



May 18, 2021

United States Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D.C. 20555

Serial No.: 21-181  
NRA/GDM: R0  
Docket No.: 50-281  
License No.: DPR-37

**VIRGINIA ELECTRIC AND POWER COMPANY (DOMINION ENERGY VIRGINIA)**  
**SURRY POWER STATION UNIT 2**  
**CORE OPERATING LIMITS REPORT**  
**SURRY 2 CYCLE 30 PATTERN SYK REVISION 1**

Pursuant to Surry Power Station (Surry) Units 1 and 2 Technical Specification (TS) 6.2.C, attached is a copy of the Core Operating Limits Report (COLR) for Surry Power Station Unit 2, Cycle 30, Pattern SYK, Revision 1. The revision incorporates changes associated with the recently approved Surry Units 1 and 2 TS Amendments 303/303 that incorporate a new small break loss of coolant accident model into the COLR.

If you have any questions or require additional information, please contact Mr. Gary Miller at (804) 273-2771.

Sincerely,

A handwritten signature in black ink that reads "B E Standley".

B. E. Standley, Director  
Nuclear Regulatory Affairs  
Dominion Energy Services, Inc. for  
Virginia Electric and Power Company

Attachment: Core Operating Limits Report, Surry Unit 2 Cycle 30 Pattern SYK, Revision 1

Commitment Summary: There are no new commitments contained in this letter.

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**Attachment**

**CORE OPERATING LIMITS REPORT**

**Surry Unit 2 Cycle 30  
Pattern SYK**

**Revision 1**

## **1.0 INTRODUCTION**

This Core Operating Limits Report (COLR) for Surry Unit 2 Cycle 30 has been prepared in accordance with the requirements of Surry Technical Specification 6.2.C.

The Technical Specifications affected by this report are:

TS 2.1 – Safety Limit, Reactor Core  
TS 2.3.A.2.d – Overtemperature  $\Delta T$   
TS 2.3.A.2.e – Overpower  $\Delta T$   
TS 3.1.E - Moderator Temperature Coefficient  
TS 3.12.A.1, TS 3.12.A.2, TS 3.12.A.3 and TS 3.12.C.3.b.1(b) - Control Bank Insertion Limits  
TS 3.12.A.1.a, TS 3.12.A.2.a, TS 3.12.A.3.c and TS 3.12.G – Shutdown Margin  
TS 3.12.B.1 and TS 3.12.B.2 - Power Distribution Limits (Heat Flux Hot Channel Factor and Nuclear Enthalpy Rise Hot Channel Factor)  
TS 3.12.F – DNB Parameters  
TS Table 4.1-2A – Minimum Frequency for Equipment Tests: Item 22 – RCS Flow

## **2.0 REFERENCES**

1. VEP-FRD-42-A, Revision 2, Minor Revision 2, “Reload Nuclear Design Methodology,” October 2017.  
Methodology for:  
TS 2.1 – Safety Limit, Reactor Core  
TS 3.1.E - Moderator Temperature Coefficient  
TS 3.12.A.1, TS 3.12.A.2, TS 3.12.A.3 and TS 3.12.C.3.b.1(b) - Control Bank Insertion Limit  
TS 3.12.A.1.a, TS 3.12.A.2.a, TS 3.12.A.3.c and TS 3.12.G – Shutdown Margin  
TS 3.12.B.1 and TS 3.12.B.2 - Heat Flux Hot Channel Factor and Nuclear Enthalpy Rise Hot Channel Factor  
TS 3.12.F – DNB Parameters  
TS Table 4.1-2A – Minimum Frequency for Equipment Tests: Item 22 – RCS Flow
2. WCAP-16009-P-A, “Realistic Large Break LOCA Evaluation Methodology Using the Automated Statistical Treatment of Uncertainty Method (ASTRUM),” (Westinghouse Proprietary), January 2005.  
Methodology for:  
TS 3.12.B.1 and TS 3.12.B.2 - Heat Flux Hot Channel Factor
3. EMF-2328(P)(A), “PWR Small Break LOCA Evaluation Model, S-RELAP5 Based,” as supplemented by ANP-3676P, “Surry Fuel-Vendor Independent Small Break LOCA Analysis,” as approved by NRC Safety Evaluation Report dated March 19, 2021.  
Methodology for:  
TS 3.12.B.1 and TS 3.12.B.2 - Heat Flux Hot Channel Factor

