

OPERATING LICENSE AMENDMENT INDEX

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5	1013	License Condition 2.C.(3)(a)	Provided a Nov. 30, 1985, deadline for the environmental qualification of electrical equipment important to safety.
6	1014	Section 4.6.1.6.1	Delay of six months in the due dates of the first three tendon surveillances.
7	1006	Figure 6.2-1 Figure 6.2-2	Revision to the offsite and unit organizational charts.
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9	1021	Section 3/4.8.4.1 Bases 3/4.8.4	Deletes the requirement for resistance testing of certain fuses. Deletes the list of containment penetration conductor overcurrent protective devices. Deletes Table 3.8-1.
10	1017	Figures 6.2-2	Revise organizational chart - title change from Supervisor, Radwaste, to Superintendent, Radwaste.
11	1018	Section 4.7.10.1.2	Allows the 18-month inspection of a fire pump diesel engine to be performed when the plant is at power, as well as when shutdown.
12	1028	Figure 3.9-1 Section 5.3.1 Section 5.6.1.1	Increases the permitted enrichment of fuel to be stored in spent fuel pool. Permits the storage of OFA in pool. Revises nominal center-to-center distance between fuel assemblies in storage racks.
13	1029	Section 3/4.6.1	Containment leak rate testing to be performed at the calculated peak containment internal pressure.
14	1035	Figure 6.2-1 Figure 6.2-2 Section 6.5.1.2	Organization changes - QA nuclear associated with new corporate Quality Systems Department.
15	1036	1.0 Definitions Figure 2.1-1 Table 2.2-1 Bases 2.1 Bases 2.2 Section 3.1.3.4 Figure 3.1-1 Section 3/4.2.3 Section 3/4.2.4 Section 3/4.2.5 Section 4.10.2.2 Bases 3/4.2 Bases 3/4.4	Transition from low-parasitic fuel assembly fueled core to optimized fuel assembly fuel core.

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17	1027	Table 3.3-1 Table 4.3-1 Bases 3/4.3	Modified the surveillance interval for analog channel operational tests and the allowable out-of-service time of reactor trip system instrumentation.
18	1030	Table 3.3-5 Table 3.6-1 Section 3/4.7.1.6 Bases 3/4.7.1.6	Modify response times pertaining to the MSIV and the main FW isolation valves by removing the valve closure time to a separate specification.
19	1041	Table 3.3-1 Table 4.3-1	Require 48 hour restoration in the event of loss of one of the diverse reactor trip features (undervoltage or shunt trip attachment).
20	1050	Table 3.3-6 Table 3.3-13 Bases 3/4.9	Deletes the trip functions of the containment atmosphere radiation monitors associated with containment purge isolation and control room ventilation.
21	1025	Section 3/4.8.1.1 Bases 3/4.8	Increase overall emergency D/G reliability and prevent undue stress and wear on the D/G engines.
22	1081	Table 3.3-5 Bases 3/4.3	Increases ESF response time by 15 seconds for: Containment Pressure-High1, SI; Pressurizer Pressure-Low, SI; Steam Line Pressure-Low, SI.
23	1055	Figure 3.9-1 Section 5.3.1 Section 5.6.1.1	Curves for Westinghouse optimized fuel (OFA) and standard fuel (SFA). States a maximum enrichment of 4.25 weight percent U-235 for fuel storage.

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24	1059	Section 5.3.1	Allows for limited replacement of fuel rods with filler rods or vacancies if supported by a cycle-specific reload analysis.
25	1053	Section 3.7.5	Regards ultimate heat sink - two separate trains of cooling is available for the essential service water system.
26	1048	Table 3.3-3	Regards blocking of aux. feedwater start signals during normal plant shutdowns and startups.
27	1051	Section 6.9.1.5 Section 6.9.1.6 Section 3.4.8 Bases 3/4.4.8	Revises requirements for reporting iodine spiking. Eliminates the requirement to shut down the plant after 800 hours of operation with a dose equivalent I-131 value of 1 micro curie/gram or greater.
28	1057	1.0 Definitions Figure 2.1-1 Table 2.2-1 Bases 2.1 Bases 2.2.1 Section 3.1.3.4 Section 3/4.2 Table 3.2-1 Table 4.3-1 Section 3.5.1 Bases 3/4.2 Section 6.9.1.9 Section 6.10.1	Transition from low-parasitic fuel assembly and optimized fuel assembly fuel core to Vantage 5 (V-5) fuel assembly fueled core.
29	1063	Section 3/4.1.3.5 Figure 3.1-1 Bases 3/4.1.3	Revises definition of the fully withdrawn shutdown and control rod position from 228 steps to 225 steps or higher.

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30	1054	Section 3/4.3.3.7 Section 3/4.7.10 Section 3/4.7.11 Bases 3/4.3.3.7 Bases 3/4.7.10 Bases 3/4.7.11 Section 6.5.1.6 Section 6.5.1.7 Section 6.8.1	Remove fire protection T/S and place them in plant procedures and in the FSAR.
31	1065	Table 4.3-3 Section 4.11.2.5	Corrects administrative oversight that references the wrong specification for Table 3.1-13.
32	1071	Table 6.2-1 Table 6.2-2 Section 6.5.1.2	Reflects nuclear function organizational changes.
33	1034	Table 3.6-1	Increases the maximum isolation time for the containment mini-purge isolation valves.
34	1067	Table 4.3-1	Addition of functional testing requirements for the reactor trip bypass breakers.
35	1058	1.0 Definitions Figure 2.1-1 Table 2.2-1	Increase in the authorized core thermal power.
36	1064	Figure 3.4-2 Figure 3.4-3 Table 4.4-5 Figure 3.4-4 Bases 3/4.4.9	Revises the plant heatup and cooldown curves, revises the maximum allowable PORV setpoint curve, and revises the reactor vessel surveillance capsule removal schedule.
37	1066	Section 6.2	Deletes the offsite and unit organizational charts. Adds the aspects of the organizational structure which were previously defined by the organizational charts.

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38	NA	License Condition 2.E	Requires compliance with the amended Physical Security Plan which conforms to the requirements of 10CFR73.55.
39	1075	Section 6.1.2 Section 6.2.1 Section 6.5.1.6 Section 6.5.1.7 Section 6.5.1.8 Section 6.5.2.1 Section 6.5.2.2 Section 6.5.2.9 Section 6.5.2.10 Section 6.6.1 Section 6.7.1	Changes the T/S to reflect recent organizational changes. All references regarding the "Vice President, Nuclear" are changed to the "Senior Vice President, Nuclear". The position of General Manager, Engineering (Nuclear) has been deleted from the Nuclear Safety Review Board (NSRB) and the Manager, Licensing and Fuels, has been appointed Chairman of the NSRB.
40	1073	Section 3/4.5.1 Bases 3/4.5.1	Allows the plant to remain in Hot Standby with RCS pressure less than or equal to 1000 psig with one accumulator inoperable, and to allow closing one accumulator isolation valve for up to 2 hours to perform leakage testing of system check valves.
41	1082	Section 5.3.2	Revises "Control Rod Assemblies" to allow the use of hafnium and/or silver-indium-cadmium as the absorber material in the rod cluster control assemblies.

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42	1081	Section 4.4.9.3 Section 4.5.2 Section 3.5.4 Section 4.9.8.1 Section 3/4.9.8.2 Bases 3/4.4.9 Bases 3/4.5.2 Bases 3/4.5.3 Bases 3/4.5.4 Bases 3/4.5.5 Bases 3/4.9.8	Reduces the required RHR system flowrate during Mode 6 operation, deletes the RHR autoclosure interlock (ACI) function, and allows the safety injection pumps to be energized with the head on and with water level not above the top of the reactor vessel flange, in Modes 5 and 6.
43	1074	Table 2.2-1 Table 3.3-1 Table 3.3-2 Table 4.3-1 Table 3.3-3 Table 3.3-4 Table 3.3-5 Table 4.3-2 Bases 2.0	Revises the provisions in the T/S relating to the steam generator low-low level trip circuitry by adding an Environmental Allowance Modifier (AM) and a Trip Time Delay (TTD).
44	1079	Bases 2.1.1 Section 3.1.1.3 Section 3.1.2.5 Section 3.1.2.6 Section 3.2.1 Section 4.2.1.4 Section 3.2.2 Figure 3.2-2 Section 4.2.2.2 Section 4.2.2.3 Section 4.2.2.4 Section 3.2.3 Section 3.5.1 Section 3.5.5 Section 3.6.2.2 Bases 3/4.1.2 Bases 3/4.2.1 Bases 3/4.2.2 Bases 3/4.2.3 Bases 3/4.5.5	Revises the T/S to support a core reload for Cycle 4. The revised T/S includes increased peaking factors, a positive moderator temperature coefficient, increased refueling water storage tank and accumulator boron concentrations, and increased spray additive tank sodium hydroxide concentration.

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45	1039	Section 4.3.3.5.3 Section 3/4.7.1.7 Bases 3/4.7.1.7	Incorporates new T/S LCO and surveillance requirements for the steam generator Atmospheric Steam Dumps (ASDs) into the Callaway License in order to assume the availability of mitigating equipment assumed in the steam generator tube rupture analysis.
46	1024	Section 3/4.6.1.6	Provides clarification and relaxation of some of the existing Surveillance Requirements, adds provisions vital for monitoring the integrity or prestressed concrete containments, and modifies some of the reporting requirements and action statements found within the LCO.
47	1060	Section 3/4.7.6	Increases the allowed flow variations of the control room emergency ventilation system and reduces the control room pressurization requirement from 1/4 inch water gauge to 1/8 inch water gauge.
48	1069	Figure 5.1-1 Figure 5.1-2 Figure 5.1-3 Figure 5.1-4	Revises the referenced figures regarding the exclusion area, low population zone, site boundary for gaseous effluents, and site boundary for liquid effluents.
49	1078	Section 3/4.3.3	Revises Radiation Monitoring for Plant Operation, by increasing the permitted period of inoperability for one channel of the control room air intake monitors and fuel building atmosphere monitors from 1 to 72 hours.

