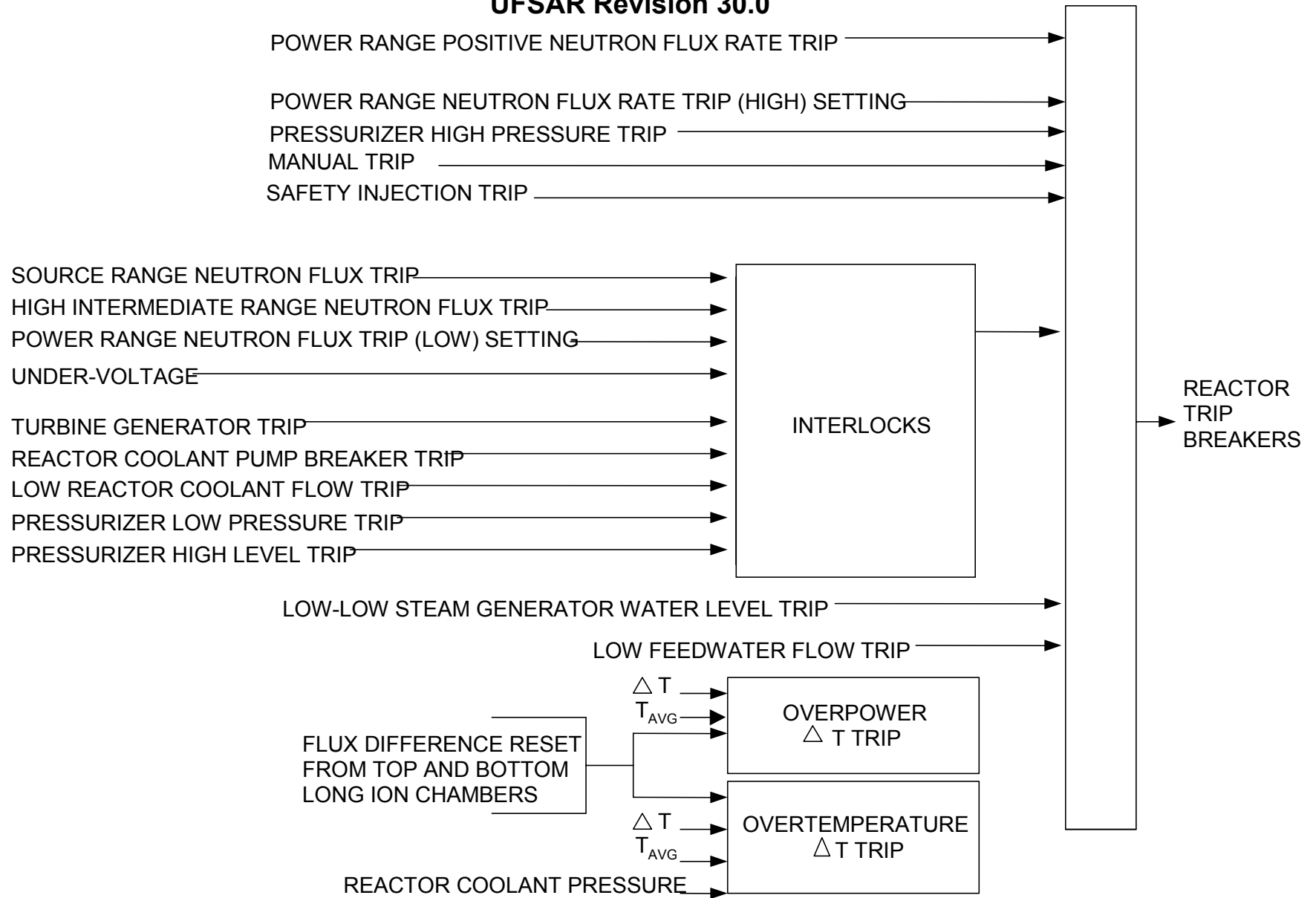
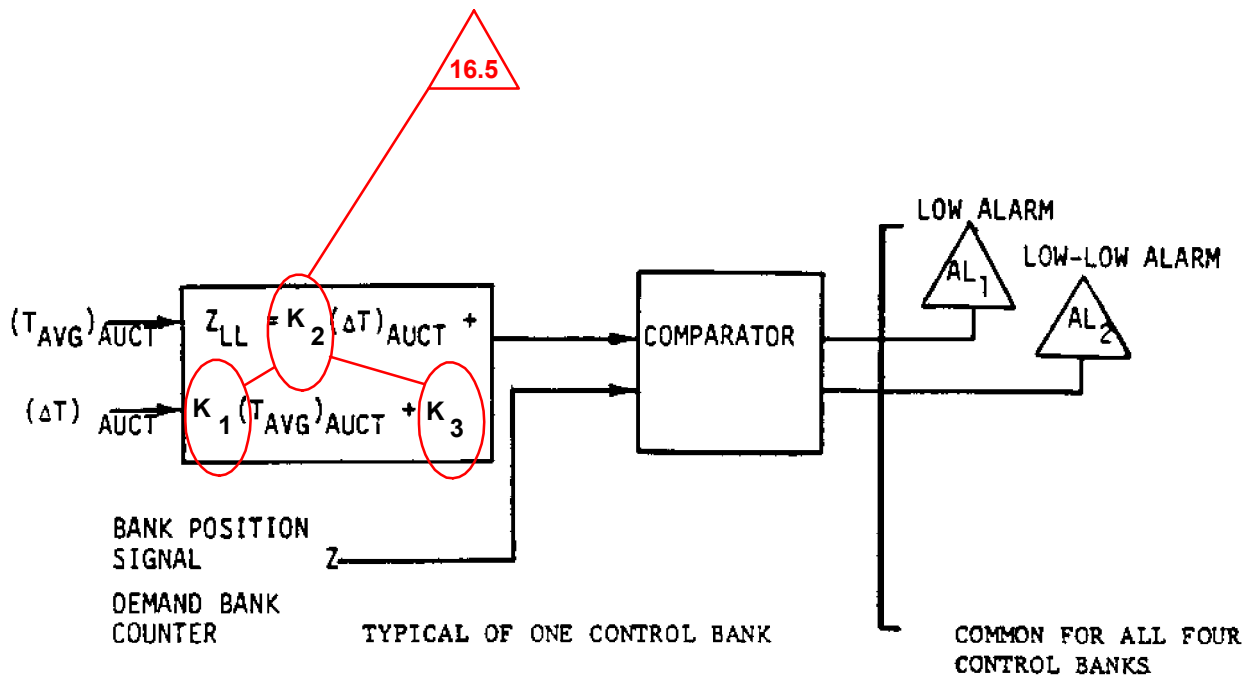


UFSAR Revision 30.0



Revision 21	Change Description: UCR-1852		
AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN		Title: REACTOR PROTECTION SYSTEMS	
DWG. NO. UFSAR FIG 7.2-2			Sheet 1 of 1