4. WCAP-12610-P-A, "VANTAGE+ Fuel Assembly Report," (Westinghouse Proprietary), April 1995.  
Methodology for:  
TS 3.12.B.1 and TS 3.12.B.2 - Heat Flux Hot Channel Factor
5. WCAP-12610-P-A and CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLO," (Westinghouse Proprietary), July 2006.  
Methodology for:  
TS 3.12.B.1 and TS 3.12.B.2 - Heat Flux Hot Channel Factor
6. VEP-NE-2-A, Rev. 0, "Statistical DNBR Evaluation Methodology," June 1987.  
Methodology for:  
TS 3.12.B.1 and TS 3.12.B.2 - Nuclear Enthalpy Rise Hot Channel Factor
7. DOM-NAF-2-P-A, Revision 0, Minor Revision 3, "Reactor Core Thermal-Hydraulics Using the VIPRE-D Computer Code," including Appendix B, "Qualification of the Westinghouse WRB-1 CHF Correlation in the Dominion VIPRE-D Computer Code," and Appendix D, "Qualification of the ABB-NV and WLOP CHF Correlations in the Dominion VIPRE-D Computer Code," September 2014.  
Methodology for:  
TS 3.12.B.1 and TS 3.12.B.2 - Nuclear Enthalpy Rise Hot Channel Factor
8. WCAP-8745-P-A, "Design Bases for Thermal Overpower Delta-T and Thermal Overtemperature Delta-T Trip Function," September 1986.  
Methodology for:  
TS 2.3.A.2.d – Overtemperature  $\Delta T$   
TS 2.3.A.2.e – Overpower  $\Delta T$

### 3.0 OPERATING LIMITS

The cycle-specific parameter limits for the specifications listed in Section 1.0 are presented in the following subsections. These limits have been developed using the NRC-approved methodologies specified in Technical Specification 6.2.C and repeated in Section 2.0.

#### 3.1 Safety Limit, Reactor Core (TS 2.1)

The Reactor Core Safety Limits are presented in **Figure A-1**.

#### 3.2 Overtemperature $\Delta T$ (TS 2.3.A.2.d)

$$\Delta T \leq \Delta T_0 [K_1 - K_2 \left( \frac{1+t_1s}{1+t_2s} \right) (T - T') + K_3 (P - P') - f(\Delta I)]$$

Where:

$\Delta T$  is measured RCS  $\Delta T$ , °F.

$\Delta T_0$  is the indicated  $\Delta T$  at RATED POWER, °F.

$s$  is the Laplace transform operator,  $\text{sec}^{-1}$ .

$T$  is the measured RCS average temperature ( $T_{\text{avg}}$ ), °F.

$T'$  is the nominal  $T_{\text{avg}}$  at RATED POWER,  $\leq 573.0^\circ\text{F}$ .

$P$  is the measured pressurizer pressure, psig.

$P'$  is the nominal RCS operating pressure  $\geq 2235$  psig.

$$K_1 \leq 1.1425$$

$$K_2 \geq 0.01059 / ^\circ\text{F}$$

$$K_3 \geq 0.000765 / \text{psig}$$

$$t_1 \geq 29.7 \text{ seconds}$$

$$t_2 \leq 4.4 \text{ seconds}$$

$$f(\Delta I) \geq 0.0268 \{-24 - (q_t - q_b)\}, \text{ when } (q_t - q_b) < -24.0\% \text{ RATED POWER}$$

$$0, \text{ when } -24.0\% \text{ RATED POWER} \leq (q_t - q_b) \leq +8.0\% \text{ RATED POWER}$$

$$0.0188 \{(q_t - q_b) - 8.0\}, \text{ when } (q_t - q_b) > +8.0\% \text{ RATED POWER}$$

Where  $q_t$  and  $q_b$  are percent RATED POWER in the upper and lower halves of the core, respectively, and  $q_t + q_b$  is the total THERMAL POWER in percent RATED POWER.