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50	1087	Section 1.0 Section 3/4.3.10 Section 3/4.3.11 Section 3/4.3.12 Bases 3/4.3.10 Bases 3/4.3.11 Bases 3/4.3.12 Section 6.8 Section 6.9 Section 6.10 Section 6.11 Section 6.12 Section 6.13 Section 6.14	Relocates the existing procedural details of the current Radiological Effluent Technical Specifications (RETS) to the Offsite Dose Calculation Manual (ODCM), and procedural details for the solid radioactive wastes to the Process Control Program (PCP). The amendment also incorporates the respective programmatic controls into the Administrative Controls section of the Technical Specifications.
51	1085	Section 3/4.1.3 Bases 3/4.1.3	Allows continued operation for 72 hours for diagnosis and repair, with one or more control rod assemblies inoperable due to a rod control urgent failure alarm or other electrical problem in the rod control system provided all affected control rods remain trippable.
52	1090	Section 4.0.2 Bases 4.0.2	Removes the 3.25 limit for surveillances as provided in Generic Letter 89-14 dated August 21, 1989.
53	1088	Section 3/4.7.1.2	Adds clarification to Surveillance Requirements 4.7.1.2.1.a(4) and 4.7.1.2.1.b(1) by identifying automatic valves that are either excluded or included in the flowpath of the Auxiliary Feedwater System whose position has to be verified to demonstrate operability.
54	1093	Section 5.3.1 Section 5.6.1.1	Revises T/S Sections 5.3.1 and 5.6.1.1 to allow fuel with a maximum initial enrichment of 4.45 weight percent to be stored in Region 1 of the Callaway Plant spent fuel pool.

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55	1095	Section 3.7.1.2	Modifies the Limiting Condition of Operation to address flowpath requirements of the motor-driven and steam turbine-driven auxiliary feedwater pumps. Additionally, it provides further requirements to the ACTION Statements if any of the Essential Service Water (ESW) system valves to the steam turbine-driven auxiliary feedwater pumps are inoperable or if one ESW loop is inoperable.
56	1096	Table 2.2-1 Table 3.3-1 Table 3.3-2 Table 4.3-1 Bases 2.2.1	Revises T/S by deleting the power range, neutron flux, high negative rate trip.
57	1099	Table 2.2-1 Table 3.3-4 Table 4.3-1	Revises T/S and associated bases to accommodate the replacement of the current Resistance Temperature Detector (RTD) bypass system with an RTD/thermowell system mounted directly into the hot and cold legs of the reactor coolant system.
58	1091	Section 1.0 Section 3/4.1.1.3 Section 3/4.1.3.5 Section 3/4.1.3.6 Section 3/4.2.1 Section 3/4.2.2 Section 3/4.2.3 Bases 3/4.1.1.3 Bases 3/4.2.1 Bases 3/4.2.2 Bases 3/4.2.3 Section 6.9.1.9	Relocates cycle-specific core parameters from the T/S and places them in the Core Operating Limits Report (COLR).

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59	1105	Section 3/4.7.1.7	Revises T/S to allow an exception to T/S 4.0.4 thus allowing sufficient plant conditions (Mode 3) to be established and completion of the required surveillance prior to applying the T/S 3/4.7.1.7 Action Statement on atmospheric steam dump valve (ASD) operability.
60	1072	Section 6.3 Section 6.4	Revised T/S by deleting references superseded by the May 26, 1987 revision to 10CFR55.
61	1102	Section 3/4.1.3.2 Bases 3/4.1.3.2	Revises T/S to add an action statement covering situations where more than one digital rod position indicator per bank is inoperable.
62	1089	Section 3/4.6.1.1	Revises T/S to clarify which containment penetration methods are acceptable.
63	1101	Section 6.5.1.2	Revises T/S to delete the specific title designations of seven of the eight members of the On-Site Review Committee (ORC).
64	1106	Table 3.3-1 Table 4.3-1 Table 3.3-3 Table 4.3-2 Bases 3/4.3.1 Bases 3/4.3.2	The amendment extends the allowable out-of-service times (AOTs) and the surveillance test intervals for the analog channels of the Engineered Safety Features Actuation System (ESFAS). These revisions also include extended AOTs for the ESFAS actuation logic and actuation relays of the solid state protection system.

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65	1123	Bases 3/4.8.1 Bases 3/4.8.2 Bases 3/4.8.3 Section 4.8.1.1.2.f.(2)	Revises Surveillance 4.8.1.1.2.f.(2) concerning diesel generator load reject testing by removing the reference to a numerical value for the largest single emergency load. This change will allow the current surveillance test to conform with the literal requirements of Technical Specifications.
66	1107	Table 3-4.1 Bases 3/4.4.6.2 Section 3/4.6.2.f	Revises Technical Specification 3/4.4.6 and associated Bases to change the allowed leakage limit for reactor coolant system pressure isolation valves.
67	1111	Section 3/4.7.8 Table 4.7-2 Bases 3/4.7.8	Revises T/S 3/4.7.8, "Snubbers", and associated Bases by changing the snubber visual inspection intervals and corrective actions. The proposed changes are consistent with the guidance of Generic Letter 90-09 dated 12/11/90.
68	1109	Section 3/4.5.2.h	Operating License Amendment 68 revises T/S 4.5.2.h to reflect new surveillance requirements for maximum and minimum Emergency Core Cooling System (ECCS) subsystem flow rates. In addition, the surveillance requirements for performing flow balance tests on specific ECCS subsystems are clarified. These changes will ensure that the performance of these ECCS subsystems is demonstrated to be consistent with the current plant safety analyses.

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69	1122	Section 3/4.3.2 Section 3/4.7.6	Amendment 69 revises T/S 3/4.3.2 and 3/4.7.6 to take exception to T/S 3.0.4 which prevents entry into an operational mode unless the conditions for the Limiting Condition for Operation are met. The revision allows operational mode changes in MODES 5 and 6 while certain control room ventilation T/S action statements are in effect.
70	1117	Section 3/4.7.12 Table 3.7-4 Bases 3/4.7.12	Amendment 70 revises T/S 3/4.7.12, Table 3.7-4 and associated Bases, "Area Temperature Monitoring", to increase the maximum temperature limit from 101°F to 106°F for the Electrical Penetration Rooms. The proposed change is consistent with the intent of the T/S because the absolute maximum temperature limit for the Electrical Penetration Rooms is maintained at 131°F.
71	1121	Table 6.2-1	Amendment 71 revises T/S Table 6.2-1 to permit an individual with a valid Senior Reactor Operator ("SRO") license and who is qualified as a Shift Technical Advisor ("STA") to assume the Control Room command function during an absence of the Shift Supervisor ("SS") from the Control Room.
72	1120	Section 3/4.2.1 Section 4.2.2.2 Section 4.2.2.3 Section 4.2.2.4 Section 6.9.1.9 Table 2.2.1 Bases 3/4.2.1	Amendment 72 revises T/S in order to implement relaxed axial offset control (RAOC) for Cycle 6 at Callaway. The RAOC methodology supporting this proposed change is consistent with WCAP-10216-PA which has been previously reviewed and approved by the NRC.

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73	1134	Section 4.3.1.1 Table 4.3-1	Amendment 73 grants a one-time only extension to the surveillance interval specified for Technical Specification (TS) Surveillance 4.3.1.1, Table 4.3-1, Functional Unit 1 (Manual Trip) by adding a footnote to Note 16 which states that "complete verification of OPERABILITY of the manual reactor trip switch circuitry shall be performed prior to startup from the first shutdown to Mode 3 occurring after August 7, 1992." This amendment was required due to the discovery that the existing surveillance procedure does not adequately verify the operability of the shunt trip contacts associated with the manual reactor trip function.
74	1119	Table 3.3-4	Amendment 74 revises Technical Specification Table 3.3-4, "Engineered Safety Features Actuation System Instrumentation Trip Setpoint," Functional Unit 8.b, to revise the trip setpoint, the allowable value, total allowance, sensor error, and "Z" value for the "4 kV Undervoltage-Grid Degraded Voltage" protection function to agree with the required design values.

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75	1089	Section 3.6.1.2 Section 4.6.1.2.b Bases 3/4.6.1.2	Amendment 75 revises the ACTION statement in T/S 3.6.1.2 and the surveillance criteria in T/S 4.6.1.2.b and associated bases to establish two conditions for determining the acceptability of the periodic Type A tests conducted pursuant to Appendix J to 10CFR50. These conditions are the "as found" and the "as left" conditions; each has separate acceptance criteria. The amendment also revises T/S 4.6.1.2a to eliminate the requirement to conduct the third test of each set of three Type A tests during the shutdown for the 10 year plant inservice inspection.
76	1118	Section 4.4.9.1.2 Figures 3.4-2 Figures 3.4-3 Figures 3.4-5 Tables 4.4-5 Tables B 3/4.4-1 Bases 3/4.4.9	Amendment 76 revises Technical Specification Section 4.4.9.1.2, Figures 3.4-2, 3.4-3, and 3.4-4, Tables 4.4-5 and B 3/4.4-1 and associated Bases for Reactor Coolant System Pressure/Temperature Limits by modifying plant heatup and cooldown curves and the maximum allowable PORV setpoint curve for cold overpressure protection.

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77	1129	Section 4.6.1.2.a Bases 3/4.6.1.2	Amendment 77 changes the Technical Specifications by extending the surveillance requirements of Technical Specification 4.6.1.2.a and its associated Bases to allow the third Type A Containment Leakage Rate Tests within the first 10-year service period to be conducted during the Cycle 7 refueling outage. This Technical Specification change is a one-time extension of the current maximum interval from 50 months to 53 months and is consistent with a one-time exemption from Appendix J to 10 CFR Part 50, being issued concurrently under separate cover, which extends the first 10-year service period by approximately three months.
78	1068	Section 4.0.3 Section 4.0.4 Bases 4.0.3 Bases 4.0.4	Amendment 78 changes the Technical Specifications 4.0.3, 4.0.4 and their associated bases to incorporate changes as recommended in Generic Letter 87-09, "SECTIONS 3.0 AND 4.0 OF THE STANDARD TECHNICAL SPECIFICATIONS (STS) ON THE APPLICABILITY OF LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS," dated June 4, 1990.
79	1139	Section 4.7.8.d	Amendment 79 revises Technical Specification Surveillance Requirement 4.7.8.d to allow a one-time schedule extension of the snubber transient event inspection.

