Southern Nuclear

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U. S. Nuclear Regulatory Commission
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Washington, D. C. 20555-0001

Joseph M. Farley Nuclear Plant - Unit 1
Inservice Inspection Program Owner's Activity Report (OAR-1) for Outage 1R30

Ladies and Gentlemen:

The Joseph M. Farley Nuclear Plant Unit 1 OAR-1 for the 1R30 Refueling Outage is provided as an Enclosure to this letter, in accordance with Article IWA-6200 and ASME Section XI Code Case N-532-5. This Enclosure includes Table 1, "Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service," which lists evaluations performed for continued service, and Table 2, "Abstract of Repair / Replacement Activities Required for Continued Service," which lists repair/replacement activities.

This report is for the second period of the 5th Inservice Inspection Interval (Interval 5, Period 2, Outage 1).

This letter contains no NRC commitments. If you have any questions, please contact Jamie Coleman at 205.992.6611.

Respectfully submitted,

Cheryl A. Gayheart
Regulatory Affairs Director

CAG/dsp/cbg

Enclosure: 1R30 Form OAR-1 Owner's Activity Report

cc: Regional Administrator, Region II
NRR Project Manager – Farley Nuclear Plant
Senior Resident Inspector – Farley Nuclear Plant
RTYPE: CFA04.054

Joseph M. Farley Nuclear Plant - Unit 1
Inservice Inspection Program Owner's Activity Report for Outage 1R30

Enclosure

1R30 Form OAR-1 Owner's Activity Report

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number 1-5-2-1 (Unit 1, 5th Interval, 2nd Period, 1st Report)

Plant Joseph M. Farley Nuclear Plant

Unit No. 1 (if applicable) Commercial service date December 01, 1977 Refueling outage no. 1R30

Current inspection interval 5th (1st, 2nd, 3rd, 4th, other)

Current inspection period 2nd (1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plans: The 2007 Edition through 2008 Addenda is applicable to the 5th Inspection Interval.

Date and revision of inspection plans: 5th Inspection Interval Inspection Plans - Volume 1- 10/11/2020 (Version 5.0), Volume 3 – 3/17/2021 (Version 4.0), Volume 4 – 3/18/2021 (Version 3.0), and Volume 5 – 3/2/2021 (Version 3.0). 1R30 Outage Plan – 3/18/2021 (Version 1.0) with 1R30 Outage Plan Scope Change SC-001 - 4/8/2021 and 1R30 Outage Plan Scope Change SC-002 - 4/16/2021

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans: Same

Code Cases used for inspection and evaluation: N-716-1, N-722-1, N-776, N-513-4, N-532-5, N-648-2, and N-770-5
(if applicable, including cases modified by Case N-532 and later revisions)

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of 1R30 (refueling outage number) conform to the requirements of Section XI.

Signed R. M. McAdams Engineering Director Date 06/28/2021
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and employed by The Hartford Steam Boiler Inspection and Insurance Company of Hartford, CT. have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Rodney Senn Commission 12603 C I N R
Inspector's Signature (National Board Number and Endorsement)

