UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
2100 RENAISSANCE BOULEVARD, SUITE 100
KING OF PRUSSIA, PENNSYLVANIA 19406-2713

June 3, 2021

Mr. David Rhoades
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Office
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – DESIGN BASIS ASSURANCE INSPECTION (PROGRAMS) INSPECTION REPORT 05000333/2021011

Dear Mr. Rhoades:

On May 20, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at James A. FitzPatrick Nuclear Power Plant and discussed the results of this inspection with Mr. Timothy Peter, Plant Manager, and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at http://www.nrc.gov/reading-rm/adams.html and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

X /RA/

Signed by: Melvin K. Gray

Mel Gray, Chief
Engineering Branch 1
Division of Operating Reactor Safety

Docket No. 05000333
License No. DPR-59

Enclosure:
As stated

cc w/encl: Distribution via LISTSERV®

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – DESIGN BASIS ASSURANCE INSPECTION (PROGRAMS) INSPECTION REPORT 05000333/2021011 DATED JUNE 3, 2021

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Number:	05000333
License Number:	DPR-59
Report Number:	05000333/2021011
Enterprise Identifier:	I-2021-011-0005
Licensee:	Exelon Nuclear
Facility:	James A. FitzPatrick Nuclear Power Plant
Location:	Oswego, NY
Inspection Dates:	May 03, 2021 to May 20, 2021
Inspectors:	A. Patel, Senior Reactor Inspector L. Dumont, Reactor Inspector J. Schoppy, Senior Reactor Inspector
Approved By:	Mel Gray, Chief Engineering Branch 1 Division of Operating Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a design basis assurance inspection (programs) inspection at James A. FitzPatrick Nuclear Power Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to https://www.nrc.gov/reactors/operating/oversight.html for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), regional inspectors were directed to begin telework. Regional based inspections were evaluated to determine if all or a portion of the objectives and requirements stated in the IP could be performed remotely. For the inspection documented below portions of the IP were completed remotely as well as on site and all the objectives and requirements for completion of the IP were met.

REACTOR SAFETY

71111.21N.02 - Design-Basis Capability of Power-Operated Valves Under 10 CFR 50.55a Requirements

POV Review (IP Section 03) (11 Samples)

The inspectors:

a. Determined whether the sampled POVs are being tested and maintained in accordance with NRC regulations along with the licensee's commitments and/or licensing bases.
b. Determined whether the sampled POVs are capable of performing their design-basis functions.
c. Determined whether testing of the sampled POVs is adequate to demonstrate the capability of the POVs to perform their safety functions under design-basis conditions.
d. Evaluate maintenance activities including a walkdown of the sampled POVs (if accessible).

(1) 10MOV-17, Residual Heat Removal Shutdown Cooling Outboard Isolation Valve
(2) 10MOV-34A, Residual Heat Removal 'A' Torus Cooling Valve Supply
(3) 10MOV-66B, Residual Heat Removal Heat Exchanger 'B' Bypass Valve
(4) 10MOV-89A, Residual Heat Removal Heat Exchanger 'A' Service Water Outlet Isolation Valve
(5) 12MOV-15, Reactor Water Cleanup Supply Inboard Isolation Valve
(6) 13MOV-21, Reactor Core Isolation Cooling Pump Discharge to Reactor Inboard Valve
(7) 23MOV-15, High Pressure Coolant Injection Steam Supply Inboard Isolation Valve
(8) 46MOV-101A, Emergency Service Water Loop 'A' Supply Header Isolation Valve
(9) 23AOV-42, High Pressure Coolant Injection Turbine Steam Supply Upstream Drain Isolation Valve

(10) 29AOV-80A, Main Steam Line 'A' Inboard Isolation Valve
(11) 11EV-14B, Standby Liquid Control 'B' Double Squib Activated Shear Explosive Valve

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On May 20, 2021, the inspectors presented the design basis assurance inspection (programs) results to Mr. Timothy Peter, Plant Manager, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.21N.02	Calculations	9663-JAF-CALC-0001	MOV Thermal Overload Sizing	Revision 2
		JAF-CALC-ELEC-02609	The 125VDC Station Battery 'A' Sizing & Voltage Drop	Revision 3
	Corrective Action Documents Resulting from Inspection	4421060		
		4421382		
		4421465		
		4421640		
		4421742		
		4421743		
		4421748		
		4421836		
		4421932		
		4421938		
		4422026		
		4422264		
		4422842		
		4424138		
		4424144		
		4424366		
		4424369		
		4424515		
		4424614		
	Engineering Evaluations	IST-2010-005	POV Stroke Time Acceptance Criteria IST Technical Position Paper	Revision 6
		JF-MISC-001	MOV Program Risk Ranking Input	Revision 2
	Procedures	ER-AA-302	Motor-Operated Valve Program Engineering Procedure	Revision 8
		ER-AA-410	Air Operated Valve Program Implementing Procedure	Revision 6