



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

October 26, 2021

Mr. Daniel G. Stoddard  
Senior Vice President and  
Chief Nuclear Officer  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT  
NOS. 305 AND 305 RE: REVISED REACTOR CORE SAFETY LIMIT TO  
REFLECT TOPICAL REPORT WCAP-177642-P-A, REVISION 1  
(EPID L-2020-LLA-0219)

Dear Mr. Stoddard:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 305 to Subsequent Renewed Facility Operating License No. DPR-32 and Amendment No. 305 to Subsequent Renewed Facility Operating License No. DPR-37 for the Surry Power Station (Surry), Unit Nos. 1 and 2, in response to your application dated September 30, 2020, as supplemented on September 28, 2021, October 13, 2021, and October 19, 2021 Agencywide Documents Access and Management System (ADAMS) Accession No. ML20274A329, ML21272A062, ML21286A737, and ML21292A302, respectively.

The amendments modify the technical specifications by revising the Reactor Core Safety Limit 2.1.A.1.b peak fuel centerline temperature to reflect the fuel centerline temperature specified in Topical Report WCAP-17642-P-A, Revision 1, "Westinghouse Performance Analysis and Design Model (WCAP-17642-P-A)."

A copy of the related safety evaluation is also enclosed. The Commission's monthly *Federal Register* notice will include the notice of issuance.

Sincerely,

***/RA/***

John Klos, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosures:

1. Amendment No. 305 to DPR-32
2. Amendment No. 305 to DPR-37
3. Safety Evaluation

cc: Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 305  
Subsequent Renewed License  
No. DPR-32

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated September 30, 2020, as supplemented by emails and letter dated September 28, 2021, October 13, 2021 and October 19, 2021 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specification as indicated in the attachment to this license amendment of the Subsequent Renewed Facility Operating License No. DPR-32 are hereby amended to read as follows:

- B. Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 305 are hereby incorporated in the subsequent renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Date of Issuance: October 26, 2021



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 305  
Subsequent Renewed License  
No. DPR-37

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated September 30, 2020, as supplemented by emails and letter dated September 28, 2021, October 13, 2021 and October 19, 2021 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specification as indicated in the attachment to this license amendment of the Subsequent Renewed Facility Operating License No. DPR-37 are hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 305 are hereby incorporated in the subsequent renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Date of Issuance: October 26, 2021

ATTACHMENT TO  
SURRY POWER STATION, UNIT NOS. 1 AND 2  
LICENSE AMENDMENT NO. 305  
SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-32  
DOCKET NO. 50-280  
AND  
LICENSE AMENDMENT NO. 305  
SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-37  
DOCKET NO. 50-281

Replace the following pages of the Licenses and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contained marginal lines indicating the areas of change.

Subsequent Renewed Facility Operating License No. DPR-32

REMOVE

3

INSERT

3

Subsequent Renewed Facility Operating License No. DPR-37

REMOVE

3

INSERT

3

TSs

REMOVE

2.1-1

INSERT

2.1-1

3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

- A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2587 megawatts (thermal).

- B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 305 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

- C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

- D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

- E. Deleted by Amendment 65

- F. Deleted by Amendment 71

- G. Deleted by Amendment 227

- H. Deleted by Amendment 227

- I. Fire Protection

The licensee shall implement and maintain in effect the provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report and as approved in the SER dated September 19, 1979, (and Supplements dated May 29, 1980, October 9, 1980, December 18, 1980, February 13, 1981, December 4, 1981, April 27, 1982, November 18, 1982, January 17, 1984, February 25, 1988, and



- E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such by product and special nuclear materials as may be produced by the operation of the facility.
- 3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
  - A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power Levels not in excess of 2587 megawatts (thermal).
  - B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 305 are hereby incorporated in this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.
  - C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.
  - D. Records

The licensee shall keep facility operating records in accordance with the Requirements of the Technical Specifications.
  - E. Deleted by Amendment 54
  - F. Deleted by Amendment 59 and Amendment 65
  - G. Deleted by Amendment 227
  - H. Deleted by Amendment 227

## 2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

### 2.1 SAFETY LIMIT, REACTOR CORE

#### Applicability

Applies to the limiting combinations of THERMAL POWER, Reactor Coolant System pressure, coolant temperature and coolant flow when a reactor is critical.

#### Objective

To maintain the integrity of the fuel cladding.

