



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1200

July 26, 2021

Mr. James Barstow
Vice President, Nuclear Regulatory Affairs and Support Services
Tennessee Valley Authority
Tennessee Valley Authority
1101 Market Street, LP 4A-C
Chattanooga, TN 37402-2801

SUBJECT: BROWNS FERRY NUCLEAR PLANT – DESIGN BASIS ASSURANCE
INSPECTION (TEAMS) INSPECTION REPORT 05000259/2021010 AND
05000260/2021010 AND 05000296/2021010

Dear Mr. Barstow:

On June 29, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Browns Ferry Nuclear Plant and discussed the results of this inspection with Mr. Chris Dunn and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

James B. Baptist, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos. 05000259 and 05000260 and 05000296
License Nos. DPR-33 and DPR-52 and DPR-68

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV®

SUBJECT: BROWNS FERRY NUCLEAR PLANT – DESIGN BASIS ASSURANCE INSPECTION (TEAMS) INSPECTION REPORT 05000259/2021010 AND 05000260/2021010 AND 05000296/2021010 – DATED July 26, 2021

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OFFICE	RII/DRS	RII/DRS	RII/DRS	RII/DRS	RII/DRS
NAME	D. Bullock	W. Deschaine	J. Winslow	R. Patterson	J. Baptist
DATE	7/19/2021	7/20/2021	7/20/2021	7/23/2021	7/26/2021

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Numbers: 05000259, 05000260 and 05000296

License Numbers: DPR-33, DPR-52 and DPR-68

Report Numbers: 05000259/2021010, 05000260/2021010 and 05000296/2021010

Enterprise Identifier: I-2021-010-0038

Licensee: Tennessee Valley Authority

Facility: Browns Ferry Nuclear Plant

Location: Athens, AL

Inspection Dates: May 17, 2021 to June 11, 2021

Inspectors: C. Baron, Contractor
D. Bollock, Senior Reactor Operations Engineer
W. Deschaine, Senior Resident Inspector
S. Kobylarz, Contractor
R. Patterson, Senior Reactor Inspector
J. Winslow, Reactor Systems Engineer

Approved By: James B. Baptist, Chief
Engineering Branch 1
Division of Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a design basis assurance inspection (teams) inspection at Browns Ferry Nuclear Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards. Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), inspectors were directed to begin telework. In addition, regional baseline inspections were evaluated to determine if all or a portion of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on site. The inspections documented below met the objectives and requirements for completion of the IP.

REACTOR SAFETY

71111.21M - Design Bases Assurance Inspection (Teams)

The inspectors evaluated the following components and listed applicable attributes, permanent modifications, and operating experience:

Design Review - Risk-Significant/Low Design Margin Components (IP Section 02.02) (4 Samples)

- (1) 250V DC Reactor Motor Operated Valve (RMOV) Board 2B
 - Material condition and configuration (e.g., visual inspection during a walkdown)
 - Operating environment
 - Consistency between station documentation (e.g. procedures) and vendor specifications
 - Maintenance effectiveness
 - Corrective maintenance records, and corrective action history
 - Panel loading
 - Load voltage adequacy
 - Overcurrent protection and coordination

- (2) Emergency Essential Cooling Water (EECW) Strainers
 - Material condition and configuration (e.g., visual inspection during a walkdown by onsite inspector)
 - Consistency between station documentation (e.g. procedures and drawings) and vendor specifications
 - Corrective maintenance records, and corrective action history
 - System Health Reports
 - Compliance with UFSAR, TS, and TS Bases

- Normal and emergency operating procedures
- Modifications
- Completed surveillance tests to ensure acceptance criteria have been met

(3) 480V RMOV BD 2D

- Material condition and configuration (e.g., visual inspection during a walkdown)
- Operating environment
- Consistency between station documentation (e.g. procedures) and vendor specifications
- Maintenance and Preventive Maintenance effectiveness
- Corrective maintenance records, and corrective action history
- Breaker short circuit capacity
- BD (MCC) loading
- BD (MCC) voltage adequacy
- Overcurrent protection and coordination

(4) Unit 2 Reactor Core Isolation Cooling (RCIC) System Pump 2B

- Normal and Emergency Operating Procedures
- Surveillance Test Procedures and Recent Results
- Inservice Test Procedures and Recent Results
- Bases for Pump Test Acceptance Criteria
- Calculation of Pump Capacity
- Calculation of Pump NPSH
- Calculation of System Pressure due to Pump Overspeed Condition
- Operation of RCIC under Station Blackout Conditions
- Evaluation of RCIC Pump Suction Transfer
- Material Condition of Pump and Associated Equipment
- Corrective Action History