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80	1137	Section 4.8.1.1.2.h(2)	Amendment 80 deletes the surveillance requirement of Technical Specification 4.8.1.1.2.h(2), which involves performing a pressure test of portions of the diesel fuel oil system. Alternate testing for the tanks and piping would include leak testing at hydrostatic head pressure with the tanks filled to design capacity and would be governed by Technical Specification 4.0.5.
81	1128	Section 3.9.7 Bases 3/4.9.7	Amendment 81 revises Technical Specification 3.9.7 and associated Bases to allow the spent fuel pool transfer gates to travel over fuel assemblies in the spent fuel pool for refueling activities, fuel handling system maintenance, and transfer gate seal replacement.
82	1132	Section 3/4.9.12, Figure 3.9-1	Amendment 82 revises Technical Specification 3/4.9.12, Figure 3.9-1 to reflect a maximum initial enrichment of 4.45 w/o U-235 for fuel storage in Region 2 of the Callaway spent fuel pool.
83	1108	Section 3/4.4.4 Section 3.4.9.3 Bases 3/4.4.4	Amendment 83 revises Technical Specifications 3/4.4.4 and its associated bases and 3/4.4.9.3 to address changes as recommended in Generic Letter 90-06, "RESOLUTION OF GENERIC ISSUE 70, 'POWER-OPERATED RELIEF VALVE AND BLOCK VALVE RELIABILITY,' AND GENERIC ISSUE 94, 'ADDITIONAL LOW-TEMPERATURE OVERPRESSURE PROTECTION FOR LIGHT-WATER REACTORS,' PURSUANT TO 10 CFR 50.54(f)" dated June 25, 1990.

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84	1141	Tables 2.2-1 Tables 4.3-1 Bases 3/4.2.2 Bases 3/4.2.3	Amendment 84 revises Technical Specification Tables 2.2-1 and 4.3-1 and associated Bases 3/4.2.2 and 3/4.2.3 by changing the axial flux difference (AFD) penalty function f_1 ($\Delta - I$) defined in Note 1 of Table 2.2-1 for the Overtemperature Delta-T reactor trip.
85	1130	Section 3.3.2 Section 3/4.8.1	Amendment 85 revises Technical Specification 3.3.2 and 3/4.8.1 to add Mode 5 and 6 applicability for OPERABILITY of the load sequencer and supplying 4 KV Bus undervoltage circuits.
86	1104	Table 3.3-4 Table 3.3-10 Section 3.3.3.6 Section 3.6.4.1 Section 3.11.2.5 Section 6.2.2	Amendment 86 revises various Technical Specifications to reflect editorial and typographical changes for consistency and clarity. The changes include trip setpoint corrections, description clarifications, reference updates, and title deletions.
87	1124	Table 4.3-1	Amendment 87 revises Technical Specification Table 4.3-1, Note 5, to reflect that integral bias curves, rather than detector plateau curves, are used to calibrate the source range instrumentation. Intermediate Range and Power Range channels will continue to be calibrated using detector plateau curves.
88	1125	Section 6.5	Amendment 88 revises Section 6 of the Technical Specifications to reflect an organizational change to the administrative controls. The change involves the title change of the General Manager, Nuclear Operations to Vice President, Nuclear Operations.

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89	1143	Section 1.18 Section 3.11.1.4 Section 3.11.2.6 Section 6.9.1.7 Section 6.14	Amendment 89 changes the submittal frequency for the Radioactive Effluent Release Report from semi-annually to annually. This change also makes the report date for Radioactive Effluent Release Report consistent with the reporting requirements for the Annual Radiological Environmental Operating Report.
90	1126	N/A	Amendment 90 deletes Sections 2.3 and 4.3, Cultural Resources, of Appendix B, Environmental Protection Plan.
91	1150	Section 3/4.5.1 Bases 3/4.5.1	Amendment 91 (tracked as O.L. 1150) revises Technical Specification 3/4.5.1 and associated Bases Section 3/4.5.1. A new Action Statement a. provides a 72-hour allowed outage time (AOT) for one accumulator inoperable due to boron concentration. The Action Statement b. AOT was changed to 24 hours. Surveillance Requirements 4.5.1.1.a.1 and 4.5.1.1.b were revised and 4.5.1.2 was deleted from the T/S.
92	1136	Section 6.3.1.2	Amendment 92 (tracked as O.L. #1136) changes Technical Specification 6.3.1.2 to allow either the Health Physics Superintendent or the Health Physics, Operations Supervisor to be designated as Radiation Protection Manager (RPM).
93	1146	Table 2.2-1 Bases 2.2.1	Amendment 93 (tracked as O.L. #1146), "Undervoltage-Reactor Coolant Pumps," revised to correct total allowance (undervoltage relay span) and allowable value (voltage) expressions.

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Revision by NRC letter of 10/28/94	NA	Bases 2.2.1	This letter of 10/28/94 (see ULNRC-02672) addresses OT Delta-T, OP Delta-T, and Delta-T calibrations at the beginning of each cycle during startup testing. The revisions reflected that, for the startup of a refueled core, until measured at 100% RTP, Delta T _o is initially assumed at a value, which is conservatively lower than the last measured 100% RTP Delta T _o for each loop.
94	1127	Table 3.3-1 Table 4.3-1 Bases B 2-8 Bases B 3/4 4-1	Amendment 94 (tracked as O.L. #1127) revises Technical Specification Table 3.3-1 and 4.3-1, and Bases pages B 2-8 and B 3/4 4-1. The changes reflect the re-analysis of the boron dilution transient for shutdown modes to address non-conservatism in the previous event analysis.
95	1163	Section 4.8.1.1.2f.7	Amendment 95 (tracked as O.L. #1163) revises Technical Specification (TS) Surveillance Requirement 4.8.1.1.2f.7. The change removes the requirement to perform the hot restart test within 5 minutes of completing the 24-hour endurance test and places that requirement in a separate TS.
96	1152	Section 3/4.6.2.2	This amendment replaces Technical Specification (TS) 3/4.6.2.2, Spray Additive System, with a new TS 3/4.6.2.2 entitled Recirculation Fluid pH control (RFPC) System.

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97	1159	Bases 3/4.9	<p>The amendment revises Bases 3/4.9, “Refueling Operations” and FSAR Sections 9.1.3. “Fuel Pool Cooling and Cleanup,” 9.1.4 “Fuel Handling System” and 15.4.6 “Chemical and Volume Control System Malfunction that result in a Decrease in the Boron Concentration in the Reactor Coolant.” The changes document the results of a safety evaluation that considers the effects using reactor water makeup to spray down the refueling pool during pool drain evolutions. The changes establish procedural controls to address the possibility of a different type of boron dilution event than previously considered.</p>
98	1164	Section 4.6.1.2.a Bases 3/4.6.1.2	<p>This amendment revises T/S Surveillance Requirement 4.6.1.2.a and its associated Bases. The change defers the requirement to perform the Type A Containment Integrated Leak Rate Test until Refuel 8 (October 1996), in conjunction with the exemption to 10 CFR Part 50, Appendix J.</p>
99	1158	Section 3/4.8.2.1 Section 3/4.8.2.2 Section 3/4.8.3.1 Section 3/4.8.3.2	<p>This amendment revises T/S 3/4.8.2.1, 3/4.8.2.2, 3/4.8.3.1, and 3/4.8.3.2. The changes address the 125-volt DC busses, adds provisions for swing battery chargers, and removes provisions for the 4160-volt and 480-volt AC emergency busses.</p>
100	1110	Section 3/4.8.1.1 Section 3/4.8.1.2	<p>This amendment revises T/S 3/4.8.1.1 and 3/4.8.1.2 to address the minimum required storage volumes of the Emergency Fuel Oil storage and day tanks.</p>

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101	1154	Section 4.2.2.2 Section 4.2.2.4 Section 6.9.19	Revises T/S 4.2.2.2, 4.2.2.4, and 6.9.19. The changes incorporate a penalty in the Core Operating Limits Report (COLR) to account for heat flux (F_Q) increases greater than 2 percent between measurements.
102	1167	Section 2.2.1 Table 2.2-1	Amendment 102 (tracked as OL 1167) revises Technical Specification 2.2.1, Table 2.2-1. The changes address reducing repeated alarms and partial reactor trips by revising the Overpower Delta-T (OP Δ T) setpoint function.
103	1151	Various. Refer to ULNRC's 3023 and 3184.	This amendment revises T/S to implement the NRC's Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors.
104	1165	Section 3.3.1 Table 3.3-2 Table 3.3-5	Amendment 105 revises T/S 3.3.1 and T/S 3.3.2 to relocate T/S Tables 3.3-2 and 3.3-5 to FSAR Table 16.3-1 and FSAR Table 16.3-2 per FSAR CN 95-07, which provide the response time limits for the reactor trip system (RTS) and the engineered safety features actuation system (ESFAS) instruments. The amendment also relocates the Bases discussion for T/S 3.3.1. and T/S 3.3.2 to Section 16.3 of the updated FSAR.

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105	1169	Section 4.1.1.1.1 Section 4.1.1.2 Section 4.1.3.1.2 Section 4.4.6.2.2.b Section 4.4.3.2 Section 4.6.2.1.d Section 4.6.4.2 Table 4.3-3 Bases 3/4.1.3	This amendment revises T/S 4.1.3.1.2, 4.4.6.2.2.b, 4.4.3.2, 4.6.2.1.d, 4.6.4.2, and Table 4.3-3 to implement the recommendations of NRC Generic Letter 93-05, "Line Item Technical Specification Improvements to Reduce Surveillance Requirements for Testing During Power Operations." Additionally, the amendment revises T/S 4.1.1.1.1, 4.1.1.2, 3/4.1.3.1 and the associated Bases to implement portions of NUREG-1431, "Standard Technical Specifications - Westinghouse Plants."
106	1168	Section 3/4.7.6	Amendment 106 revises T/S 3/4.7.6 to reduce the upper limit on the flow rate through the control room filtration subsystem and adopts ASTM D-3803-1989 as the laboratory testing standard for control room filtration and control building pressurization charcoal adsorber. The amendment also revises the Bases for T/S 3/4.7.6 to reflect the changes.
107	1156	Section 6.2.3 Section 6.5.1 Section 6.5.2 Section 6.5.3	The amendment revises T/S 6.5.1, 6.5.2, and 6.5.3 to relocate the review and audit requirements of the On-Site Review Committee (ORC) and the Nuclear Safety Review Board (NSRB) to the Operational Quality Assurance Manual (OQAM). In addition, the amendment deletes reference to the Manager, Nuclear Safety and Emergency Preparedness, in T/S 6.2.3.

