance criteria	05/29/2021
	Procedures	0-SI-SXT-000-201.0	ASME Section XI Inservice Pressure Test	Revision 25
		0-SI-SXV-003-266.0	ASME Code Valve Testing	Revision 51
		1-SI-SFT-063-001.0	Safety Injection System Hot Leg and Cold Leg Injection Flow Test	Revision 12

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		1-SI-SXI-068-201.0	Leakage Test of the Reactor Coolant System Pressure Boundary	Revision 16
		1-SI-SXP-003-201.B	Motor Driven Auxiliary Feedwater Pump 1B-B Performance Test	Revision 18
		N-VT-4	System Pressure Test Visual Examination Procedure	Revision 30
	Work Orders	WO 120573825	1-SI-SXI-068-201.0 Unit 1 leakage test of the reactor coolant pressure boundary	05/08/2021
		WO 121842073	Perform Seat Leak Test on 1-FCV-74-003	04/29/2021
71111.20	Corrective Action Documents	CR 1685395	CR to document lessons learned from SI-127 performance (documentation of cooldown rates)	04/11/2021
	Corrective Action Documents Resulting from Inspection	CR 1685321	1-VLV-068-0580 has dry boron	04/10/2021
		CR 1685322	1-VLV-068-0557 has dry boric acid on stem	04/10/2021
		CR 1685323	White powder (boric acid or borax) under RCPs on floor	04/10/2021
		CR 1690093	Fluid Coming Out of Conduit with White Residue around Joints	04/28/2021
	Procedures	CR 1692251	Ice Condenser Drain Pipe leaking on #4 RCP Platform	05/06/2021
		1-SI-OPS-082-026.B	Loss of Offsite Power with Safety Injection D/G 1B-B Test	Revision 56
71111.22	Corrective Action Documents	CR 1684542	MSSV 1-VLV-001-0516 is suspected of leakage at inlet flange	04/07/2021
		CR 1684653	"B" AFW flow indicator 3-147A is showing zero flow during comprehensive test	04/07/2021
		CR 1697806	MDAFW critical spare exhibited marginal performance	05/31/2021
		CR 1697913	Low Margin for MDAFW 1B-B	06/01/2021
	Corrective Action Documents Resulting from Inspection	CR 1690959	Unit 1 Ice Condenser Bay 18 RH Lower Inlet Door Spring Loose	05/02/2021
	Drawings	1-47W881 sheet 6	Mechanical Flow Drawing Water Quality and Sampling System	Revision 4
71152	Corrective Action Documents	CR 1691538	Unit 1 Ice Condenser basket trend	05/04/2021
		CR 1691616	U1R24 Ice Condenser Partial Bay Melt Out Lessons Learned	05/04/2021
	Procedures	0-SI-MIN-061-	Ice Condenser - Ice Weighing	Revision 13

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		105.0		