Design Review - Large Early Release Frequency (LERFs) (IP Section 02.02) (1 Sample)

(1) Unit 2 Automatic Depressurization System (ADS) Valves

- Normal and Emergency Operating Procedures
- Calculation of Instrument Air Supply and Air Accumulator Capacity
- Instrument Air Leakage Test Procedures and Recent Results
- Recent Valve Test Results
- Evaluation of Electrical Control and Power Supplies
- Evaluation of Testing of Backup Electrical Control and Power Supplies
- Corrective Action History

Modification Review - Permanent Mods (IP Section 02.03) (6 Samples)

- (1) DCN 71425, Upgrade Unit 1/2 Emergency Diesel Generator Fuel Oil Piping Configuration
- (2) DCN 72338-03, Replacement of 4160V/480V Emergency Shutdown Board 2A & 2B Transformer
- (3) BFN-19-918-01-U1, High Pressure Coolant Injection Gland Seal Check Valve Addition
- (4) DCN 69466A, Replace Core Spray and Residual Heat Removal Room Cooler Fan Motors, Fans, and Shafts
- (5) BFN-1-2020-068-003, Temporary Replacement of Recirculation Loop 1B Flow Measurement (T-Mod)
- (6) DCN 70747-71, 480V RMOV BD 2A/15D ALT FDR from SD BD

Review of Operating Experience Issues (IP Section 02.06) (2 Samples)

- (1) NRC IN 2009-09, Improper Flow Controller Settings Renders Injection Systems Inoperable and Surveillance Did Not Identify
- (2) NRC IN 91-50: A Review of Water Hammer Events After 1985

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On June 29, 2021, the inspectors presented the design basis assurance inspection (teams) inspection results to Mr. Chris Dunn and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.21M	Calculations	BFN-1-47E859-1	Flow Diagram, Emergency Equipment Cooling Water	Rev. 99
		CDQ-0067-881511	Seismic Qualification for the Automatic Self-Cleaning Strainer	Rev. 1
		CDQ-0067894672	Evaluation and Review of Small Bore Unit 0 Piping and Supports within the Scope of the System 067 Safe Shutdown Boundary	Rev. 12
		CDQ-1067895300	Pipe Stress Analysis of Stress Problem No. 1-67-859-1-56	Rev. 3
		CDQ-2067890748	Strainer Backwash Blowdown Piping at Pumping Station	Rev. 2
		CDQ0303880436	Assess the RHRSW and the EECW Piping and Components for the Affects of Tornadic Wind- RHRSW Pump	Rev. 2
		CDQ0999910005	Evaluation of Seismic- Induced Spray Hazards at BFN, Unit 2 and Common	Rev. 3
		CDQ107320021062	Small Bore Piping and Supports Program System Calculation for Seismic Class 1 HPCI System (73) Piping	Rev. 6
		CDQ107320031063	Pipe Stress Analysis of Stress Problem N1-173-67R	Rev. 1
		EDQ0057920034	4.16KV and 480V Busload, Voltage Drop and Short Circuit Calculation	Rev. 92
		EDQ024820020042	250V DC Unit Battery Load Study	Rev. 91
		EDQ2000870054	Control Circuit Voltage Drop Calculations – 250V DC circuits	Rev. 43
		EDQ2000870550	Cable and Bus Protection/Breaker/Fuse Coordination for 250V DC System	Rev. 54
		EDQ2092900118	Setpoint and Scaling Calculation for Neutron Monitoring System & Recirculation Flaw Loops	Rev. 37
		EDQ2999870549	Power Cable Protection Analysis for 480V MCCs	Rev. 54
		EQD29998801715	Thermal Overload Heater Calculation	Rev. 50
		MD-N0026-910163	Combustible Load Table	Rev. 71
		MDQ-0000672019000460	MOV 0-FCV-067-0001/0005/0008/0011, Operator Requirements and Capabilities	Rev. 1
MDQ-0067890059	Analytical Limits for the EECW Strainers and Strainer	Rev. 2		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date	
			Drain Valves Start/Stop Controls		
		MDQ0000712012000031	RCIC Pump NPSHa and System Hydraulic Analysis (Pump Water Suction and Discharge)	Rev. 3	
		MDQ0000732012000062	Calculation of Effects of Gas Accumulation in ECCS Piping	Rev. 0	
		MDQ0032870288	Control Air Volume and Wall Thickness of Accumulators	Rev. 14	
		MDQ0071910235	Reactor Core Isolation Cooling System Design Pressure/Temperature	Rev. 12	
		MDQ099920040034	Setpoint Controls Parameters Review Calculation for BFN Category 2 Air Operated Valves (AOVs)	Rev. 17	
		MDQ099920040040	HPCI and RCIC Test Requirements	Rev. 11	
		NDN00099920070032	BFN Probabilistic Risk Assessment - Human Reliability Analysis	Rev. 8	
	Corrective Action Documents	0044691			
		0128951			
		0175232			
		0200183			
		1648705			
		1649288			
		1649610			
1652318					
1369055					
1406271					
1499731					
1500527					
1557241					
1606454					
1607278					
1613986					
1614688					
1617438					
1627370					
1638651					
1655591					