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Revision by NRC letter dated 2/1/ 96	NA	Bases 2.1.1 Section 3/4.3.3.6 Section 3/4.6.1.4 Section 3/4.1.1.3 Section 3/4.9.4	Bases 2.1.1, Safety Limits Reactor Core, Bases 3/4.3.3.6, Accident Monitoring Instrumentation, Bases 3/4.6.1.4, Internal Pressure (all tracked by O.L. 1116); Bases 3/4.1.1.3, Moderator Temperature Coefficient (tracked by O.L. 1140); and Bases 3/4.9.4, Containment Building Penetrations (tracked by O.L. 1148).
108	1162	Section 4.3.2.2 Section 4.6.1.2.1 Bases 3/4.7.1.2	Amendment 108 (tracked as OL 1162) revises TS 4.3.2.2, TS 4.7.1.2.1, and the Bases for TS 3/4.7.1.2 to decrease the frequency of auxiliary feedwater pump testing, remove inconsistencies in testing requirements for the turbine-driven auxiliary feedwater pump, and clarify performance parameters in the TS Bases.
109	1171	Section 5.3.1 Section 5.6.1.1 Figure 3.9-1	Amendment 109 (tracked as OL 1171) revises TS 5.3.1 to reflect a change in the maximum initial enrichment for reload fuel, subject to the integral fuel burnable absorber (IBFA) requirements, and a change in the maximum fuel enrichment not requiring IFBAs. The amendment also changes the maximum reference k_{∞} in TS 5.6.1.1 for fuel storage in Region 1 of the spent fuel pool and revises TS Figure 3.9-1 to reflect a change to the maximum initial enrichment for fuel stored in Region 2 of the spent fuel pool.
110	1175	Section 5.3.1	Amendment 110 (tracked as OL 1175) revises TS 5.3.1 to allow the use of ZIRLO clad fuel rods and ZIRLO filler rods.

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111	1172	Section 1.7 Section 4.6.1.1 Section 3.6.1.3 Section 4.6.1.3 Section 6.8.4 Bases 3/4.6.1.1	Amendment 111 (tracked as OL 1172) revises T/S 1.7, 4.6.1.1, 3.6.1.3, 4.6.1.3, 6.8.4 and the associated Bases section to directly reference Regulatory Guide 1.163 as required by 10 CFR 50, Appendix J, Option B for the Type A containment integrated leak rate tests and the Type B and C local leak rate tests.
112	1170	Section 3/4.8.1 Bases 3/4.8	Amendment 112 revises T/S 3/4.8.1 and its associated Bases to improve overall emergency diesel generator reliability and availability.
113	1153	Section 1.7 Section 3/4.6.1 Section 3/4.6.3 Bases 3/4.6.2.3	Amendment 113 revises the allowed outage times for component cooling water motor operated containment isolation valves, removes the list of containment isolation valves from T/S, and allows containment penetration check valves to be used as isolation devices.
114	1173	Section 3/4.9.4 Bases 3/4.9.1	Amendment 114 (O.L. 1173) revises T/S 3.9.4 and its associated Bases to allow the containment personnel airlock doors to be open during core alterations and movement of irradiated fuel in containment.
115	1181	FSAR 7.1.2.5.2 FSAR 10.4.7 FSAR 10.4.7.2.2 FSAR Fig. 7.2-1 (513 & 514)	Modification to reduce the single failure trip potential for the main feedwater control and bypass valves.
116	1178	Section 3/4.4	Amendment 116 (O.L. 1178) revises T/S 3/4.4.5 and its associated Bases to address the installation of laser welded tube sleeves in the steam generators.

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117	1176	Section 3/4.3	Amendment 117 (O.L. 1176) revises T/S 3/4.3 to support a modification to replace existing analog portions of the main steam and feedwater isolation system (MSFIS) with digital processor equipment and would authorize revision of the FSAR to include a description of the MSFIS mod.
118	1182	Section 3/4.7.7 Section 3/4.9.13 Bases 3/4.7.7 Bases 3/4.9.13	Amendment 118 (O.L. 1182) revises the surveillance requirements of Technical Specifications 3/4.7.7, 3/4.9.13 and the corresponding Bases. The changes implement an updated charcoal test methodology for the emergency exhaust system.
119	1187	Table 3.3-3	Amendment 119 revises T/S Table 3.3-3 to correct administrative errors associated with the start logic of the turbine driven auxiliary feedwater pump.
120	1179	Operating License 1.A	Amendment 120 (O.L. 1179) revises Page 1 of the Operating License to incorporate Union Electric's merger with CIPS and the formation of Ameren Corporation.
121	1185	Section 3/4.7.4	Amendment 121 (O.L. 1185) revises T/S 3/4.7.4 to remove the during shutdown requirement for the ESW surveillances.
122	1188	Section 6.1 Section 6.2 Section 6.6 Section 6.7	Amendment 122 revises the title "Senior Vice President, Nuclear" to "Vice President and Chief Nuclear Officer" due to the elimination of the Vice President, Nuclear Operations position.

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123	1184	Table 3.3-3 Table 4.3-2	Correct the # of MSFIS channels listed in Table 3.3-3 for items 4.b.2 and 5.a.2 to be consistent with SSPS requirements and change Table 4.3-2 surveillance requirements for items 4.b.2 and 5.a.2 from a quarterly slave relay test to a monthly staggered actuation logic test.
124	1191	Figures 3.4-2 Figures 3.4-3 Figures 3.4-4 Bases 3/4.4.9 Bases 3/4.5.2	This amendment modifies the plant heatup and cooldown curves and the maximum allowable PORV setpoints for cold overpressure protection.
125	1174	Table 2.2-1 Table 4.3-1 Table 3.3-4 Bases 2.2.1	This amendment modifies setpoint and allowable values of certain reactor trip system (RTS) and engineered safety features actuation system (ESFAS) function units.
126	1189	Table 3.3-3 Table 3.3-4 Table 4.3-2 Bases 3/4.3.1	This amendment revises the feedwater isolation engineered safety feature actuation system (ESFAS) functions.
127	1195	Section 4.5.2b.1 Bases 3/4.5.2	This amendment eliminates the requirement to vent the centrifugal charging pump casings.
128	1186	Table 3.7-2 Table 2.2-1	This amendment revises the lift setting tolerance of the Main Steam Safety Valves. Additionally, Table 2.2-1 has been revised to reduce the sensor error for the pressurizer pressure-high trip.
129	1196	Section 3/4.9.12 Bases 3/4.9.12 Section 5.3 Section 5.6	Spent Fuel Pool Rerack Amendment.

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130	1193	Table 3.3-3 Table 3.3-4 Table 4.3-2	This amendment revises the engineered safety features actuation system (ESFAS) functional units 6.f, "Loss of Offsite Power – Start Turbine Driven Pump" and 8.a and 8.b "Loss of Power".
131	1197	Section 3/4 7-9b Section B 3/4 7-3 Section B 3/4 7-3a	This amendment revises operability requirements to require four atmospheric steam dump (ASD) lines to be operable.
132	1180	Section 4.4.5.4 Section 4.4.5.5 Table 4.4-3 Bases 3/4.4.5	This amendment allows the repair of Callaway Plant, Unit 1 steam generator tubes with the Electrosleeve tube repair method.
133	1177	All Sections	This amendment converted the Callaway Unit 1 Technical Specifications to the Improved Technical Specifications
134	1203	Section 5.6.6	This amendment revises Section 5.6.6, "Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)," of the ITSs that were issued in amendment 133.
135	1204	TOC 1.1 Definition (ACTIONS) (STAGGERED TEST BASIS) EXAMPLE 1.3-6 ACTION 3.4.15.B ACTION 3.7.1.B ACTION 3.7.2.D ACTION 3.7.4.D LCO 3.7.13 ACTION 3.7.16.A ACTION 3.8.5.A Section 5.6.5.a.7	This amendment corrected 14 errors in the ITSs that were issued in amendment 133.

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136	1198	LCO 3.7.1 ACTION 3.7.1	This amendment revises LCO 3.7.1 in that the maximum allowable reactor power for a given number of operable MSSVs per steam generator is reduced in Table 3.7.1-1 and Action A.1. Additionally, two format errors in the actions for LCO 3.7.1 are corrected.
137	1206	ACTION 3.3.2.H SR 3.3.2.14 Table 3.3.2-1 LCO 3.4.10 SR 3.4.10.1 CONDITION 3.4.11.A CONDITION 3.4.11.B	This amendment expanded the range of acceptable lift settings and tolerance of the as-found, measured lift settings for the pressurizer safety valves (PSVs) to be considered operable. Following testing, the as-left lift settings of the PSVs would remain at the current tolerance of +/- 1%, but the nominal lift setting would be reduced.
138	1209	LCO 3.9.4 NOTE COMPLETION TIME 3.8.3.E.2	This amendment allows containment penetrations with direct access to the outside atmosphere to be open under administrative controls during refueling operations, by adding a note to the LCO 3.9.4. In addition, there was a format and editorial correction to TS 3.8.3 to correct an error in the conversion to the improved TS issued in Amendment No. 133.
139	1199	FSAR Changes	This amendment request revised the description of the steam generator tube rupture and main steam line break accident in the Callaway Final Safety Analysis Report (FSAR) to reflect increases in the radiological dose consequences calculated for these accidents. These revisions were incorporated into FSAR update revision OL-12.

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140	1212	SR 3.5.2.5	This amendment added a footnote to Surveillance Requirement (SR) 3.5.2.5 that states that verification of the automatic closure function of the residual heat removal (RHR) pump suction Valve BNHV8812A shall be performed prior to startup from the first shutdown to Mode 5 occurring after September 8, 2000, but not later than June 1, 2001. The next refueling outage for Callaway is scheduled for April 2001.
141	1208	Table 3.3.2-1	This amendment added Surveillance Requirement (SR) 3.3.2.10 for the following two ESFAS instrumentation in the table: item 6.f, loss of offsite power, and item 6.h, auxiliary feedwater pump suction transfer on suction pressure-low.
142	1213	Section 5.5.14	This amendment revised “Technical Specifications (TS) Bases Control Program” to reflect the changes made to 10 CFR 50.59 as published in the Federal Register on October 4, 1999.
143	1210	FSAR Changes	This amendment authorized changes to the FSAR related to the installation of replacement engineered safety feature (ESF) transformers. This amendment allows the replacement of the ESF transformers with new transformers having active automatic load tap changers (LTCs).
144	1214	Section 5.5.3	This amendment deletes Section 5.5.3, “Post Accident Sampling,” from the Technical Specifications and thereby eliminates the requirements to have and maintain the post-accident sampling system (PASS).