Date 06/29/2021

Table 1
Farley Nuclear Plant Unit 1, 5th Interval, 2nd Period, 1st Report
Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
D-B D2.10	A through-wall leak was noted in the 8" Service Water piping of the Unit 1 B train Service Water pumps mini-flow, downstream of Q1P16V579, B TRN SW MINIFLOW.	CR 10675320 documents the evaluation for operability. The location was isolated and is being tracked per clearance order 1-DT19-P16-00607.
D-B D2.10	A through-wall leak was noted on the 6" Service Water piping of the 1C Service Water pump mini-flow line on an elbow in the horizontal section of piping that ties into the main Service Water header between Q1P16V506 and Q1P16V507 and is upstream of the mini-flow isolation valve Q1P16V575.	CR 10735066 documents the evaluation for operability. The flaw was evaluated in accordance with Code Case N-513-4, reference Request for Engineering Review (RER) SNC1115980, and the piping was later replaced by work order SNC930301.
D-B D2.10	A through-wall leak was noted on the Service Water return piping from the 2B Diesel Generator near the flange upstream of valve Q1P16V593. This was identified during the Unit 2 ISI pressure test 160.28-3. Note: This was also included in 2R27 OAR-1 Report, since it was identified during Unit 2 pressure test on common piping.	CR 10742390 documents the evaluation for operability, reference Prompt Determination of Operability (PDO) 2-20-01. The flaw was evaluated in accordance with Code Case N-513-4, reference RER SNC1121545-01, and the impacted piping was later replaced by work order SNC1126770 and SNC1121637.
D-B D2.10	A through-wall leak of less than 1 drop per minute was discovered between the 1B Component Cooling Water (CCW) Heat Exchanger (HX) and Service Water vent valve Q1P16V004B. The through-wall indication is approximately 2" above the socket weld where the line attaches to the heat exchanger.	CR 10777229 documents the evaluation for operability. The piping was replaced by work order SNC1144530.
E-A E1.30	During the Containment Moisture Barrier visual exam, a small section of concrete to concrete interface was found to have degraded sealant. This approximately 12" section is located on the 105' outer circumference of the bio-wall near AZ 15.	CR 10785469 documents the condition, Indication Notification Form (INF) I-21-F1-001 documents the evaluation, and WO SNC1091821 repaired moisture barrier sealant.

F-A F1.40	During the visual inspection for support D170239-SW 501A, SNC877640-340, (Unit 1 B-train Service Water Strainer Q1P16F501A), one anchor bolt (stud/nut) was found to have corrosion and loss of cross-sectional area greater than 10%. The three remaining anchor bolts are acceptable.	CR 10790278 documents the condition and evaluation along with INF I-21-F1-002. The anchor bolt still maintains thread engagement and structural integrity which is sufficient to maintain the full load bearing capacity of the bolt and support and is acceptable per IWF-3410(b)(5) - Roughness or general corrosion which does not reduce the load bearing capacity of the support.
F-A F1.20C	Replacement 32" Main Steam Loop 1 spring can piping support ALA2-4100-MS3-H6 hot load setting was found out of adjustment. The main steam spring can MS3-H6 was replaced in 1R30, The hot load setting was found to be out of tolerance during the NOPT walkdown.	CR 10790795 documents the condition and INF I-21-F1-003 and RER SNC1152832-01 document the evaluation. The hot load settings were not within the acceptance standards in accordance with ASME Section XI, IWF-3410(a)(4). This condition was evaluated by design engineering in accordance with IWF-3112.3 and is acceptable without corrective actions.

Table 2
Farley Nuclear Plant Unit 1, 5th Interval, 2nd Period, 1st Report
Abstract of Repair / Replacement Activities Required for Continued Service

Code Class	Item Description	Description of Work	Date Completed	Repair / Replacement Plan Number
2	1" Balancing Line piping to flange weld leak from the 1C Charging Pump casing.	Replaced piping and weld due to leak.	1/16/2020	SNC1070049
3	8" Service Water return piping from the 2B Diesel Generator near the flange upstream of valve Q1P16V593. Note: This was also included in 2R27 OAR-1 Report, since it was identified during Unit 2 pressure test on common piping.	Replaced section of leaking Service Water piping.	11/6/2020	SNC1126770
3	3/4" Service Water inlet vent piping to the 1B CCW Heat Exchanger.	Replaced section of leaking Service Water piping.	2/26/2021	SNC1144530
3	1C CCW Heat Exchanger tube.	The 1C CCW Heat Exchanger affected leaking tubes were plugged.	1/25/2021	SNC1121067
2	32" Main Steam Loop 1 spring can piping support ALA2-4100-MS3-H6	Replaced east and west spring cans due to repetitively out of design specification tolerance.	4/19/2021	SNC1056981
3	6" Service Water piping of the 1C Service Water pump mini-flow line on an elbow in the horizontal section of piping that ties into the main Service Water header between Q1P16V506 and Q1P16V507 and is upstream of the mini-flow isolation valve Q1P16V575.	Replaced section of leaking Service Water piping.	4/20/2021	SNC930301
CC	12" degraded sealant in concrete to concrete moisture barrier.	Repaired moisture barrier sealant.	4/11/2021	SNC1091821