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		1656541		
		1656544		
		1658162		
		1658186		
		1658998		
		1664478		
		1665909		
		1676979		
		1687954		
		1688353		
		1695185		
		1201387		
		1421012		
		1434874		
		1435350		
		1613271		
	Corrective Action Documents Resulting from Inspection	1695185	A3 RHRSW Lube Oil Strainer Sight Glass	
		1695266	Control of HPCI and RCIC System Flow Controller Settings	
		1695742	Danger sign was identified on the top of 480V RMOV Board 2D	
		1695748	Review and Evaluate Unit 2 RCIC System CQS Test Results	
		1695790	NRC DBAI Inspection - Minor Correction Needed on FPPCIS for ECP 71425	
		1695802	NRC DBAI 2021 Inspection - Correction Needed to 1/2-ARP-9-23A/B/C/D and 0-ARP-25-41A/B/C/D	
		1696259	Pipe Wrench Laying on Top of the C# EECW Strainer's Gearbox01	
		1696423	NRC DBAI Inspection - Incorrect M&TE Number Documented in WO 116092256	
		1696542	250V Reactor MOV Board 2B board is missing 3 finishing plugs	
		1696552	480V Reactor MOV board 2A missing 1 finishing plug	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		1696556	480V Reactor MOV board 2D remove RTV from Finishing Plug hole and install Finishing Plug	
		1696713	Request PCRs for RCIC Flowrate Procedures	
		1698638	MDQ0071910235 Evaluation Didn't Account for Tolerance	
		1699579	Fire seal label is required at the S25932501 penetration per MAI-3.4A and is missing	
		1699961	Fire Seal Penetration R26395139 not conforming to design output drawing 0-45E830-27	
		1699971	Fire Seal Penetration R36395359 not conforming to design output drawing 0-45E830-27	
		1700011	DP Between RCIC Pump Discharge and RPV	
		1700151	Response to RCIC Overspeed	
		1700334	RCIC Surveillance Procedure Non-Conservative with Respect to TS	
		1700335	NRC Observation Recirc Temp Mod	
	Drawings	0-45E830-27	Conduit & Grounding Cable Trays Fire Stop Dets-SH 14	Rev. 7
		0-47E392-1	Fire Protection – 10 CFR 50 – NFPA 805 Penetration Seal Tabular Drawings General Notes and Legend	Rev. 5
		0-47E840-3	Flow Diagram, Fuel Oil System	Rev. 27
		0-731E700	One Line Diagram of DC & Instrument Control AC Systems	Rev. 8
		1-45E751-3	Wiring Diagram, 480V Reactor MOV Board 1B	Rev. 43
		2-45E712-2	Wiring Diagram 250V Reactor MOV BD 2B Single Line	Rev. 35
		2-47E610-1-1	Mechanical Control Diagram – Main Steam System	Rev. 43
		2-47E610-71-1	Mechanical Control Diagram – RCIC System	Rev. 45
		2-47E610-71-2	Mechanical Control Diagram – RCIC System	Rev. 7
		2-47E801-1	Flow Diagram – Main Steam	Rev. 34
		2-47E813-1	Flow Diagram – Reactor Core Isolation Cooling System	Rev. 60
		2-47W2392-702	Fire Protection – 10 CFR 50 Appendix R Penetration Seal Location Drawings Plan View Area I	Rev. 1
2-47W2392-708	Fire Protection – 10 CFR 50 Appendix R Penetration Seal Location Drawings EL 639	Rev. 1		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		2-47W2392-716	Fire Protection – 10 CFR 50 Appendix R Penetration Seal Tabular Drawings EL 639	Rev. 3
		2-730E929-2	Automatic Blowdown System	Rev. 24
		2-730E929-3	Elementary Diagram - Automatic Blowdown System	Rev. 16
		2-730E929-4	Automatic Blowdown System	Rev. 18
		BFN-0-47E610-67-2	Mechanical Control Diagram, Emergency Equipment Cooling Water	Rev. 38
		BFN-1-47E610-67-1	Mechanical Control Diagram, Emergency Equipment Cooling Water	Rev. 50
		BFN-1-47E812-1-CC	Flow Diagram, High Pressure Coolant Injection System	Rev. 48
		BFN-2-45E751-1	Wiring Diagram 480V Reactor MOV BD 2A Single Line	Rev. 068
		BFN-2-45E751-8	Wiring Diagram 480V Reactor MOV BD 2D Single Line	Rev. 2
		BFN-2-47E812-1-CC	Flow Diagram, High Pressure Coolant Injection System	Rev. 77
	BFN-3-47E812-1-CC,	Flow Diagram, High Pressure Coolant Injection System	Rev. 73	
	Engineering Changes	DCN 72338-03	Replacement of 4160V/480V Emergency Shutdown Board 2A & 2B Transformer	
		EC 71425	Upgrade Unit 1/2 DG Fuel Oil Piping Configuration	Rev. A