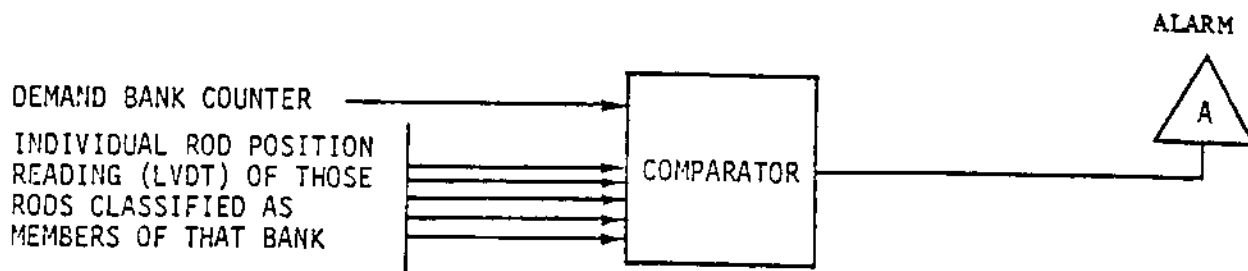
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NOTE :

1. ANALOG CIRCUITRY IS USED FOR THE COMPARATOR NETWORK.
2. COMPARISON IS DONE FOR ALL CONTROL BANKS.

16.5	REVISED PER 98-UFSAR-375				
REV. NO.	DESCRIPTION				
REVISIONS					
AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN	<table style="width: 100%; border-collapse: collapse;"> <tr> <td style="width: 10%; padding: 5px; vertical-align: top;">TITLE</td> <td style="padding: 5px;">CONTROL ROD BANK INSERTION MONITOR</td> </tr> <tr> <td style="padding: 5px;">DWG. NO.</td> <td style="padding: 5px;">FSAR FIG. 7.2-3</td> </tr> </table>	TITLE	CONTROL ROD BANK INSERTION MONITOR	DWG. NO.	FSAR FIG. 7.2-3
TITLE	CONTROL ROD BANK INSERTION MONITOR				
DWG. NO.	FSAR FIG. 7.2-3				
SH 1 of 1					



NOTE:

1. DIGITAL OR ANALOG SIGNALS MAY BE USED FOR THE COMPARATOR COMPUTER INPUTS.
2. THE COMPARATOR WILL ENERGIZE THE ALARM IF THERE EXISTS A POSITION DIFFERENCE GREATER THAN A PRESET LIMIT BETWEEN ANY INDIVIDUAL ROD AND THE DEMAND BANK COUNTER.
3. COMPARISON IS INDIVIDUALLY DONE FOR ALL CONTROL BANKS.

Ch. I (Typ. for Ch II & III)