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145	1215	Section 5.2.1	This amendment revised Technical Specification 5.2.1.c to replace the title "Vice President and Chief Nuclear Officer" with Senior Vice President and Chief Nuclear Officer."
146	1219	LCO 3.5.5 ACTION 3.5.5.A SR 3.5.5.1	This amendment revised Technical Specification 3.5.5 to remove the phrase "and the charging flow control valve full open" from Limiting Condition for Operation, Required Action A.1, and Surveillance Requirement 3.5.5.1 for the reactor coolant pump seal injection flow.
147	1222	SR 3.0.3	Revised Surveillance Requirement (SR) 3.0.3 to extend the delay period before entering a limiting condition for operation upon a missed SR from the current limit of "...up to 24 hours or up to the limit of the specified Frequency, whichever is less" to "...up to 24 hours or up to the limit of the specified Frequency, whichever is greater." In addition, the following requirement is added to SR 3.0.3: "A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed."
148	1211	SR 3.3.1.2 SR 3.3.1.3	The amendment revised Surveillance Requirements (SRs) 3.3.1.2 and 3.3.1.3 on reactor trip system (RTS) instrumentation. The change to SR 3.3.1.2 would replace the reference to the nuclear instrumentation system channel output with a reference to the power range channel output and would delete Note 1 to this SR. The change to SR 3.3.1.3 is editorial.

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149	1221	ACTIONS 3.3.1.G ACTIONS 3.3.1. I ACTIONS 3.3.9.B LCO 3.4.5 Note ACTIONS 3.4.5.D LCO 3.4.6 Note ACTIONS 3.4.6.B LCO 3.4.7 Notes ACTIONS 3.4.7.B LCO 3.4.8 Notes ACTIONS 3.4.8.B ACTIONS 3.8.2.B ACTIONS 3.8.8.A ACTIONS 3.8.5.A ACTIONS 3.8.10.A ACTIONS 3.9.3.A LCO 3.9.5 Note ACTIONS 3.9.5.A ACTIONS 3.9.6.B	<p>This amendment revised several of the Required Actions in the Technical Specifications that require suspension of operations involving positive reactivity additions or suspension of operations involving reactor coolant system (RCS) boron concentration reductions. In addition, the amendment revised several Limiting Condition for Operation (LCO) Notes that preclude reductions in RCS boron concentration. The Required Actions and LCO Notes were revised to allow small, controlled, safe insertions of positive reactivity, but limits the introduction of positive reactivity such that compliance with the required shutdown margin or refueling boron concentration limits will still be satisfied. This amendment is based on a NRC-approved traveler, Technical Specification Task Force (TSTF)-286, Revision 2.</p>
150	1220	LCO 3.5.5 ACTIONS 3.5.5.A SR 3.5.5.1 Figure 3.5.5-1	<p>This amendment revised Technical Specification 3.5.5 to replace the flow and differential pressure limits that were stated for the reactor coolant pump seal injection flow by limits provided in Figure 3.5.5-1, which was added to the Technical Specification.</p>
151	1231	SR 3.3.1.16 Table 3.3.1-1	<p>The amendment revised Technical Specification 3.3.1 by adding Surveillance Requirement 3.3.1.16 to Function 3 of Table 3.3.1-1. Surveillance Requirement 3.3.1.16 verifies that the reactor trip system response times are within limits every 18 months on a staggered test basis.</p>

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152	1218	ACTIONS 3.3.6.C Table 3.3.6-1 ACTIONS 3.3.7.E SR 3.3.7.6 Table 3.3.7-1 ACTIONS 3.3.8.A ACTIONS 3.3.8.B ACTIONS 3.3.8.C LCO 3.9.4.a LCO 3.9.4.b SR 3.9.4.2 SR 3.9.4.3	This amendment revised the Limiting Conditions for Operation (LCO's), Required Actions for LCO's Surveillance Requirements, and Tables specifying requirements on instrumentation in Technical Specification 3.3.6, Technical Specification 3.3.8, and Technical Specification 3.9.4. This amendment allows the equipment hatch and the emergency air lock to be open in refueling outages during core alterations and/or movement of irradiated fuel within containment.
153	1216 1235	Section 5.5.9.d.1.j)2)	The amendment revised paragraph d.1.j)2) of Technical Specification 5.5.9 to 1) delete the requirement that all steam generator tubes containing an Electrosleeve be removed from service within two operating cycles following installation; 2) added requirement that Electrosleeves cannot be installed in the outermost periphery tubes of the steam generator bundles where potentially locked tubes would cause high axial loads; 3) revised references describing electrosleeving; and 4) added the requirement that all sleeves with detected inside diameter flaw indications will be removed from service upon detection.
154	1227	LCO 3.1.8	The amendment revised Limiting Condition for Operation (LCO) 3.1.8, to reduce the required number of channels from four to three channels for certain functions in Technical Specification Table 3.3.1-1 during physics testing in MODE 2.

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155	1241	Section 5.1.1 Section 5.2.1 Section 5.2.2 Section 5.3.1.2 Section 5.5.1	This amendment revised paragraphs in Section 5.0, "Administrative Controls," to allow the use of generic personnel titles in place of plant-specific personnel titles and require either the operations manager or the assistant operations manager to hold a senior reactor operator license
156	1243	ACTIONS 3.8.3.A	This amendment revised Technical Specification 3.8.3 Condition A to increase the specified minimum fuel oil inventories maintained in the fuel oil storage tanks for the diesel generators.
157	1230	TABLE 3.3.1-1 TABLE 3.3.1-2	This amendment revised Technical Specification Tables 3.3.1-1 (Reactor Trip System (RTS) Instrumentation) and 3.3.2-1 (Engineered Safety Feature Actuation System (ESFAS) Instrumentation) of Limiting Conditions for Operation 3.3.1, "RTS Instrumentation," and 3.3.2, "ESFAS Instrumentation," of the TSs. The revisions are for the SG water level low- low (adverse and normal containment environment) functions.
158	1251	ACTION 3.7.5.C	Incorporate a one-time provision that extended the allowed outage time (AOT) for an inoperable turbine-driven auxiliary feedwater pump (TDAFP) by an additional 72 hours, as specified per Required Action C.1. This Amendment permitted additional time to complete troubleshooting, repair, and restoration of the TDAFP

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159	1226	Section 1.0 SR 3.7.3.1	This amendment 1) revised the definition of dose equivalent radioiodine 131 (I-131), and 2) increased the maximum allowed closure time of each main feedwater isolation valve (MFIV) from 5 seconds to 15 seconds. In addition, this amendment approved the re-analysis of the steam generator tube rupture with overfill accident.
160	1245	Section 5.5.6 Section 5.5.16	The amendment 1) revised Technical Specification (TS) Section 5.5.6, "Containment Tendon Surveillance Program," for consistency with the requirements of 10 CFR 50.55a(g)(4) for components classified as Code Class CC and 2) deleted the provisions of Surveillance Requirement (SR) 3.0.2 from this TS. In addition, the amendment revised TS 5.5.16, "Containment Leakage Rate Testing Program," to add exceptions to Regulatory Guide 1.163, "Performance-Based Containment Leak-Testing Program."
161	1244	None	The amendment approved the application of leak-before-break (LBB) for the accumulator and residual heat removal (RHR) lines, and the installation of an opening in the secondary shield wall in terms of the effects of the opening on occupational exposure

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162	1228	SR 3.8.1.10 SR 3.8.1.11 SR 3.8.1.12 SR 3.8.1.13 SR 3.8.1.14 SR 3.8.1.16 SR 3.8.1.17 SR 3.8.1.18 SR 3.8.1.19	This amendment revised several surveillance requirements (SRs) in TS 3.8.1 on alternating current sources for plant operation. The revised SRs have notes deleted or modified to adopt in part the Staff-approved TSTF-283, Revision 3, which will allow these revised SRs to be performed, or partially performed, in reactor modes that previously were not allowed by the TSs.
163	1247	Section 5.5.7	The amendment changed the reactor coolant pump flywheel inspection interval from 10 years to 20 years.
164	1237	LCO 3.0.4 SR 3.0.4 APPL 3.1.1 ACTION 3.1.1.C ACTION 3.3.2.O ACTIONS Note 3.3.3 ACTIONS Note 3.3.4 ACTIONS 3.4.8 ACTIONS Note 3.4.11 ACTIONS Note 3.4.12 ACTIONS 3.4.15 ACTION 3.4.16.A Note ACTIONS Note 3.5.3 ACTION 3.6.8.A ACTION 3.7.4.A ACTION 3.7.4.D ACTIONS Note 3.7.5 ACTIONS Note 3.8.1 APPL 3.9.1 APPL 3.9.6	This amendment modifies the TS requirements for MODE change limitations in Limiting Condition for Operation (LCO) 3.0.4 and Surveillance Requirement (SR) 3.0.4 to adopt the provisions of TSTF-359, "Increase Flexibility in Mode Restraints." This change also deletes or revises notes in individual TSs that are associated with the changes to LCO 3.0.4 and SR 3.0.4 above.