	Miscellaneous		System Health Report Scorecard for RHRSW and EECW (April 2020-September 2020)	
			System Health Report Scorecard for RHRSW and EECW (October 2020-March 2021)	
			Vendor Letter, Instructions for Changing Assembly of DSMM40A Reducer	dated 03/12/02
			System Health Report Scorecard for RHRSW and EECW (October 2019-March 2020)	
		0-SI-4.5.C.1(A3-COMP)	RHRSW Pump A3 IST Comprehensive Pump Test	Rev. 16
		0-TI-444	Augmented Inservice Testing Program	Rev. 12
		0-TI-641	Time Critical Operator Actions	Rev. 0
0048-0051-LTR-002		BFN HPCI System Waterhammer Evaluation	Rev. 0	
0048-0065-RPT-001		HPCI and RCIC Systems Overpressure Transient Technical Review and Development of Mitigation Options	Rev. 0	
70747-71		480V RMOV BD 2A/15D ALT FDR from SD BD	Rev. A	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		BFN-1-2020-068-003	Rev. 0	
		BFN-19-918	U1, U2, and U3 HPCI Gland Seal CKV Addition	Rev. 1
		BFN-19-918-1	U1 HPCI Gland Seal CKV Addition	Rev. 0
		BFN-50-7071	Design Criteria – Reactor Core Isolation Cooling System	Rev. 23
		BFN-50-7200C	Design Basis Document for the 250V DC Power Distribution System	Rev. 8
		BFN-50-720D, 480V Auxiliary System	480V Auxiliary System	Rev. 12
		BFN-VTD-D088-0010	Installation, Operation and Maintenance for Delroyd Double Worm Gear Speed Reducers	Rev. 2
		BFN-VTD-G080-1440	Vendor Document – Instructions Installation and Maintenance of 7700 Line Motor Control Center	Rev. 2
		BFN-VTD-K143-0020	Installation and Maintenance Instructions for Kinney Automatic Self-Cleaning Strainers Model AV Series	Rev. 8
		BFN-VTD-K143-0040	Preventative Maintenance Instructions for S.P. Kinney Strainers	Rev. 0
		BFN-VTD-K143-0050	Instruction Manual for S.P. Kinney Model AV Series Motorized Automatic Self-Cleaning Strainers	Rev. 0
		BFPER971270	BF response to NRC IN 91-50 Supplement 1	Rev. 0
		DCN 69466A	Replace CS and RHR Room Cooler Fan Motors, Fans, and Shafts	Rev. A
		DCN 70747A	50.59 Evaluation	Rev. 0
		DCN 71376	Increase Rated Speed of RCIC Pump/Turbine to 4650 RPM	Rev. A
		EDC 50911	Option for Rotating EECW Strainer Gear Reducer 180 Degrees	Rev. B
		Fire Protection Impairment Permit (FPIP) # 21-90 – Unit 3 Reactor Building Elevation 621/638		
		Fire Protection Impairment Permit (FPIP) # 21-91 – Unit 2		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		Reactor Building Elevation 621/638		
		General Design Criteria Document No. BN-50-7082	Standby Diesel Generator	Rev. 29
		OPL171.051	Emergency Equipment Cooling Water	Rev. 9U1
		PMCR 1189814	Perform 4 Year Inspection of Stand-by Diesel Engine "C"	
		PMCR 1189815	Perform 12-Year Inspection of Standby Diesel "C" Engine "C"	
		PO 2740686	Spare Part, Engine, QA1	dated 01/19/17
	Procedures	0-ARP-25-41C	Panel 25-41, 0-XA-55-41C	Rev. 16
		0-FSS-001	Fire Safe Shutdown	Rev. 5
		0-FSS-2-2	U-2, RB EL 519', from R14 to a Line 10' West of R11	Rev. 8
		0-FSS-2-3	U-2, RB EL593' North of Column Line R	Rev. 13
		0-GOI-300-1/ATT-12	Outside Operator Round Log	Rev. 264
		0-OI-57B	480V/240V AC Electrical System	Rev. 199
		0-OI-57D	DC Electrical System	Rev. 179
		0-OI-67	Emergency Operating Cooling Water System	Rev. 126
		0-OI-82	Standby Diesel Generator System	Rev. 174
		0-SI-4.5.C.1(A3)	RHRWS Pump A3 IST Group A Quarterly Pump Test	Rev. 19
		0-SR-3.4.3.1.A	Bench Test Relief Valves - As Left	Rev. 6
		0-TI-346	Maintenance Rule Performance Indicator Monitoring, Trending, and Reporting - 10CFR50.65	Rev. 54
		0-TI-362	Inservice Testing Program	Rev. 59
		0-TI-636RLY	Relay Testing and Maintenance Instruction	Rev. 2
		0-TI-641	Time Critical Operator Actions	Rev. 0
		1-ARP-9-8C	Panel 1-9-8, 1-XA-55-8C	Rev. 17
		1/2-ARP-9-23C	Panel 0-9-23-8, 0-XA-55-23C	Rev. 35
		2-SR-3.3.5.1.3(E)	HPCI Suppression Chamber High Level Calibration and Functional Test	Rev. 21
		2-SR-3.3.5.1.6(ADS B)	ADS Logic System Functional Test - Bus B Time Delay Relay Calibration and Bus Power Monitor Test	Rev. 22