#### Specification

- A. The combination of reactor THERMAL POWER level, pressurizer pressure, and Reactor Coolant System (RCS) highest loop average temperature shall not:
1. Exceed the limits specified in the CORE OPERATING LIMITS REPORT when full flow from three reactor coolant pumps exists, and the following Safety Limits shall not be exceeded:
    - a. The design limit for departure from nucleate boiling ratio (DNBR) shall be maintained  $\geq 1.27$  for transients analyzed using the Statistical DNBR Evaluation Methodology and the WRB-1 DNB correlation. For transients analyzed using the deterministic methodology, the DNBR shall be maintained greater than or equal to the applicable DNB correlation limit ( $\geq 1.17$  for WRB-1,  $\geq 1.30$  for W-3,  $\geq 1.14$  for ABB-NV).
    - b. The peak fuel centerline temperature shall be maintained  $< 5080^{\circ}\text{F}$ , decreasing by  $9^{\circ}\text{F}$  per 10,000 MWD/MTU of burnup.
  2. The reactor THERMAL POWER level shall not exceed 118% of rated power.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
LICENSE AMENDMENT REQUEST TO REVISE THE REACTOR CORE SAFETY LIMIT  
AMENDMENT NO. 305 TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO.

DPR-32

AMENDMENT NO. 305 TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO.

DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NOS. 1 AND 2,

DOCKET NOS. 50-280 AND 50-281

## 1.0 INTRODUCTION

By application dated September 30, 2020, as supplemented on September 28, 2021, October 13, 2021, and October 19, 2021, Agencywide Documents Access and Management System (ADAMS) Accession No. ML20274A329, ML21272A062, ML21286A737 and ML21292A302 respectively, Virginia Electric and Power Company (the licensee) submitted a license amendment request (LAR) for Surry Power Station Unit Nos 1 and 2 (Surry). The proposed amendments would revise the Surry Technical Specifications (TSs) reactor core safety limit (SL) 2.1.A.1.b to reflect the peak fuel centerline melt temperature specified in Topical Report (TR) WCAP-17642-P-A, Revision 1, "Westinghouse Performance Analysis and Design Model (WCAP-17642-P-A)," dated November 2017, ADAMS Accession No. ML17338A396 (non-proprietary version).

The supplements dated September 28, 2021, October 13, 2021, and October 19, 2021 provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination.

## 2.0 REGULATORY EVALUATION

### 2.1 System Description

The proposed amendments are related to the Surry nuclear fuel. Reactor core SL 2.1.A.1.b protects against overheating of the nuclear fuel and fuel cladding, which would lead to the fuel centerline temperature rising above the fuel melting point. The consequences of fuel and

cladding overheating include fuel clad perforation could lead to nuclear fission product release to the reactor coolant. Clad perforation can occur when the fuel centerline temperature reaches the melting point for the fuel and the fuel pellet expands. Reactor core SL 2.1.A.1.b is dependent on burnup and is applicable whenever the reactor is critical.

## 2.2 Regulatory Requirements and Guidance

The U. S. Nuclear Regulatory Commission (NRC) issued construction permits for Surry Units 1 and 2 before May 21, 1971; consequently, Surry Units 1 and 2 were not subject to the requirements in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, "Domestic Licensing of Production and Utilization Facilities," Appendix A, "General Design Criteria [GDCs] for Nuclear Power Plants" (see SECY-92-223, "Resolution of Deviations Identified during the Systematic Evaluation Program," dated September 18, 1992 (ADAMS Accession No. ML003763736)). Surry Units 1 and 2 meet the intent of the GDCs published in 1967 (draft GDCs), including Criterion 6, Reactor Core Design (Category A), which was the precursor to the current GDC 10, Reactor Design. The NRC staff considered the following proposed draft GDC as being applicable to the review of the proposed Surry TS SL 2.1.A.1.b change:

[1967] proposed Draft GDC 6, which requires that the reactor core, with its related controls and protection systems, be designed to function throughout its design lifetime without exceeding acceptable fuel limits specified to preclude damage.

Additional statutes and regulations that are applicable to the proposed TS SL 2.1.A.1.b change include:

Section 182.a of the Atomic Energy Act of 1954, as amended, requires applicants for nuclear power plant operating licenses to include TSs as part of the license application to enable the Commission to find that the utilization of special nuclear material will be in accord with the common defense and security and will provide adequate protection to the health and safety of the public.

Section 50.36 (10 CFR 50.36), "Technical specifications," paragraph (a)(1) also requires that an application for an operating license include proposed TSs and paragraph (c) requires that TSs include items such as: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls.

Section 50.36(b) (10 CFR 50.36(b)) also requires that technical specifications will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto.

Section 50.90 (10 CFR 50.90), "Application for amendment of license, construction permit, early site permit," requires NRC approval for any modification to, addition to, or deletion from the TSs.

## 2.3 Licensee's Proposed Changes

The licensee proposes to revise the TS reactor core SL 2.1.A.1.b peak fuel centerline temperature to reflect the fuel centerline temperature specified in TR WCAP-17642-P-A, Revision 1.