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165	1205	ACTIONS 3.3.1.D ACTIONS 3.3.1.E ACTIONS 3.3.1.M ACTIONS 3.3.1.O ACTIONS 3.3.1.P ACTIONS 3.3.1.Q ACTIONS 3.3.1.R ACTIONS 3.3.1.W ACTIONS 3.3.1.X SR 3.3.1.4 SR 3.3.1.5 SR 3.3.1.7 SR 3.3.1.8 ACTIONS 3.3.2.C ACTIONS 3.3.2.D ACTIONS 3.3.2.E ACTIONS 3.3.2.G ACTIONS 3.3.2.I ACTIONS 3.3.2.K ACTIONS 3.3.2.M ACTIONS 3.3.2.N ACTIONS 3.3.2.O ACTIONS 3.3.2.S SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6 TABLE 3.3.2-1 SR 3.3.9.3	<p>This amendment revised required actions for certain action conditions; increase the completion times for several required actions (including some notes); delete notes in certain required actions; increase frequency time intervals (including certain notes) in several surveillance requirements (SRs); add an action condition and required actions; revise notes in certain SRs; and revise Table 3.3.2-1. These changes were made to adopt the completion time, test bypass time, and surveillance time interval changes in NRC-approved WCAP-14333-P-A, "Probabilistic Risk Analysis of the RPS [reactor protection system] and ESFAS Test Times and Completion Times," and WCAP-15376-P-A, "Risk-Informed Assessment of the RTS and ESFAS Surveillance Test Intervals and Reactor Trip Breaker Test and Completion Times." In addition, an administrative correction to the format of the TSs.</p>
166	1254	Section 5.6.1 Section 5.6.4	<p>This amendment eliminated the requirements in Technical Specification to submit monthly operating reports and annual occupational radiation exposure reports.</p>

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167	1252	All Sections of TS 3.7.3	Revises TS 3.7.3, to adopt the NUREG-1431, "Standard Technical Specifications Westinghouse Plants," Revision 3. The requirements add operability and suitable surveillance requirements for main feedwater regulating valves (MFRV) and MFRV bypass valves (MFRVBV), and allow for an extended out-of-service time for one or more main feedwater isolation valves (MFIVs).
168	1248	TOC Section 1.1 Figure 2.1.1-1 ACTIONS 3.3.1.W Table 3.3.1-1 ACTIONS 3.3.2.M Table 3.3.2-1 LCO 3.4.1 SR 3.4.1.1 SR 3.4.1.2 SR 3.4.5.2 SR 3.4.6.2 LCO 3.4.7 SR 3.4.7.2 LCO 3.4.13 ACTION 3.4.13.A ACTION 3.4.13.B SR 3.4.13.1 SR 3.4.13.2 New Spec 3.4.17 Table 3.7.1-1 Section 5.5.9 Section 5.5.16 Section 5.5.10	The amendment is to support the installation of replacement steam generators (SGs) at Callaway during the refueling outage in the fall of 2005. The amendment also revises (1) the affected transient analyses such as an excessive increase in secondary steam flow event, loss of normal feedwater event, transient mass and energy releases, radiological consequences of associated events, and containment pressure/temperature responses; and (2) the nuclear steam and supply system (NSSS) design parameters and transients, and fatigue usage factors and stresses for the replacement SGs.

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169	1217	Operating License	The amendment (1) deletes Conditions 2.C.(3), 2.C.(4), 2.C.(6) through 2.C.(14), Section 2.F, and Attachments 1 and 2, and (2) revises Conditions 2.C.(1) and 2.C.(5), to the facility operating license, to reflect completed requirements. In addition, the list of the attachments and appendices to the operating license is revised to reflect the deletion of Attachments 1 and 2.
170	1258	SR 3.7.3.1 SR 3.7.3.2 SR 3.7.3.3 Figure 3.7.3-1	This amendment revises the stroke time test requirement for the MFIVs by replacing the single-value 15-second stroke time limit with a figure that specifies the stroke time limit as a function of steam generator steam pressure.
171	1260	ACTION 3.6.6.D ACTION 3.6.6.E	The amendment revises Required Action D.1, in TS 3.6.6, "Containment Spray and Cooling Systems," to require plant shutdown if both containment cooling trains are out of service, which is more conservative than the previous requirement that allowed 72 hours to restore one of the inoperable trains. There are also changes to other required actions in TS 3.6.6 to reflect the revision to Required Action D.1.

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172	1262	LCO 3.7.2 ACTION 3.7.2.A ACTION 3.7.2.B ACTION 3.7.2.C ACTION 3.7.2.D ACTION 3.7.2.E ACTION 3.7.2.F ACTION 3.7.2.G ACTION 3.7.2.H ACTION 3.7.2.I	This amendment revised TS 3.7.2 by adding the MSIV actuator trains to (1) the limiting condition for operation (LCO) and (2) the conditions, rrequired actions, and completion times for the LCO. The existing conditions, required actions in TS 3.7.2 were renumbered to account for the new conditions and required actions.
173	1267	Section 5.1.2 Section 5.2.2.c Section 5.2.2.d Section 5.2.2.f Section 5.3.1.1 Section 5.7.2.a.1	This amendment revises Technical Specification (TS) 5.0, "Administrative Controls", by removing references to the specific position title "Shift Supervisor" and replacing it with a new position title "Shift Manager". In addition, TS 5.0 is revised to change references to "health physics technician(s)" and "Health Physics Supervision" to "Radiation Protection Department technician(s)" and "Radiation Protection Department Supervision".
174	1256	TS 3.1.9 (New) ACTION 3.3.1.V ACTION 3.3.1.Y ACTION 3.3.1.Z TABLE 3.3.1-1	The amendment adds a new TS 3.1.9, "RCS [Reactor Coolant System] Boron Limitations <500°F," and revises TS 3.3.1, "Reactor Trip System (RTS) Instrumentation," for the power range neutron flux - low reactor trip function.

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175	1246	LCO 3.4.15 ACTIONS 3.4.15.B ACTIONS 3.4.15.C ACTIONS 3.4.15.D ACTIONS 3.4.15.E ACTIONS 3.4.15.F SR 3.4.15.1 SR 3.4.15.2 SR 3.4.15.4	The amendment revises Technical Specification (TS) 3.4.1 5 , "RCS [Reactor Coolant System] Leakage Detection Instrumentation" to delete the containment atmosphere gaseous radioactivity monitor from TS 3.4.1 5 and revise the existing conditions, required actions, completion times, and surveillance requirements in TS 3.4.1 5 to account for the monitor being deleted.
176	1255	SR 3.7.2.1 SR 3.7.3.1 SR 3.7.3.3	This amendment revised TS Surveillance Requirements 3.7.2.1, 3.7.3.1, and 3.7.3.3 on verifying the closure time of the main steam isolation valves, main feedwater regulating valves, main feedwater regulating valve bypass valves, and main feedwater isolation valves in the TSs. This amendment also moved TS Figure 3.7.3-1 to TSB.
177	1274	Section 1.1 Section 5.6.6	This amendment revised (1) the definition of the Pressure and Temperature Limits Report (PTLR) in TS 1.1, "Definitions," and (2) TS 5.6.6, "Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)." to allow the use of previously reviewed and approved analytical methods that are specified by number and title only.

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178	1257	Section 1.1 LCO 3.4.16 APP 3.4.16 ACTION 3.4.16.A ACTION 3.4.16.B ACTION 3.4.16.C SR 3.4.16.1 FIG 3.4.16-1 (Del)	This amendment revised Technical Specifications (TSs) 1.1, "Definitions," and 3.4.16, "RCS [Reactor Coolant System] Specific Activity" to replace the current Limiting Condition for Operation (LCO) 3.4.16 limit on RCS gross-specific activity with limits on RCS Dose Equivalent I-131 31 and Dose Equivalent Xe-133 (DEX). In addition, the current definition of Dose Equivalent I-131 in TS 1.1 would be revised to allow alternate, NRC-approved thyroid dose conversion factors.
179	1269	LCO 3.0.1 LCO 3.0.8 (New)	The amendment revises the Technical Specifications by adding a new Limiting Condition for Operation (LCO) 3.0.8. which modifies TS requirements for inoperable snubbers. This new LCO allowing a delay time for entering a supported system TS, when the inoperability is due solely to an inoperable snubber, if the risk is assessed and managed.
180	1270	SR 3.5.2.8 SR 3.6.7.1 SR 3.6.7.2 Appendix C	This amendment revised Technical Specification Surveillance Requirements (SR) 3.5.2.8, SR 3.6.7.1, and adding new SR 3.6.7.2 as needed to reflect the replacement of the containment emergency sumps suction inlet trash racks and screens with strainers in response to Generic Letter 2004-02. In addition the one time note to SR 3.5.2.5 is being removed as a clean up item since it is no longer applicable..

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181	1238	ACTION 3.3.9.B ACTION 3.3.9.C LCO 3.9.2 SR 3.9.2.1	The amendment deletes reference to specific isolation valves in the chemical and volume control system (CVCS) and modifies notes to allow exceptions for decontamination activities and for CVCS resin vessel operation.
182	1268	ACTION 3.1.7.A ACTION 3.1.7.B ACTION 3.1.7.C SR 3.2.1.2 SR 3.2.4.2 SR 3.3.1.3 SR 3.3.1.6	The amendment revises TSs 3.1.7, "Rod Position Indication," 3.2.1, "Heat Flux Hot Channel Factor (FQ(Z)) (F, Methodology)," 3.2.4, "Quadrant Power Tilt Ratio (QPTR)," and 3.3.1, "Reactor Trip System (RTS) Instrumentation," to allow use of the Westinghouse proprietary computer code, the Best Estimate Analyzer for Core Operations - Nuclear (BEACON).
183	1236	Section 2.1.1 Figure 2.1.1-1 (Del) Table 3.3.1-1 3.4.1 LCO SR 3.4.1.1 SR 3.4.1.2 SR 3.4.1.4 Section 5.6.5	This amendment relocated cycle-specific parameter limits from TS to the Core Operating Limits Report (COLR) and added several topical reports and allowed all cited topical reports to be identified by title and number only. These change are consistent with TSTF-339 and TSTF-363.

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184	1259	ACTION 3.7.10.D.1 ACTION 3.7.11.C.1 ACTION 3.7.13.A ACTION 3.7.13.D	The amendment deletes Required Action D.1.2 in TS 3.7.1 0, "Control Room Emergency Ventilation System (CREVS)," and Required Action C.1.2 in TS 3.7.1 1, "Control Room Air Conditioning System (CRACS)." For TS 3.7.13, "Emergency Exhaust System (EES)," the amendment also deletes the phrase "in MODE 1, 2, 3, or 4" from Condition A (one EES train inoperable) and revises Condition D to state the following: "Required Action and associated Completion Time of Condition A not met during movement of irradiated fuel assemblies in the fuel building."
185	1271	ACTION 3.8.3.A SR 3.8.3.1	This amendment revises TS 3.8.3, "Diesel Fuel Oil, Lube Oil, and Starting Air," and its associated Surveillance Requirement 3.8.3.1 to increase the current minimum emergency diesel generator (EDG) fuel oil inventory required to be maintained onsite.
186	1282	ACTION 3.7.8.A.1 ACTION 3.8.1.B.4	This amendment revised the TSs to allow a one-time extension to the Completion Times for both essential service water (ESW) trains and the emergency diesel generators from 72 hours to 14 days. The revision to the TSs would apply when each train of the ESW system is inoperable during their respective ESW system piping replacements.