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		2-SR-3.5.3.3	RCIC System Rated Flow at Normal Operating Pressure	dated 01/04/21
		2-SR-3.5.3.3	RCIC System Rated Flow at Normal Operating Pressure	dated 10/05/09
		2-SR-3.5.3.3(COMP)	RCIC Comprehensive Pump Test	dated 10/10/20
		2-SR-3.5.3.4	RCIC System Rated Flow at Low RPV Pressure	Rev. 26
		ADS Logic System Functional Test - Bus B Time Delay Relay Calibration and Bus Power Monitor Test	Rev. 17	
		ECI-0-000-BRK008	Testing and Troubleshooting of Molded Case Circuit Breakers and Motor Starter Overload Relays	dated 03/21/15
		EII-0-000-TCC106	Troubleshooting, Documentation and Configuration Control of Electrical Activities	dated 03/26/15
		EPI-0-000-MCC001	Maintenance and Inspection of 480VAC and 250VDC Motor Control Centers	dated 03/20/15
		EPI-0-281-RLY001	Relay Calibration and Functional Tests on 250V DC Reactor MOV Boards	Rev. 13
		MAI-3.4A	Internal Conduit Seals	Rev. 17
		MCI-0-001-VLV002	Main Steam Relief Valves Target Rock Model 7567 Disassembly, Inspection, Repair and Reassembly	Rev. 54
		MSI-0-071-GOV001	Reactor Core Isolation Cooling (RCIC) Overspeed Trip Test	Rev. 36
		MSI-2-001-TST002	Testing of Air Supply System (Drywell Side-East) for Main Steam Isolation Valves and Automatic Depressurization System Main Steam Relief Valves	Rev. 14
		MSI-2-001-TST003	Testing Air Supply System (Drywell Side-West) for Main Steam Isolation Valves and Automatic Depressurization System Main Steam Relief Valves	Rev. 12
		NEDP-10	Rev. 25	
		NEDP-2	Design Calculation Process Control	Rev. 25
		NEDP-22	Operability Determinations and Functional Evaluations	Rev. 21

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		NPG-SPP-09.3	Plant Modifications and Engineering Change Control	Rev. 35
		NPG-SPP-22.206	Verification Program	Rev. 7
	Work Orders	116092256		
		117821889		
		117821892		
		117821901		
		117821908		
		118569627		
		118800332		
		119041410		
		121166362		
		120898561		
		118571292		
		118571299		
		120561899		
		119698618		
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		121260764		
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118323602				
115753060				
01-012729-000				
01-012729-001				
01-012729-002				
01-012729-003				
01-012730-000				
01-012730-001				
01-012730-002				
121325467				

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		121331211 121385840 121390551 121504079 121504079 121785957 121808045		
		119585512		
		119639739		