Ch. IV

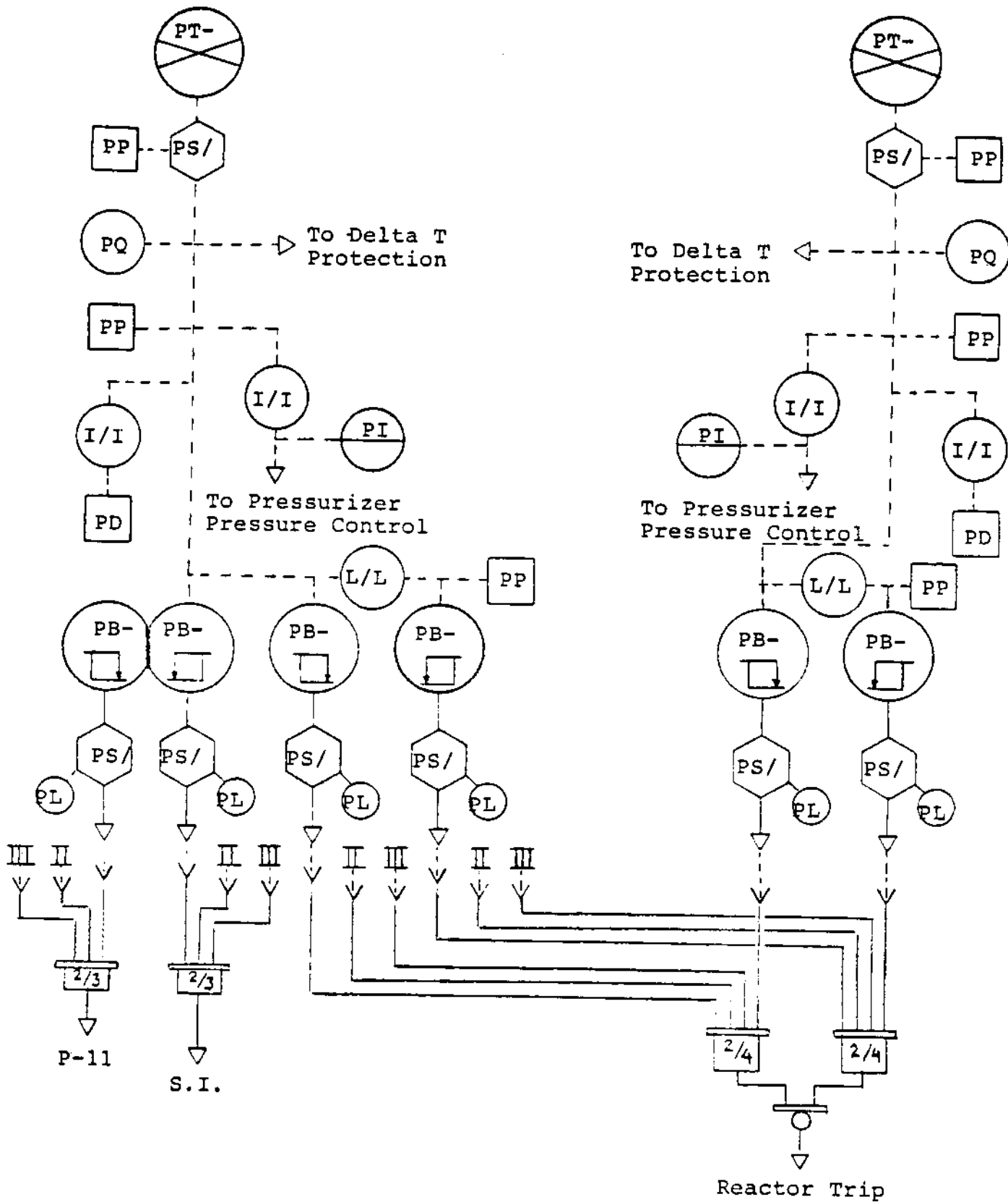


Fig. 7.2-5
PRESSURIZER PRESSURE PROTECTION

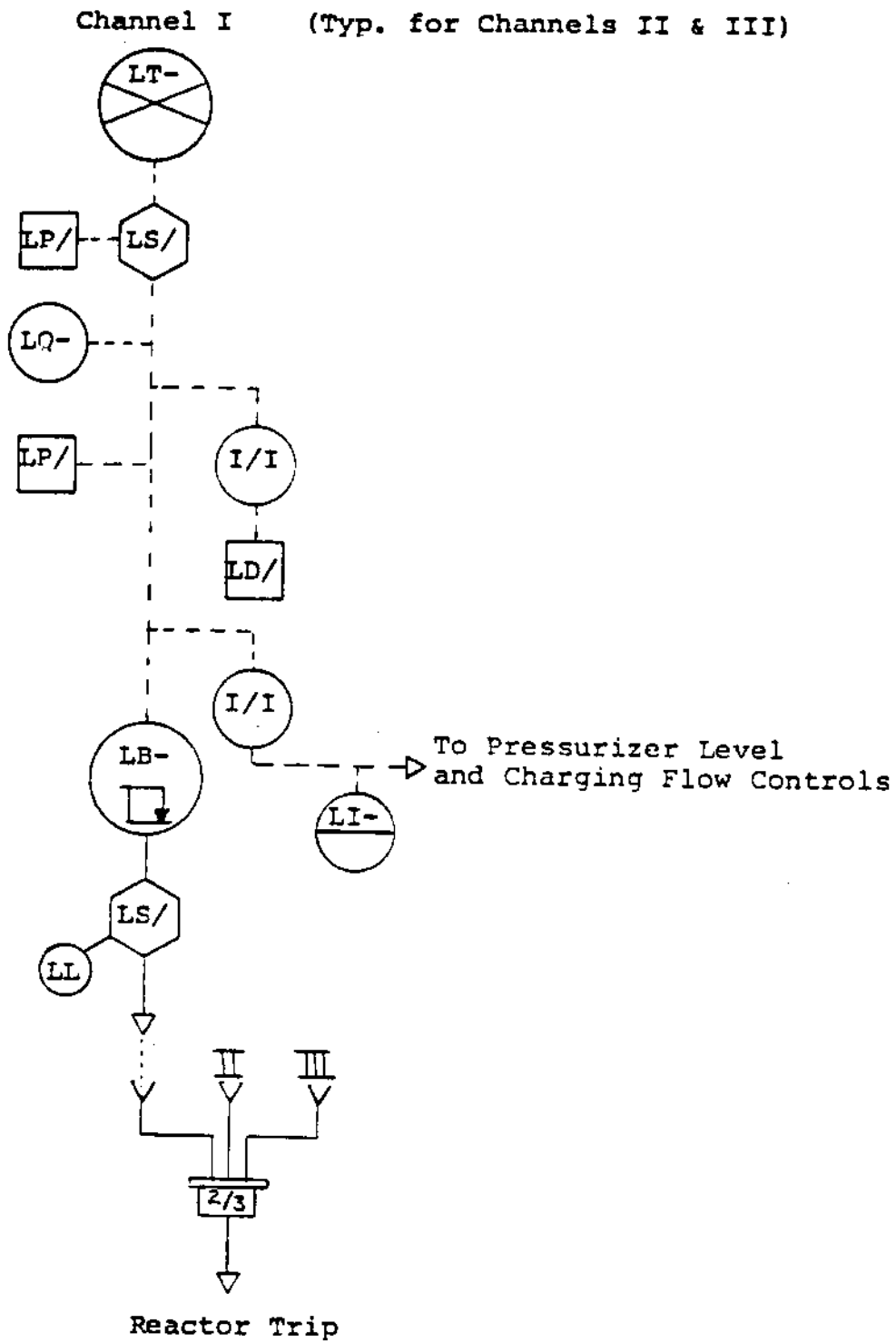
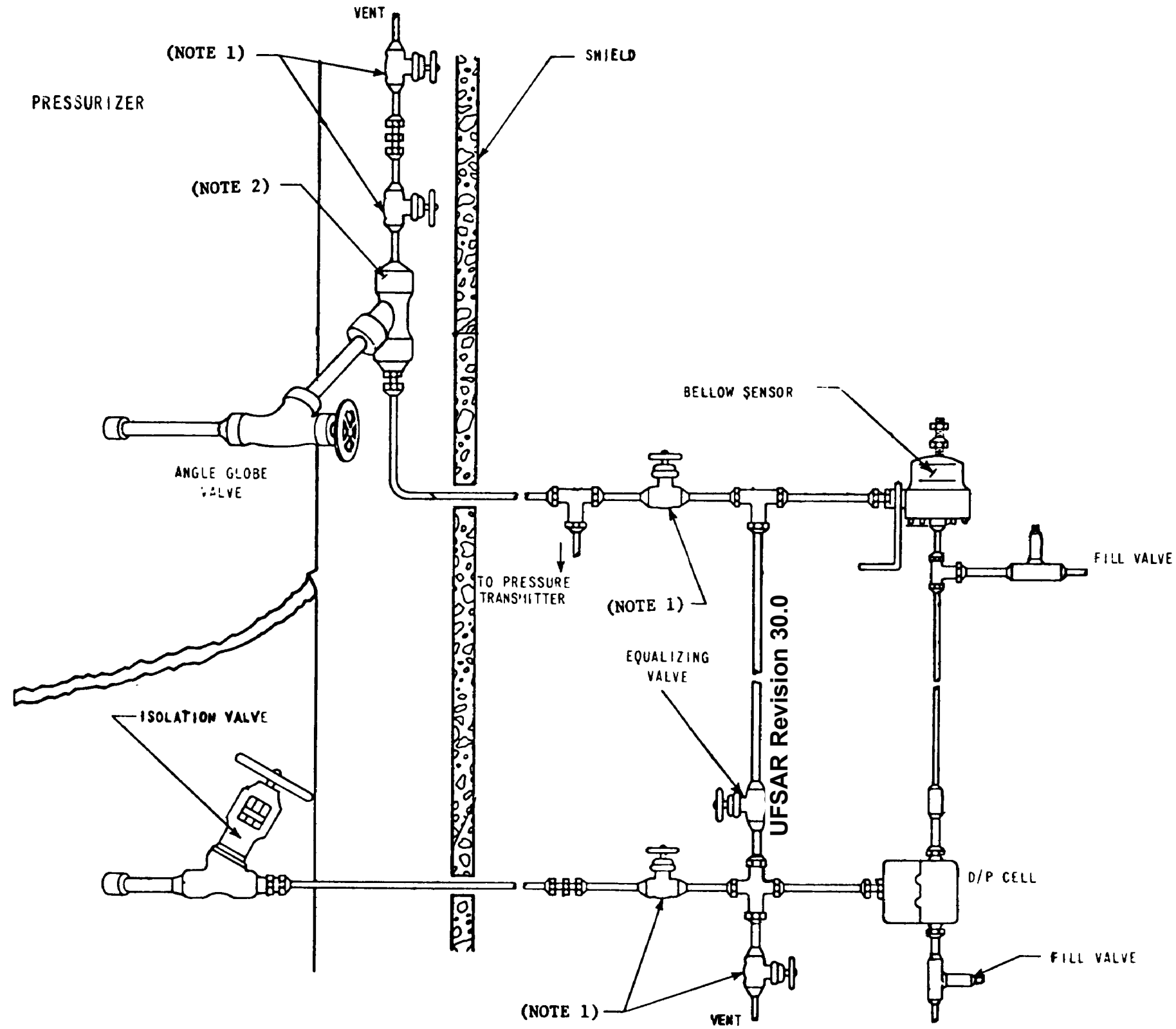


Fig. 7.2-6
Pressurizer Level Protection

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NOTES:

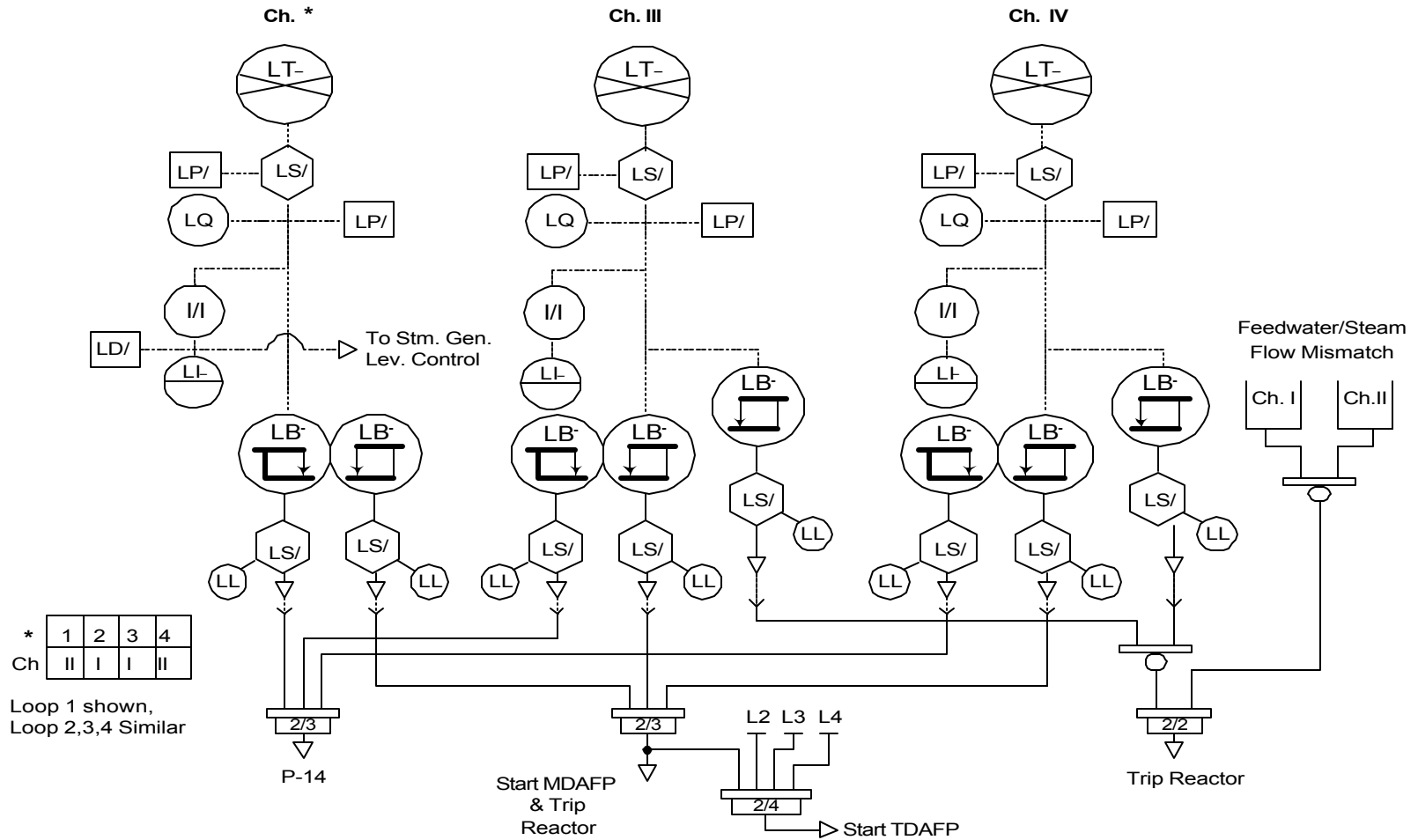
- 1. INSTRUMENT SHUT-OFF AND VENT
- 2. CONDENSATE RESERVOIR

PRESSURIZER SEALED REFERENCE
LEG LEVEL SYSTEM

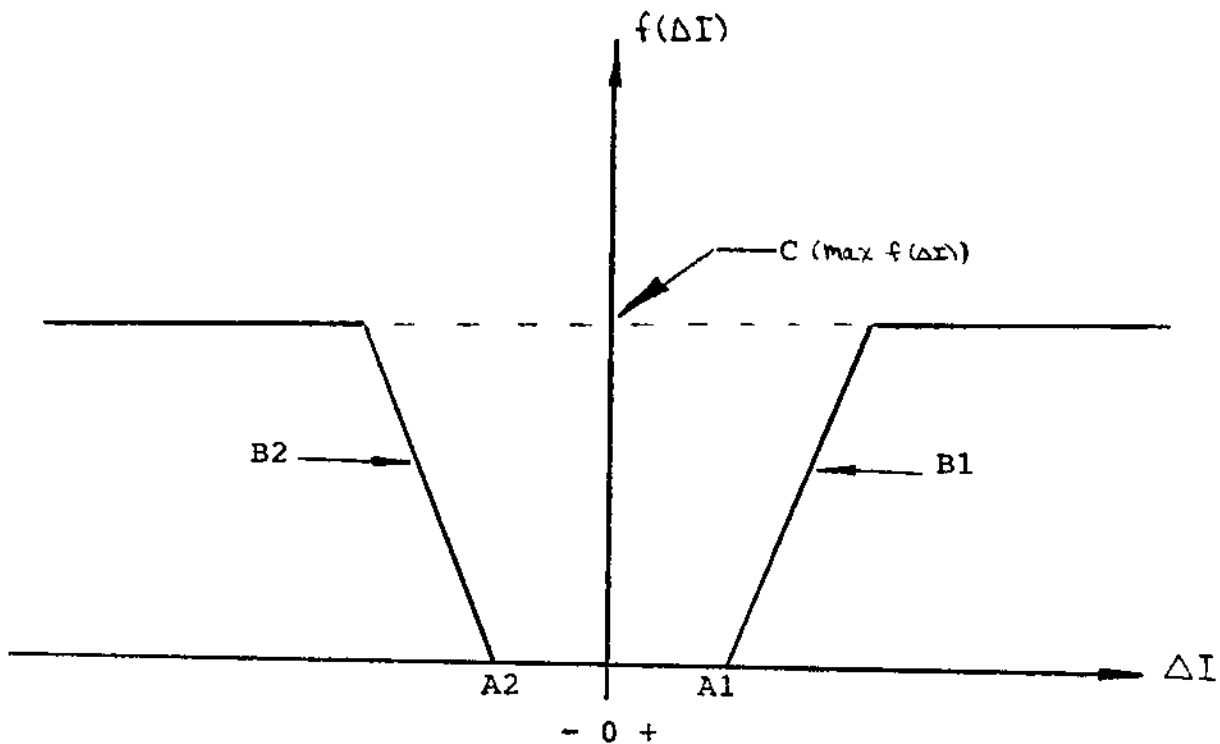
FIGURE 7.2-7

July, 1982

UFSAR Revision 30.0



Revision 17	Change Description: UCR-1181		
AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN		Title: STEAM GENERATOR LEVEL PROTECTION	
DWG. NO. UFSAR FIG 7.2-8			Sheet 1 of 1



ΔI = Difference between upper and lower long ion chambers.
($q_t - q_b$)