The current Surry TS SL 2.1.A.1.b states:

The peak fuel centerline temperature shall be maintained <5080°F, decreasing by 58°F per 10,000 MWD/MTU of burnup.

The licensee requests that Surry TS SL 2.1.A.1.b be revised to state:

The peak fuel centerline temperature shall be maintained <5080°F, decreasing by 9°F per 10,000 MWD/MTU of burnup.

### 3.0 TECHNICAL EVALUATION

The scope of the NRC staff's review is limited to the proposed change to the TS SL for peak fuel centerline temperature. The proposed limit is the limit in TR WCAP-17642-P-A, Revision 1 (WCAP-17642-P-A).

The empirically derived fuel centerline melt temperature described in WCAP-17642-P-A is based on fuel properties described in open literature. The description of the fuel properties can be found in (1) S.G. Popov; J.J. Carbajo; V.K. Ivanov; and G.L. Yoder, "Thermophysical Properties of MOX and UO<sub>2</sub> Fuels Including the Effects of Irradiation," ORNL/TM-2000/3S1 (2000) and (2) J.J. Carbajo; G.L. Yoder; S.G. Popov; and V.K. Ivanov, "A Review of the Thermophysical Properties of MOX and UO<sub>2</sub> Fuels," Journal of Nuclear Materials, 299, 181 (2001). As noted beneath the caption for Figure 59, on page 92 of the NRC staff SE on WCAP-17642-P-A (ADAMS Accession no. ML21007A264), in its approval of WCAP-17642-P-A, the NRC staff determined that this melting limit is acceptable. The burnup dependent fuel centerline melt temperature is based on inherent fuel properties and does not depend upon any specific calculational methodology. Therefore, the NRC staff considers it acceptable as a standalone limit.<sup>1</sup>

#### 3.1 Consideration of WCAP-17642-P-A Limitations and Conditions

The licensee described how applicable limitations and conditions in the NRC staff's safety evaluation of WCAP-17642-P-A would be addressed. The NRC staff notes that there are other limitations and conditions in the WCAP-17642-P-A safety evaluation that are not addressed in the LAR. Since the licensee is not requesting in the LAR to implement the portions of the WCAP-17642-P-A methodology that would necessitate those limitations and conditions, the NRC staff determined that the LAR need not address the limitations and conditions.

The Surry LAR addressed limitations and conditions (a) and (b) in Section 4.1 of the NRC staff safety evaluation on WCAP-17642-P-A. Adherence to the limitation and condition (a) ensures that the proposed SL applies to Surry. If limitation and condition (a) is not satisfied, then a different SL for fuel melting would be needed. In addition, limitation and condition (b) requires that, should WCAP-17642-P-A be implemented at any point, the Surry specific analyses are not to be used to predict fuel conditions beyond the melting point.

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<sup>1</sup> The two identified references provide the data describing the fuel properties. The specific burnup dependence is provided in Section 6.1.5 of TR WCAP-17642-P-A based on an assessment of these data. The NRC staff determined that this burnup dependence was acceptable as described in Section 3.7.12 of the staff's safety evaluation approving TR WCAP-17642-P-A.

Limitation and condition (a) states, in part:

The NRC staff limits the applicability of the PAD5 code and methodology to the cladding, fuel [types] and reactor parameters for the ranges listed in Section 4.1 of the [NRC staff's] Safety Evaluation [of the PAD5 methodology] in Reference 1.

The NRC staff confirmed that Surry uses Westinghouse fuel that meets the constraints identified in limitation and condition (a). Surry also stated that "these parameters are validated on a cycle-specific basis." As such, the NRC staff is assured that the proposed SL is applicable to Surry Units 1 and 2 in that it reflects an acceptable limit on the peak fuel centerline temperature that would preclude fuel melting. Based on the confirmation provided, the NRC staff determined that this approach to address limitation and condition (a) is acceptable to change TS SL 2.1.A.1.b for Surry.

Limitation and condition (b) states:

The application of PAD5 should at no time exceed the fuel melting temperature as calculated by PAD5 due to the lack of properties for molten fuel in PAD5 and other properties such as thermal conductivity and fission gas release.

In the LAR, the licensee stated that it "will limit the peak fuel centerline temperature per this amendment request." The NRC staff determined that the licensee's statement is acceptable because the LAR specifically proposes to change the peak fuel centerline temperature SL to ensure that fuel melt is precluded during conditions for normal operation and anticipated operational occurrences, consistent with the requirements of the 1967 Proposed Draft GDC 6. The staff also notes that the LAR only proposed to implement the empirically derived TS SL and therefore, this SE does not approve the use of PAD5 to calculate any associated parameters.

