

Michael J. Yox Regulatory Affairs Director Vogtle 3 & 4 7825 River Road Waynesboro, GA 30830 706-848-6459 tel

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Docket Nos.:

52-025

52-026

ND-21-0552 10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555-0001

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3 and Unit 4
Resubmittal of Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load
Item 2.1.03.07.i [Index Number 78]

Ladies and Gentlemen:

Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of June 2, 2021, Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4 Uncompleted Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Item 2.1.03.07.i [Index Number 78] has not been completed greater than 225-days prior to initial fuel load. The Enclosure describes the plan for completing this ITAAC. Southern Nuclear Operating Company will, at a later date, provide additional notifications for Unit 4 ITAAC that have not been completed 225-days prior to initial fuel load.

Southern Nuclear Operating Company (SNC) previously submitted Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load for Item 2.1.03.07.i [Index Number 78] ND-18-0709 [ML18179A080], dated June 27, 2018. This resubmittal supersedes ND-18-0709 in its entirety and incorporates changes approved in Unit 3/Unit 4 license amendments 151/150.

This notification is informed by the guidance described in NEI 08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact Kelli Roberts at 706-848-6991.

Respectfully submitted,

Regulatory Affairs Director Vogtle 3 & 4

Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4

Completion Plan for Uncompleted ITAAC 2.1.03.07.i [Index Number 78]

MJY/DLW/sfr

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To:

Southern Nuclear Operating Company/ Georgia Power Company

Mr. Peter P. Sena III

Mr. D. L. McKinney

Mr. M. D. Meier

Mr. G. Chick

Mr. S. Stimac

Mr. P. Martino

Mr. M. J. Yox

Mr. A. S. Parton

Ms. K. A. Roberts

Mr. C. T. Defnall

Mr. C. E. Morrow

Mr. J. M. Fisher

Mr. R. L. Beilke

Mr. S. Leighty

Ms. A. C. Chamberlain

Mr. J. C. Haswell

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cc:

Nuclear Regulatory Commission

Ms. M. Bailey

Mr. M. King

Mr. G. Bowman

Ms. A. Veil

Mr. C. P. Patel

Mr. G. J. Khouri

Mr. C. J. Even

Mr. B. J. Kemker

Ms. N. C. Coovert

Mr. C. Welch

Mr. J. Gaslevic

Mr. O. Lopez-Santiago

Mr. G. Armstrong

Mr. M. Webb

Mr. T. Fredette

Mr. C. Santos

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Mr. J. Eargle

Mr. E. Davidson

Mr. T. Fanelli

Ms. K. McCurry

Mr. J. Parent

Mr. B. Griman

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Oglethorpe Power Corporation

Mr. R. B. Brinkman Mr. E. Rasmussen

Municipal Electric Authority of Georgia

Mr. J. E. Fuller Mr. S. M. Jackson

Dalton Utilities

Mr. T. Bundros

Westinghouse Electric Company, LLC

Dr. L. Oriani

Mr. D. C. Durham

Mr. M. M. Corletti

Mr. Z. S. Harper

Mr. J. L. Coward

Other

Mr. S. W. Kline, Bechtel Power Corporation

Ms. L. Matis, Tetra Tech NUS, Inc.

Dr. W. R. Jacobs, Jr., Ph.D., GDS Associates, Inc.

Mr. S. Roetger, Georgia Public Service Commission

Mr. R. L. Trokey, Georgia Public Service Commission

Mr. K. C. Greene, Troutman Sanders

Mr. S. Blanton, Balch Bingham

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Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4 Completion Plan for Uncompleted ITAAC 2.1.03.07.i [Index Number 78]

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ITAAC Statement

Design Commitment

- 7. The reactor internals will withstand the effects of flow induced vibration.
- 10. The reactor lower internals assembly is equipped with holders for at least eight capsules for storing material surveillance specimens.

Inspections/Tests/Analyses

- i) Not Used per Amendment No. 151 [150].
- ii) A pre-test inspection, a flow test and a post-test inspection will be conducted on the as-built reactor internals.

Inspection of the reactor lower internals assembly for the presence of capsules will be performed.

Acceptance Criteria

- i) Not Used per Amendment No. 151 [150].
- ii) The as-built reactor internals have no observable damage or loose parts.

At least eight capsules are in the reactor lower internals assembly.

ITAAC Completion Description

This ITAAC requires that tests and inspections be performed and documented to ensure the reactor internals will withstand the effects of flow induced vibration and that the reactor lower internals assembly is equipped with holders for at least eight capsules for storing material surveillance specimens.

7.ii) The as-built reactor internals have no observable damage or loose parts.

A pre-Hot Functional Test (HFT) inspection is performed to document the initial condition of the reactor internals, including all major load-bearing elements that retain the position of the core support structure; lateral, vertical, and torsional restraints; locking and bolting components; contact surfaces; and the reactor vessel interior. Next, a flow test is conducted on the reactor vessel internals during the HFT. A post-HFT inspection is then performed on the reactor internals to check for observable damage or loose parts. These inspections are documented on data sheets included in the inspection procedures (References 1 and 2). The test and inspections, described in UFSAR Section 3.9.2.4, are performed consistent with the guidelines of Regulatory Guide 1.20, Rev 2.

The inspection results, as documented in the Unit 3 and Unit 4 inspection procedures (References 1 and 2), conclude the as-built reactor internals have no observable damage or loose parts.

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10. At least eight capsules are in the reactor lower internals assembly.

An inspection of the reactor lower internals assembly is performed to confirm that at least eight material surveillance specimen capsules are installed in holders for that purpose, in accordance with the installation manuals (References 3 and 4). These inspections are documented on data sheets included in the inspection procedures (References 1 and 2).

The inspection results, as documented in the Unit 3 and Unit 4 inspection procedures (References 1 and 2), conclude that at least eight capsules are in the reactor lower internals assembly.

References 1 and 2 provide evidence that the ITAAC Acceptance Criteria requirements are met:

- The as-built reactor internals have no observable damage or loose parts; and
- At least eight capsules are in the reactor lower internals assembly.

References 1 through 4 are available for NRC inspection as part of the Unit 3 and Unit 4 ITAAC 2.1.03.07.i Completion Packages (References 5 and 6, respectively).

List of ITAAC Findings

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review, which included now-consolidated ITAAC Indexes 79 and 85, found there are no relevant ITAAC findings associated with this ITAAC.

References (available for NRC inspection)

- 1. 3-RXS-ITPP-501, "Pre- and Post-Hot Functional Test Inspection of Reactor Vessel Internals"
- 2. 4-RXS-ITPP-501, "Pre- and Post-Hot Functional Test Inspection of Reactor Vessel Internals"
- 3. SV3-MI01-Z0M-001, "AP1000 RVI Installation Requirements Manual"
- 4. SV4-MI01-Z0M-001, "AP1000 RVI Installation Requirements Manual"
- 5. 2.1.03.07.i-U3-CP-Rev0, ITAAC Completion Package
- 6. 2.1.03.07.i-U4-CP-Rev0, ITAAC Completion Package
- 7. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"