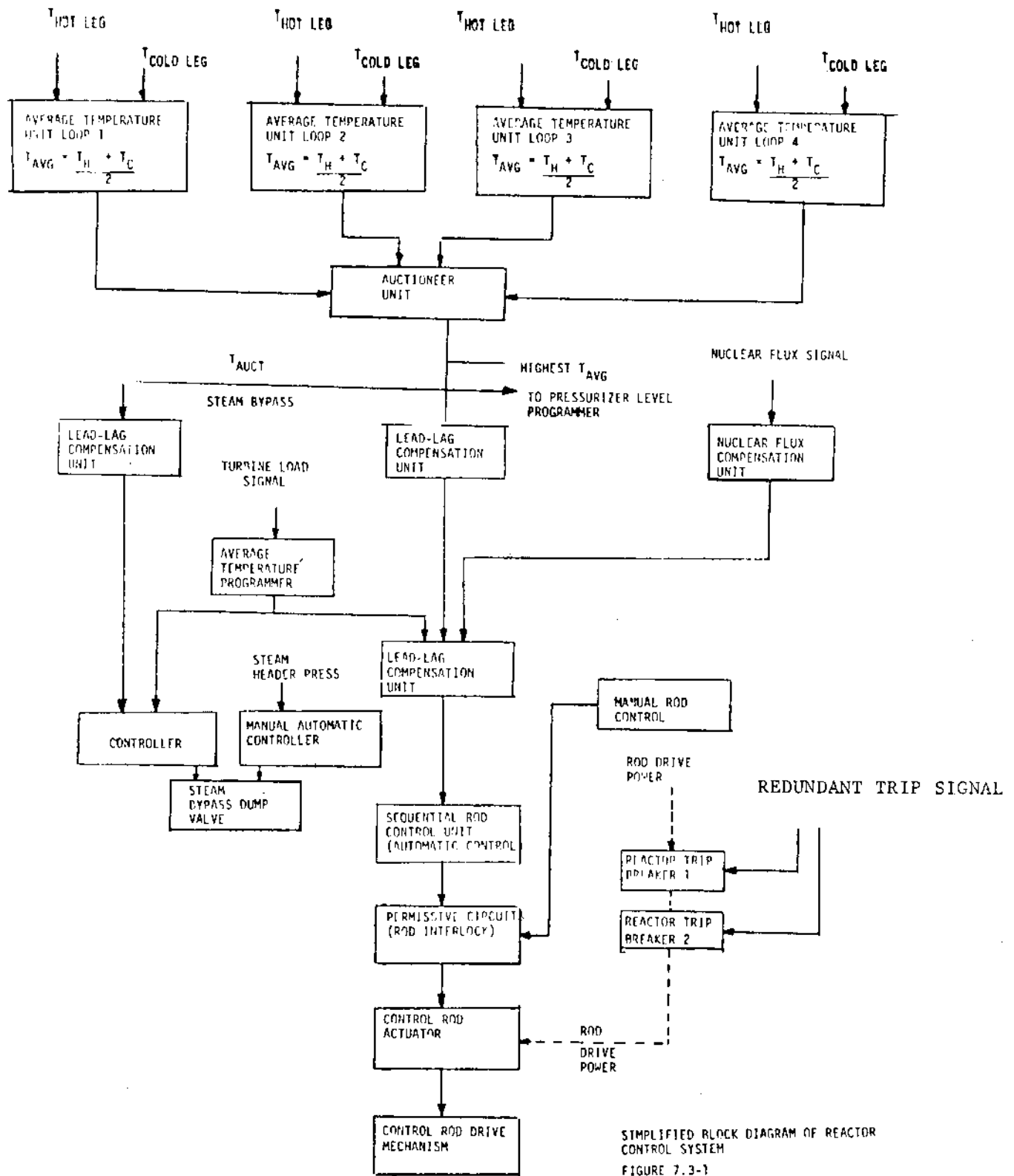
1. For $q_t - q_b$ between A2% and A1%, $f(\Delta I) = 0$
Where q_t and q_b are percent Rated Thermal Power in the top and bottom halves of the core respectively, and $q_t + q_b$ is the total thermal power in percent of Rated Thermal Power.
2. For each % that the magnitude of $(q_t - q_b)$ exceeds A2%, the ΔT trip setpoint is automatically reduced by B2% of its value at Rated Thermal Power.
3. For each % that the magnitude of $(q_t - q_b)$ exceeds A1%, the ΔT trip setpoint is automatically reduced by B1% of its value at Rated Thermal Power

SETPOINT REDUCTION FUNCTION FOR OVERPOWER AND OVERTEMPERATURE

DELTA-T TRIPS

FIGURE 7.2-9

- NOTES: 1) Temperatures are measured at steam generator's inlet and outlet.
 2) Pressure is measured at the pressurizer.