### 3.3 Overpower $\Delta T$ (TS 2.3.A.2.e)

$$\Delta T \leq \Delta T_0 \left[ K_4 - K_5 \left( \frac{t_3 s}{1 + t_3 s} \right) T - K_6 (T - T') - f(\Delta I) \right]$$

Where:

$\Delta T$  is measured RCS  $\Delta T$ , °F.

$\Delta T_0$  is the indicated  $\Delta T$  at RATED POWER, °F.

$s$  is the Laplace transform operator,  $\text{sec}^{-1}$ .

$T$  is the measured RCS average temperature ( $T_{\text{avg}}$ ), °F.

$T'$  is the nominal  $T_{\text{avg}}$  at RATED POWER,  $\leq 573.0^\circ\text{F}$ .

$$K_4 \leq 1.0965$$

$$K_5 \geq 0.0198 / ^\circ\text{F for increasing } T_{\text{avg}}$$

$$K_6 \geq 0.001074 / ^\circ\text{F for } T > T'$$

$$\geq 0 / ^\circ\text{F for decreasing } T_{\text{avg}}$$

$$\geq 0 \text{ for } T \leq T'$$

$$t_3 \geq 9.0 \text{ seconds}$$

$$f(\Delta I) = \text{as defined above for OT}\Delta T$$

### 3.4 Moderator Temperature Coefficient (TS 3.1.E)

The Moderator Temperature Coefficient (MTC) limits are:

+6.0 pcm/°F at less than 50 percent of RATED POWER, and

+6.0 pcm/°F at 50 percent of RATED POWER and linearly decreasing to 0 pcm/°F at RATED POWER

### 3.5 Control Bank Insertion Limits (TS 3.12.A.1, TS 3.12.A.2, TS 3.12.A.3, and TS 3.12.C.3.b.1(b))

3.5.1 The control rod banks shall be limited in physical insertion as shown in **Figure A-2**.

3.5.2 The rod insertion limit for the A and B control banks is the fully withdrawn position as shown on **Figure A-2**.

3.5.3 The rod insertion limit for the A and B shutdown banks is the fully withdrawn position as shown on **Figure A-2**.

### 3.6 Shutdown Margin (TS 3.12.A.1.a, TS 3.12.A.2.a, TS 3.12.A.3.c and TS 3.12.G)

Shutdown margin (SDM) shall be  $\geq 1.77 \% \Delta k/k$ .