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187	1284	Section 5.5.8	The amendment revised TS 5.5.8, "Inservice Testing Program," to indicate that Inservice Testing (IST) Program shall include testing frequencies applicable to the ASME Code for operation and Maintenance of Nuclear Power Plants (OM Code), and to indicate that there may be some non-standard frequencies, specified as once every 2 years or less in the IST Program, to which provisions of surveillance requirement (SR) 3.0.2 are applicable.
188	1281	ACTION 3.4.10.B.2 ACTION NOTE ACTION 3.4.11.F.1 ACTION 3.4.12.G.1	This amendment revised TS 3.4.10, "Pressurizer Safety Valve," TS 3.4.11, "Pressurizer Power Operated Relief Valves (PORVs)," and TS 3.4.12, "Cold Overpressure Mitigation System (COMS)," to adopt NRC approved TS Task Force (TSTF) travelers to the Standard TSs TSTF-247-A and TSTF-352-A
189	1279	TABLE 3.3.2-1 SR 3.7.2.1 SR 3.7.2.2	The amendment revised the exception in TS Table 3.3.2-1 for the main feedwater isolation system to be consistent with the modes of applicability for that same system in TS 3.7.3, "Main Feedwater Isolation Valves (MFIVs) and Main Feedwater Regulating Valves (MFRVs) and Main Feedwater Regulating Valve Bypass Valves (MFRVBVs)," and deleted the same note in SRs 3.7.2.1 and 3.7.2.2 on the MSIVs because the note is no longer needed or appropriate for the two SRs.

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190	1287	3.7.10 LCO Note ACTIONS 3.7.10.A ACTIONS 3.7.10.B ACTIONS 3.7.10.E SR 3.7.10.2 SR 3.7.10.4 SECTION 5.5.17	The amendment adds a new license condition on the control room envelope (CRE) habitability program, revises the TS requirements related to the CRE habitability in TS 3.7.10, "Control Room Emergency Ventilation System (CREVS)," and establishes a CRE habitability program in TS Section 5.5, "Administrative Controls -Programs and Manuals." These changes are consistent with the NRC-approved IndustryTS Task Force (TSTF) Traveler TSTF-448, Revision 3, "Control Room Habitability."
191	1282	ACTION 3.7.8.A.1 ACTION 3.8.1.B.4	This amendment revised the TSs to allow a one-time extension to the Completion Times for both essential service water (ESW) trains and the emergency diesel generators from 72 hours to 14 days. The revision to the TSs would apply when "B" train of the ESW system is inoperable during the ESW system piping replacements.
192	1283	ACTION 3.3.7.E Table 3.3.7-1 3.3.8 ACTION Notes 3.7.10 APPLICABILITY ACTION 3.7.10.D ACTION 3.7.10.E 3.7.13 ACTION Note 3.8.2 APPLICABILITY 3.8.2 ACTION Note 3.8.5 APPLICABILITY 3.8.5 ACTION Note 3.8.8 APPLICABILITY 3.8.8 ACTION Note 3.8.10 APPLICABILITY 3.8.10 ACTION Note	This amendment (1) deletes MODES 5 and 6 from the Control Room Emergency Ventilation System and its actuation instrumentation in TS 3.7.10 and TS 3.3.7; (2) adopts NRC-approved Technical Specification Task Force (TSTF) change traveler TSTF-36-A for TSs 3.3.8, 3.7.13, 3.8.2,3.8.5,3.8.8, and 3.8.10; and (3) adds a restriction to the Limiting Condition for Operation (LCO) applicability for TSs 3.8.2, 3.8.5, 3.8.8, and 3.8.10 such that these LCOs apply not only during MODES 5 and 6, but also during the movement of irradiated fuel assemblies regardless of the MODE in which the plant is operating.

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193	09-0014	SECTION 5.2.2.d	<p>This amendment revised the TS 5.2.2, "Unit Staff," to eliminate working hour restrictions in paragraph d to support compliance with Title 10 of the <i>Code of Federal Regulations</i> (10 CFR) Part 26. The request is consistent with the guidance contained in the U.S. Nuclear Regulatory Commission (NRC)-approved Revision 0 to Technical Specifications Task Force (TSTF) change traveler TSTF-511, "Eliminate Working Hour Restrictions from TS 5.2.2 to Support Compliance with 10 CFR Part 26."</p>
194	08-0002	SR 3.8.4.2 SR 3.8.4.5	<p>This amendment revises the battery connection resistance verification limits in Surveillance Requirement (SR) 3.8.4.2 and SR 3.8.4.5, by lowering the acceptance criteria for cell-to-cell (i.e., inter-cell) and terminal battery connection resistance from 150 micro-ohms to 69 micro-ohms.</p>
195	09-0008	Section 5.5.16	<p>This amendment revised TS 5.5.16, "Containment Leakage Rate Testing Program." The revision reflects a one-time extension of the current containment Type A leak rate test (integrated leak rate test or ILRT) interval requirement of Title 10 of the Code of Federal Regulations (10 CFR) Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors," Option B, "Performance Based Requirements," from 10 years to 15 years. The change allows the next ILRT to be performed no later than October 25, 2014.</p>

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196	10-0011	ACTIONS 3.3.2.J	The amendment revises TS 3.3.2, "Engineered Safety Feature Actuation System (ESFAS) Instrumentation," Condition J for ESFAS instrumentation function 6.g to read, "One or more Main Feedwater Pumps trip channel(s) inoperable," make corresponding changes to Required Action J.1, and place a Note above Required Actions J.1 and J.2 for consistency with the revised Condition.
197	1277	ACTIONS 3.3.2.P.2 Table 3.3.2-1 LCO 3.7.2 Applicability 3.7.2 ACTIONS 3.7.2.E ACTIONS 3.7.2.H ACTIONS 3.7.2.H.1 ACTIONS 3.7.2.H.2 ACTIONS 3.7.2.I ACTIONS 3.7.2.I.1 ACTIONS 3.7.2.I.2 ACTIONS 3.7.2.J ACTIONS 3.7.2.J.1 ACTIONS 3.7.2.J.2 ACTIONS 3.7.2.K ACTIONS 3.7.2.K.1 ACTIONS 3.7.2.K.2 SR 3.7.2.2 SR 3.7.2.3 TS 3.7.19 (New)	This amendment made the following changes. 1) Revised TS 3.7.2, "Main Steam Isolation Valves (MSIVs)," to add the main steam isolation valve bypass valves and main steam low point drain isolation valves to the scope of the TS 3.7.2; 2) Revise footnotes (i) and (k) in TS Table 3.3.2-1, to make the Applicability of TS Table 3.3.2-1 consistent with the Applicability of TS 3.7.2; and 3) Add new TS 3.7.19, "Secondary System Isolation Valves (SSIVs)," Correspondingly, add new Function 10 in TS Table 3.3.2-1.

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198	1289	Table 3.3.2-1 LCO 3.7.3 Applicability 3.7.3 ACTIONS 3.7.3.C.1 ACTIONS 3.7.3.C.2	This amendment made the following changes. 1) Revised TS 3.7.3 so that the LCO and Applicability more accurately reflect the conditions for when the LCO should be applicable and more effectively provide appropriate exceptions to the Applicability for certain valve configurations.; 2)Revise the title of TS 3.7.3; and 3) Revised the exception footnotes in TS 3.3.2, Table 3.3.2-1, to improve the application of existing notes and/or incorporate more appropriate notes as applicable.
199	1253	ACTIONS 3.3.3.C ACTIONS 3.3.3.D ACTIONS 3.3.3.E ACTIONS 3.3.3.F ACTIONS 3.3.3.G SR 3.3.3.1 TABLE 3.3.3-1 SPEC 3.6.8 (Del) ACTIONS 3.7.4.D ACTIONS 3.7.4.D.1ACTIONS 3.8.1.A.2 ACTIONS 3.8.1.C.1 Section 5.5.16 Section 5.6.3 Section 5.6.8	The amendment deletes the TS requirements for the containment hydrogen recombiners and relaxed the requirements for hydrogen and oxygen monitoring. The TS changes would support implementation of the revisions to Title 10 of the <i>Code of Federal Regulations</i> (10 CFR) Section 50.44, "Combustible gas control for nuclear power reactors," that became effective on October 16, 2003. In addition, the amendment corrected four typographical errors in the TSs.
200	09-0017	Applicability 3.3.9	This amendment revises the LCO 3.3.9 Applicability Note to clarify the situations during which the Boron Dilution Mitigation System signal may be blocked in MODES 2 and 3.

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201	09-0039	ACTIONS 3.3.2.J ACTIONS 3.3.2.M ACTIONS 3.3.2.O ACTIONS 3.3.2.Q Table 3.3.2-1	The amendment revises TS 3.3.2, "Engineered Safety Feature Actuation System (ESFAS) Instrumentation," to provide a 24-hour Completion Time (CT) for restoration of an inoperable Balance of Plant (BOP) ESFAS train and extends the CTs associated with individual instrument channels in the BOP ESFAS train to maintain overall consistency of related TS actions.

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202		SR 3.1.1.1 SR 3.1.2.1 SR 3.1.4.1 SR 3.1.4.2 SR 3.1.5.1 SR 3.1.6.2 SR 3.1.6.3 SR 3.1.8.2 SR 3.1.8.3 SR 3.1.8.4 SR 3.1.9.1 SR 3.2.1.1 SR 3.2.1.2 SR 3.2.2.1 SR 3.2.3.1 SR 3.2.4.1 SR 3.2.4.2 SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.4 SR 3.3.1.5 SR 3.3.1.6 SR 3.3.1.7 SR 3.3.1.8 SR 3.3.1.9 SR 3.3.1.10 SR 3.3.1.11 SR 3.3.1.13 SR 3.3.1.14 SR 3.3.1.16 SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6 SR 3.3.2.7 SR 3.3.2.8 SR 3.3.2.9 SR 3.3.2.10	<p>The amendment revised the TSs by relocating specific surveillance frequencies to a ICallaway controlled program with the guidance of Nuclear Energy Institute (NEI) 04-10, "Risk-Informed Technical Specifications Initiative 5b, Risk-Informed Method for Control of Surveillance Frequencies." This amendment adopts NRC-approved Technical Specification Task Force (TSTF)-425, Revision 3, "Relocate Surveillance Frequencies to Licensee Control - RITSTF [Risk-Informed TSTF] Initiative 5b." When implemented, TSTF-425 relocates most periodic frequencies of TS surveillances to a licensee-controlled program, the Surveillance Frequency Control Program (SFCP), and provides requirements for the new program in the Administrative Controls section of the TSs.</p>

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202 (cont.)		SR 3.3.2.11	
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		SR 3.3.2.13	
		SR 3.3.2.14	
		SR 3.3.3.1	
		SR 3.3.3.2	
		SR 3.3.4.1	
		SR 3.3.4.2	
		SR 3.3.4.3	
		SR 3.3.5.1	
		SR 3.3.5.2	
		SR 3.3.5.3	
		SR 3.3.5.4	
		SR 3.3.6.1	
		SR 3.3.6.2	
		SR 3.3.6.3	
		SR 3.3.6.4	
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		SR 3.3.9.6	
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		SR 3.4.1.4	
		SR 3.4.2.1	