SIMPLIFIED BLOCK DIAGRAM OF REACTOR CONTROL SYSTEM
 FIGURE 7.3-1

July , 1982

UFSAR Revision 30.0

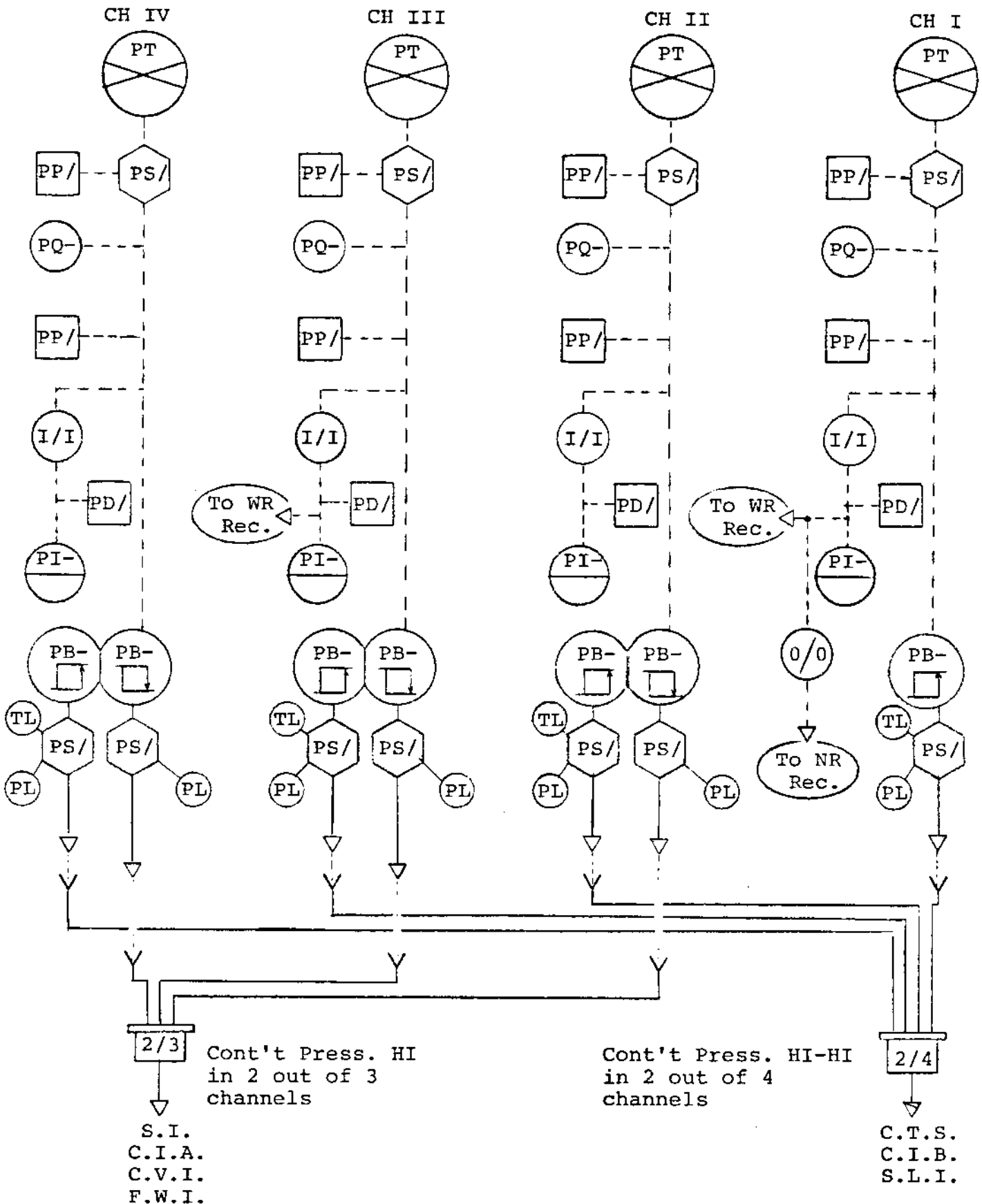


Fig. 7.5-1

CONTAINMENT PRESSURE PROTECTION

for definitions
see Table 7.2-4
July, 1982

UFSAR Revision 30.0

ENVIRONMENTAL CONDITIONS INSIDE CONTAINMENT

LOSS-OF-COOLANT ACCIDENT

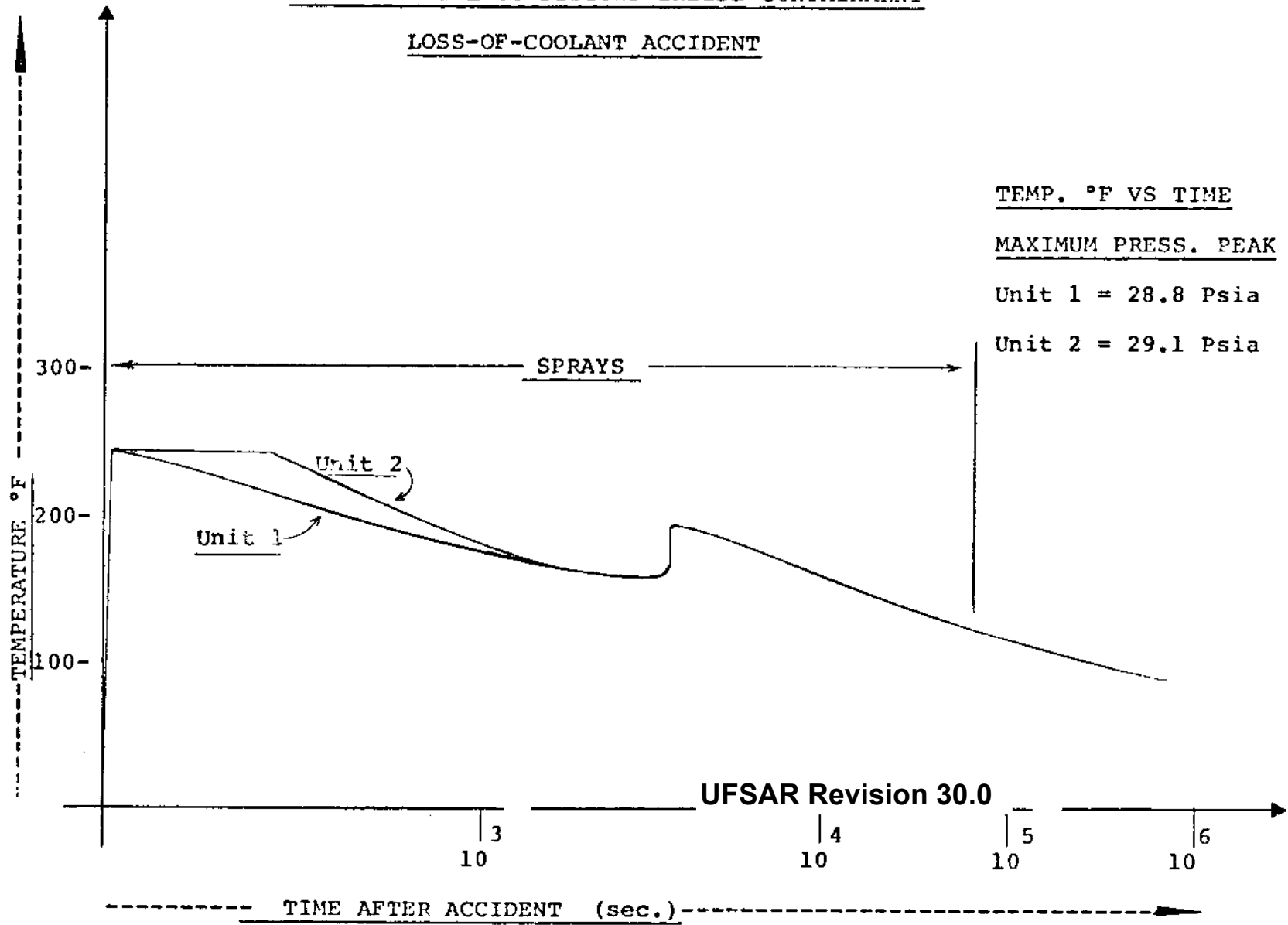


FIGURE 7.5-2