### 3.2 TS SL 2.1.A.1.b Change Evaluation

In the LAR, the licensee requests to change Surry TS SL 2.1.A.1.b to reflect the fuel centerline melt temperature found in WCAP-17642-P-A. Surry TS SL 2.1.A.1.b limits the peak fuel centerline temperature. For normal operation and anticipated operational occurrences, the reactor protection system is designed to ensure that the peak fuel centerline temperature does not exceed the fuel melt temperature criterion. The intent of this criterion is to avoid gross fuel melting.

The NRC staff evaluated the peak fuel centerline temperature limit and determined that it is applicable to Surry because, as discussed below, the properties of the fuel design in use at Surry Units 1 and 2 are consistent with the fuel property data used to generate the proposed SL.

The proposed SL was empirically derived and covers the lower bound of the fuel property data, which is conservative because the lower bound assures that the SL does not underestimate the applicable data. Additionally, as documented in the SE on WCAP-17642-P-A, the NRC staff compared the fuel centerline melting temperatures in the TR to the fuel centerline melting temperature in the NRC's code, "FRAPCON" (now known as "FAST"). The WCAP-17642-P-A fuel centerline melting temperature is less than that in FRAPCON, which is conservative. The NRC staff determined that its analysis and technical justification for the fuel centerline melting temperature in the SE on WCAP-17642-P-A is applicable to Surry because the properties of the fuel design in use at Surry Units 1 and 2 are consistent with the fuel property data used to

generate the fuel centerline melting temperature. Therefore, the NRC staff determined that the proposed SL is conservative for Surry.

In the licensee's UFSAR Section 3.4.1.1 (ADAMS Accession no. ML21208A424), the licensee discusses the basis for the existing Surry SL. Section 3.4.1.1 states that the thermal-hydraulic design assures that the maximum fuel temperature is below UO<sub>2</sub> melting point of 5080 °F for unirradiated fuel, which decreases by 58 °F per 10,000 MWD/MTU. This UFSAR section also states that a calculated fuel centerline temperature of 4700 °F has been selected as the overpower limit. This conservative limit assures that the applicable safety analyses demonstrates that the proposed TS SL is not exceeded. Additionally, the licensee stated in its supplement dated October 13, 2021 (as affirmed by letter, dated October 19, 2021) that UFSAR Section 3.4.1.1 will be updated to describe the new TS SL and its technical basis. The licensee also stated in its supplement dated September 28, 2021 (as affirmed by letter, dated October 19, 2021) that the UFSAR Chapter 14 safety analysis, will be updated, as required during the amendment's implementation period. Based on the above, including the LAR information that supports the use of the empirically derived and less restrictive temperature, the NRC staff concludes that the proposed increase in the peak fuel centerline temperature TS SL is consistent with the 10 CFR 50.36(b) requirement that TSs be derived from licensee safety analyses, as amended.

The proposed TS change meets the functional requirements of the 1967 Proposed Draft GDC 6 and meets the requirements of Section 182a of the Atomic Energy Act, because (1) the peak fuel centerline temperature is based on a conservative evaluation of test data that is applicable to the fuel design used at Surry and (2) the proposed limit will ensure that fuel melt is precluded during conditions of normal operations and under anticipated operational occurrences. Therefore, the NRC Staff concludes that the proposed TS change meets 10 CFR 50.36(c)(1)(i)(A) because the revised TS limits an important variable that is necessary to reasonably protect the integrity of a physical barrier that guards against the uncontrolled release of radioactivity, and is acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the NRC staff notified the Commonwealth of Virginia State official of the proposed issuance of the amendment on July 7, 2021 (ADAMS Accession no. ML21270A023). On July 7, 2021, the State official confirmed the Commonwealth of Virginia had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change the requirements for the installation or use of facility components located within the restricted area as defined in (10 CFR) Part 20, "Standards for protection against radiation." The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* (85 FR 85678, December 29, 2020). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that public health and safety will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to public health and safety.

Principal Contributor: Joshua Miller

Dated: October 26, 2021



SUBJECT: SURRY POWER STATION, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 305 AND 305 RE: REVISED REACTOR CORE SAFETY LIMIT TO REFLECT TOPICAL REPORT WCAP-177642-P-A, REVISION 1 (EPID L-2020-LLA-0219) DATED OCTOBER 26, 2021

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NAME	VThomas	KGoldstein	RPatton
DATE	07/07/2021	07/22/2021	06/29/2021
OFFICE	NRR/DSS/STSB/BC (A)	OGC (NLO)	NRR/DORL/LPL2-1/BC
NAME	NJordan	MYoung	MMarkley (GEMiller for)
DATE	10/20/2021	10/20/2021	10/22/2021
OFFICE	NRR/DORL/LPL2-1/PM		
NAME	JKlos		
DATE	10/26/2021		

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