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202 (cont.)		SR 3.4.3.1	
		SR 3.4.4.1	
		SR 3.4.5.1	
		SR 3.4.5.2	
		SR 3.4.5.3	
		SR 3.4.6.1	
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		SR 3.4.6.3	
		SR 3.4.7.1	
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		SR 3.4.8.2	
		SR 3.4.9.1	
		SR 3.4.9.2	
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		SR 3.4.15.3	
		SR 3.4.15.4	
		SR 3.4.15.5	
		SR 3.4.16.1	
		SR 3.4.16.2	
		SR 3.5.1.1	
		SR 3.5.1.2	
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		SR 3.5.1.4	
		SR 3.5.1.5	
		SR 3.5.2.1	

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		SR 3.5.2.7	
		SR 3.5.2.8	
		SR 3.5.4.1	
		SR 3.5.4.2	
		SR 3.5.4.3	
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		SR 3.6.3.1	
		SR 3.6.3.2	
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		SR 3.7.8.1	
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203	10-0022	Operating License E	The amendment approves the Callaway Plant, Unit 1, Cyber Security Plan and associated implementation schedule, and revises Paragraph 2.E of Facility Operating License No. NPF-30 to provide a license condition to require the licensee to fully implement and maintain in effect all provisions of the NRC-approved Cyber Security Plan.

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204	10-0036	SR 3.3.8.6 Table 3.3.8-1	The amendment adds new Surveillance Requirement (SR) 3.3.8.6, to TS 3.3.8, "Emergency Exhaust System (EES) Actuation Instrumentation" and to Table 3.3.8-1, "EES Actuation Instrumentation." The new SR requires the performance of response time testing on the portion of the EES required to isolate the normal fuel building ventilation exhaust flow path and initiate the fuel building ventilation isolation signal mode of operation.
205	09-0040	ACTION 3.3.6.B.1	This amendment adds a Note to Required Action B.1 of TS 3.3.6, "Containment Purge Isolation Instrumentation," to conditionally allow containment mini-purge supply and exhaust valves that have been closed in accordance with the Action to be opened under administrative controls as required for certain operational needs.
206	11-0012	Operating License C(2) Operating License C(5) Section 5.4.1	The amendment transitions the Callaway fire protection program to a risk-informed, performance-based program based on National Fire Protection Association (NFPA) 805, "Performance-Based Standard for Fire Protection for Light Water Reactor Electric Generating Plants, 2001 Edition" (NFPA 805), in accordance with 10 CFR 50.48(c).

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207	12-0041	Operating License C(2)	This amendment revises the methodology in the licensing basis as described in the FSAR-SP to include damping values for the seismic design and analysis of the integrated head assembly that are consistent with the recommendations of NRC Regulatory Guide 1.61, "Damping Values for Seismic Design of Nuclear Power Plants," Revision 1, March 2007.
208	12-0015	SR 3.7.9.1 SR 3.7.9.3 SR 3.7.9.4	This amendment revised TS 3.7.9, "Ultimate Heat Sink (UHS)," to incorporate a more restrictive UHS level and pond temperature limits which are specified in Surveillance Requirements (SRs) 3.7.9.1 and 3.7.9.2, respectively. A new SR 3.7.9.4 is added to verify that the UHS cooling tower fans respond appropriately to automatic start signals.
209	13-0010	SR 3.7.10.1 SR 3.7.13.1	The amendment revises TS Surveillance Requirement (SR) 3.7.10.1 and SR 3.7.13.1 to reduce the required run time for periodic operation of the control room pressurization system filter trains and emergency exhaust system filter trains, with heaters on, from 10 hours to 15 minutes. The amendment is consistent with plant-specific options provided in the NRC's model safety evaluation in Technical Specifications Task Force (TSTF) Traveler TSTF-522, Revision 0, "Revise Ventilation System Surveillance Requirements to Operate for 10 hours per Month," as part of the consolidated line item improvement process.

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210	13-0003	Table 3.3.1-1 3.4.12 LCO 3.4.12 LCO Notes ACTIONS 3.4.12.B ACTIONS 3.4.12.B.1 SR 3.4.12.2	The amendment revises TS 3.4.12, "Cold Overpressure Mitigation System (COMS)," to reflect the mass input transient analysis that assumes an Emergency Core Cooling System centrifugal charging pump and the normal charging pump capable of injecting into the reactor coolant system when TS 3.4.12 is applicable. The amendment also revises TS Table 3.3.1-1, "Reactor Trip System Instrumentation," to remove unnecessary page number references.
211	13-0016	SP SEC 3.6.2.1.2.4 SP TBL 3.6-2	The amendment adds a new pipe crack exclusion allowance to FSAR-Standard Plant Section 3.6.2.1.2.4, "ASME [American Society of Mechanical Engineers] Section III and Non-Nuclear Piping-Moderate-Energy," and FSAR-Standard Plant Table 3.6-2, "Design Comparison to Regulatory Positions of Regulatory Guide 1.46, Revision 0, dated May 1973, titled 'Protection Against Pipe Whip Inside Containment,'" in particular regard to the high-density polyethylene (HOPE) piping installed in ASME Class 3 line segments of the essential service water system. Also, new Reference 25 is added to FSAR-Standard Plant Section 3.6.3 to cite the NRG-approved version of the HOPE requirements covered by Relief Request 13R-10 dated October 31, 2008.

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212	NA	Emergency Action Level (EAL)	The amendment revises Callaway's EAL scheme to a scheme based on Nuclear Energy Institute (NEI) 99-01, Revision 6, "Development of Emergency Action Levels for Non-Passive Reactors," November 2012.
213	14-0003	SR 3.4.6.4 SR 3.4.7.4 SR 3.4.8.3 SR 3.5.2.2 SR 3.5.2.3 SR 3.6.6.1 SR 3.6.6.9 SR 3.9.5.2 SR 3.9.6.3	This amendment revised the TS requirements to address NRC Generic Letter 2008-01, "Managing Gas Accumulation in Emergency Core Cooling, Decay Heat Removal, and Containment Spray Systems as described in Technical Specification Task Force (TSTF) Traveler TSTF-523, Revision 2, Generic Letter 2008-01, Managing gas Accumulation."
214	15-0009	Operating License 17(E)	This amendment revised the final implementation date associated with the Callaway Plant Cyber Security Plan and License Amendment 203
215	14-0014	3.4.17 LCO 3.4.17 ACTION A SR 3.4.17.2 Section 5.5.9 Section 5.6.10	This amendment revised the TS requirements to adopt TSTF-510, "Revision to Steam Generator Program Inspection Frequencies and Tube Sample Selection"
216	15-0005	2.1.1.1 LCO Section 5.6.5.b	This amendment revise the TS requirements to adopt WCAP-14565-P-A Addendum 2-P-A methodology and revise the DNBR to align with previously approved WCAP-14483
217	16-0011	Section 5.6.5.b	This amendment revises the TS requirements to adopt the approved core design methodology described in WCAP-16045-P-A and WCAP-16045-P-A Addendum 1-A.

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218	16-0001	FSAR SP Appendix 3A FSAR SP 8.3.1.1.2	This amendment revises the commitment to Reg. Guide 1.106 to specify use of Regulatory Position C.2 for the auxiliary feedwater control valves (ALHV0005, ALHV0007, ALHV0009, and ALHV0011).
219	16-0013	License Condition 2.C.(18)	This amendment adds a new TS 3.7.20, "Class 1E Electrical Equipment Air Conditioning System," and adds a new License Condition 2.C.(18) for completion of plant modifications to install supplemental cooling trains and reduce the power of each control room pressurization system charcoal adsorber unit heater from 15 kW to 5 kW.
220	N/A	N/A	This amendment revises Callaway's EAL scheme by changing EAL CA6.1, "Cold Shutdown/Refueling System Malfunction — Hazardous event affecting a SAFETY SYSTEM needed for the current operating MODE: Alert," and EAL SA9.1, "System Malfunction — Hazardous event affecting a SAFETY SYSTEM needed for the current operating MODE: Alert," and deleting Initiating Condition HG1 and associated EAL HG1.1, "Hazard — HOSTILE ACTION resulting in loss of physical control of the facility: General Emergency."

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221	18-0025	TS SR 3.3.2.6 TS SR 3.3.2.14 TS TBL 3.3.2-1	This amendment revises TS Table 3.3.2-1 Function 5.a and relocates TS SR requirements for SLAVE RELAY TEST of slave relay K602 from TS SR 3.3.2.14 to TS SR 3.3.2.6.
222	18-0021	TS 1.1 TS SR 3.4.10.1 TS SR 3.4.11.1 TS SR 3.5.2.4 TS SR 3.6.3.5 TS SR 3.6.6.4 TS SR 3.7.1.1 TS SR 3.7.2.1 TS SR 3.7.2.3 TS SR 3.7.3.1 TS SR 3.7.3.3 TS SR 3.7.4.1 TS SR 3.7.4.2 TS SR 3.7.19.1 TS AC 5.5.8	This amendment deletes TS AC 5.5.8, "Inservice Testing Program," and revises TS SR 3.4.10.1, TS SR 3.4.11.2, TS SR 3.5.2.4, TS SR 3.6.3.5, TS SR 3.6.6.4, TS SR 3.7.1.1, TS SR 3.7.2.1, TS SR 3.7.2.3, TS SR 3.7.3.1, TS SR 3.7.3.3, TS SR 3.7.4.1, TS SR 3.7.4.2, and TS SR 3.7.19.1, by replacing TS AC 5.5.8, "Inservice Testing Program" with a new term "INSERVICE TESTING PROGRAM" defined in TS 1.1 as per TSTF-545, Rev. 3.
223	20-0013	TS AC 5.5.9	This amendment revises TS AC 5.5.9, "Steam Generator (SG) Program," to implement a one-time deferral of the steam generator inspection scheduled for Refuel Outage 24 (fall 2020) to Refuel Outage 25 (spring 2022).
224	19-0012	License Condition 2.C.(5)	This amendment revises License Condition 2.C.(5) to reflect NRC approval for acceptance, by use of performance-based methods, of installed and new installations of anti-sweat insulation (not conforming to NFPA-805) in limited applications at Callaway Plant.