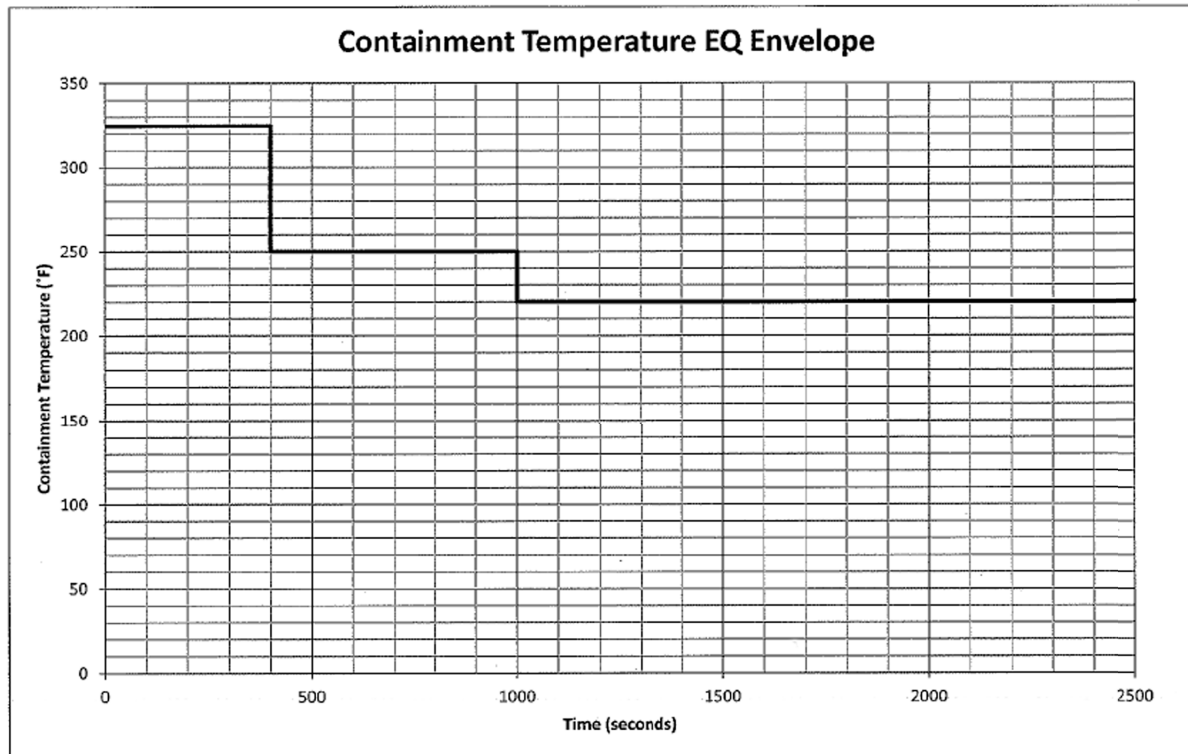
July, 1982

UFSAR Revision 30.0



INDIANA MICHIGAN POWER D. C. COOK NUCLEAR PLANT UPDATED FINAL SAFETY ANALYSIS REPORT

Revised: 27.0
Chapter: 7
Sheet: 1 of 1



UFSAR Figure: 7.5-3A

Change Description:
UCR-2054, Rev. 0

Unit 1

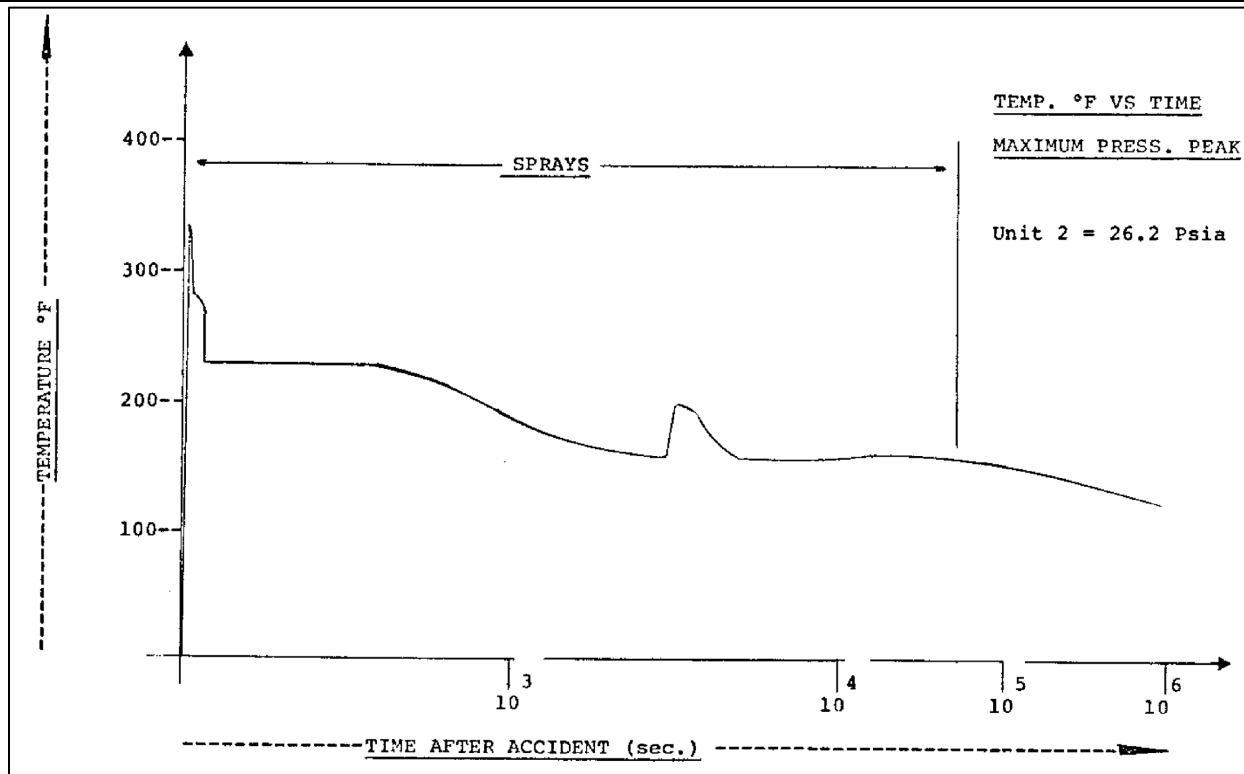
Title: Environmental Conditions Inside Containment - Main Steam Break (Unit 1)