**3.7 Power Distribution Limits** (TS 3.12.B.1 and TS 3.12.B.2)

**3.7.1 Heat Flux Hot Channel Factor - FQ(z)**

$$FQ(z) \leq \frac{CFQ}{P} K(z) \text{ for } P > 0.5$$

$$FQ(z) \leq \frac{CFQ}{0.5} K(z) \text{ for } P \leq 0.5$$

$$\text{where: } P = \frac{\text{THERMAL POWER}}{\text{RATED POWER}}$$

$$CFQ = 2.5$$

$$K(z) = 1.0 \text{ for all core heights, } z$$

**3.7.2 Nuclear Enthalpy Rise Hot Channel Factor - FΔH(N)**

$$F\Delta H(N) \leq CFDH * \{1 + PFDH(1 - P)\}$$

$$\text{where: } P = \frac{\text{THERMAL POWER}}{\text{RATED POWER}}$$

$$CFDH = 1.635$$

$$PFDH = 0.3$$

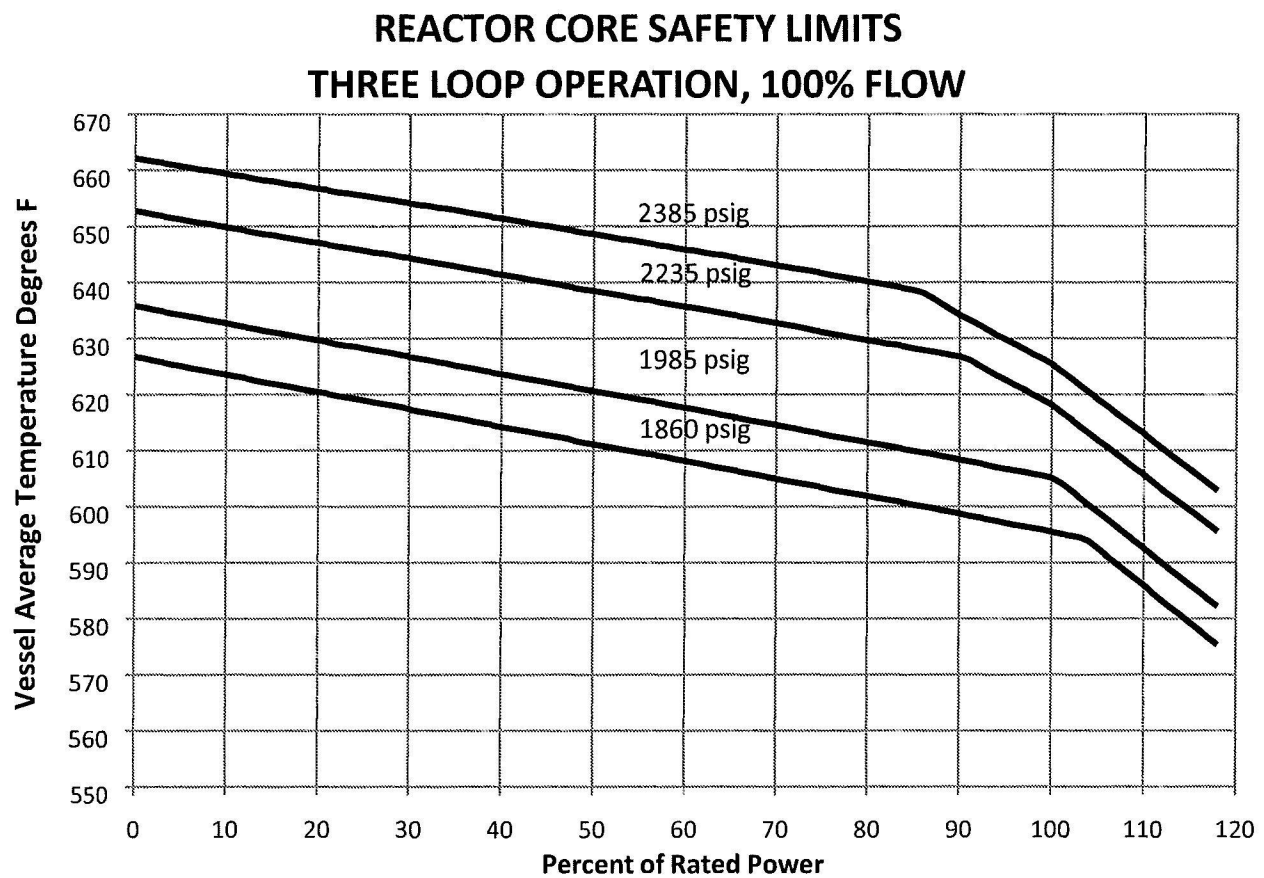
**3.8 DNB Parameters** (TS 3.12.F and TS Table 4.1-2A)

Departure from Nucleate Boiling (DNB) Parameters shall be maintained within their limits during POWER OPERATION:

- Reactor Coolant System  $T_{\text{avg}} \leq 577.0^\circ\text{F}$
- Pressurizer Pressure  $\geq 2205 \text{ psig}$
- Reactor Coolant System Total Flow Rate  $\geq 273,000 \text{ gpm}$  (Tech Spec Limit)  
and  $\geq 274,000 \text{ gpm}$  (COLR Limit)



**Figure A-1**



**Figure A-2**  
**Surry 2 Cycle 30**  
**Rod Group Insertion Limits**

Max w/d position = 227 steps