UFSAR Revision 30.0



INDIANA MICHIGAN POWER D. C. COOK NUCLEAR PLANT UPDATED FINAL SAFETY ANALYSIS REPORT

Revised: 27.0
Chapter: 7
Sheet: 1 of 1

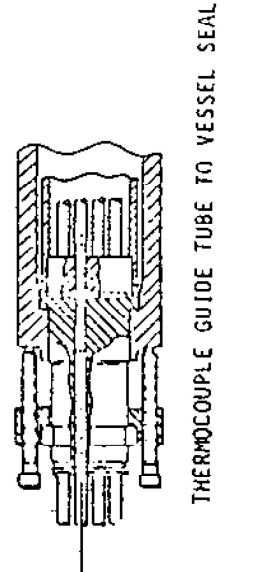
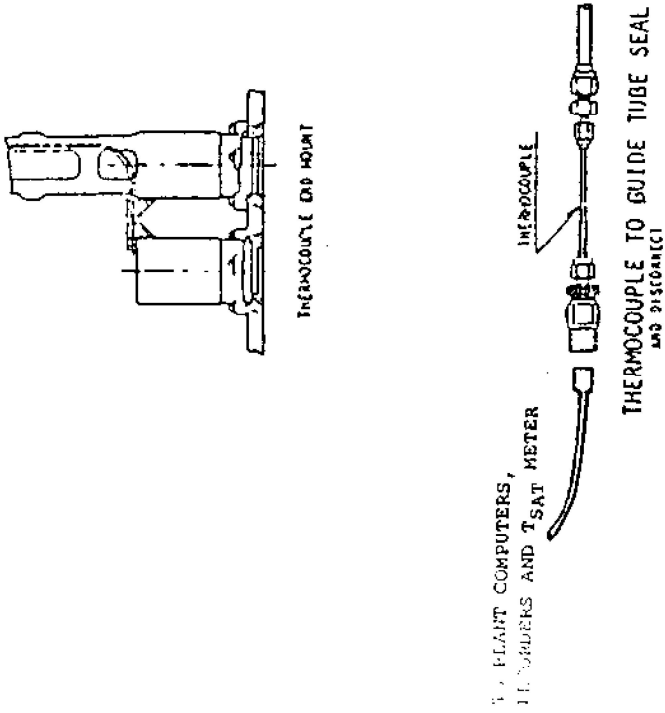
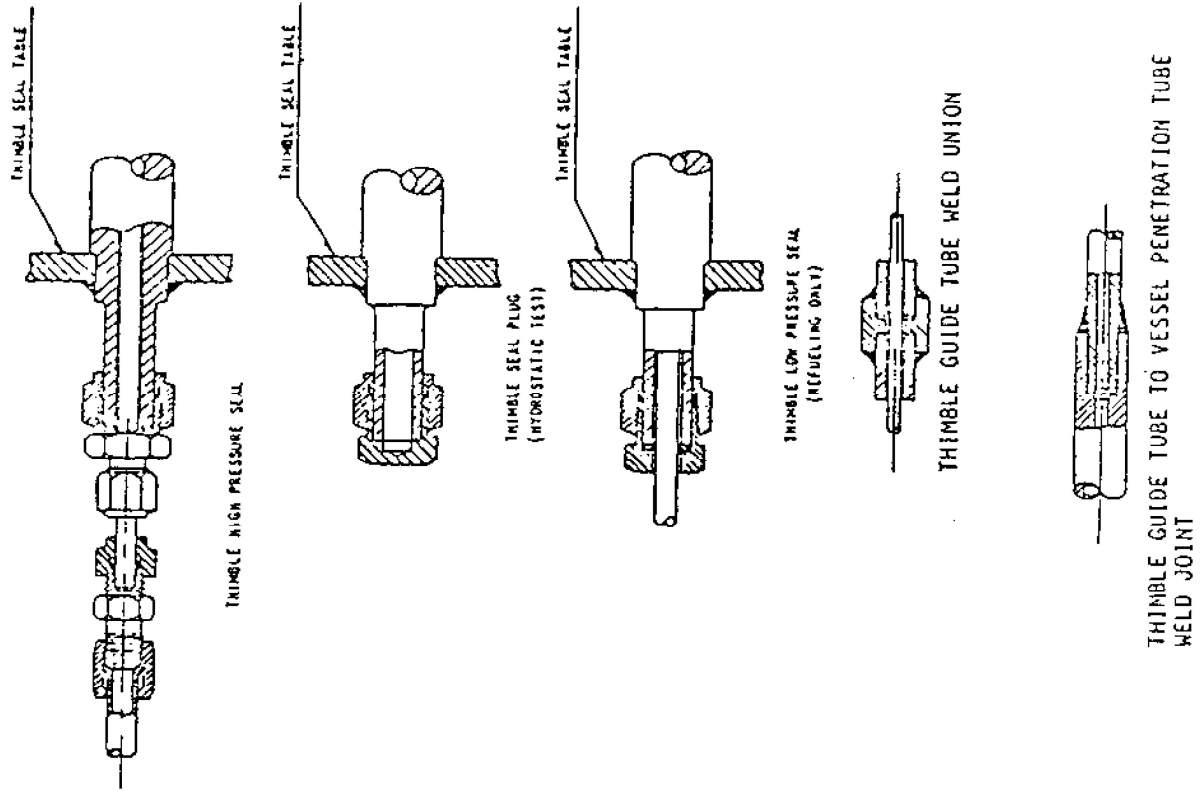


UFSAR Figure: 7.5-3B

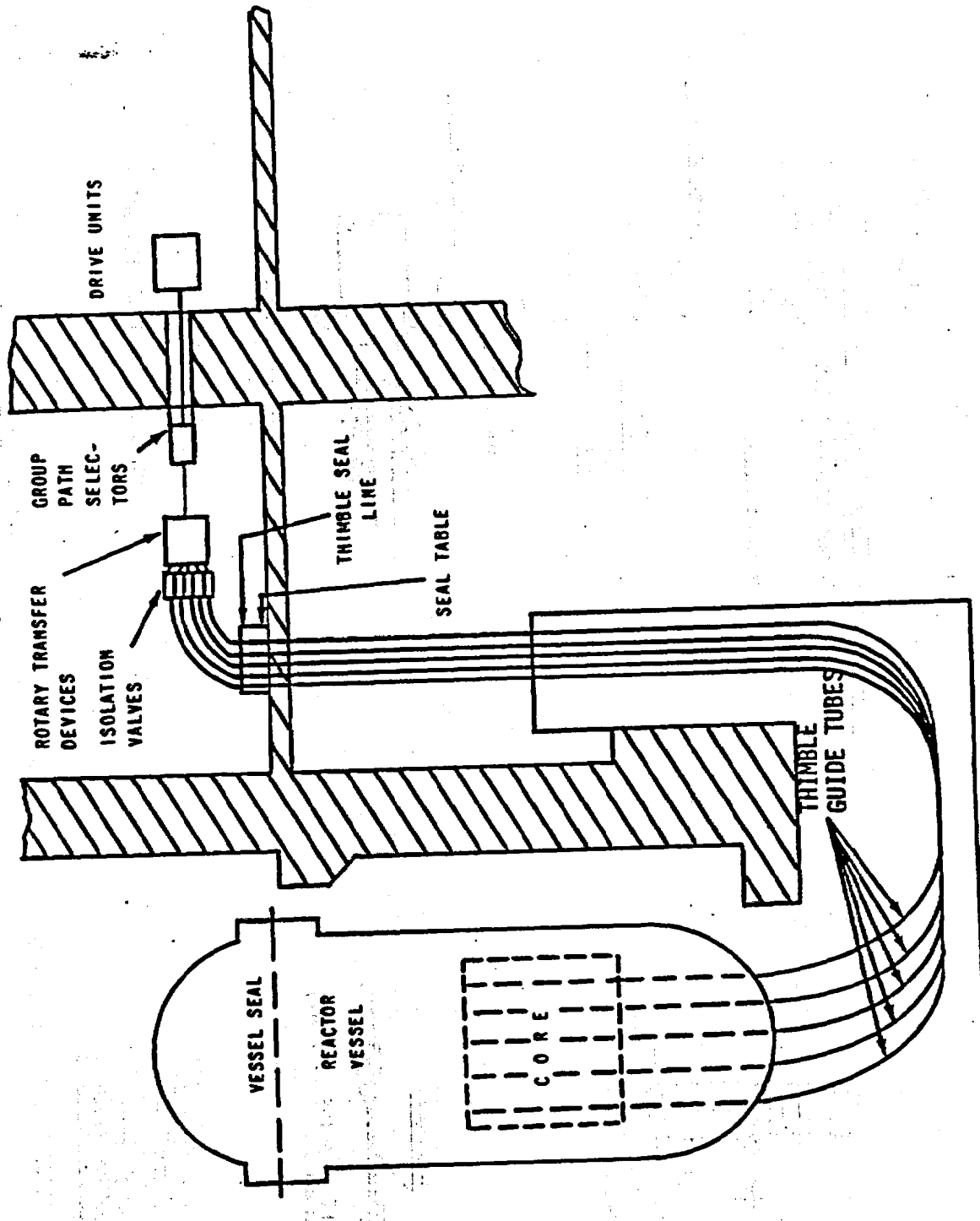
Change Description:
UCR-2054, Rev. 0

Unit 2

Title: Environmental Conditions Inside Containment - Main Steam Break (Unit 2)



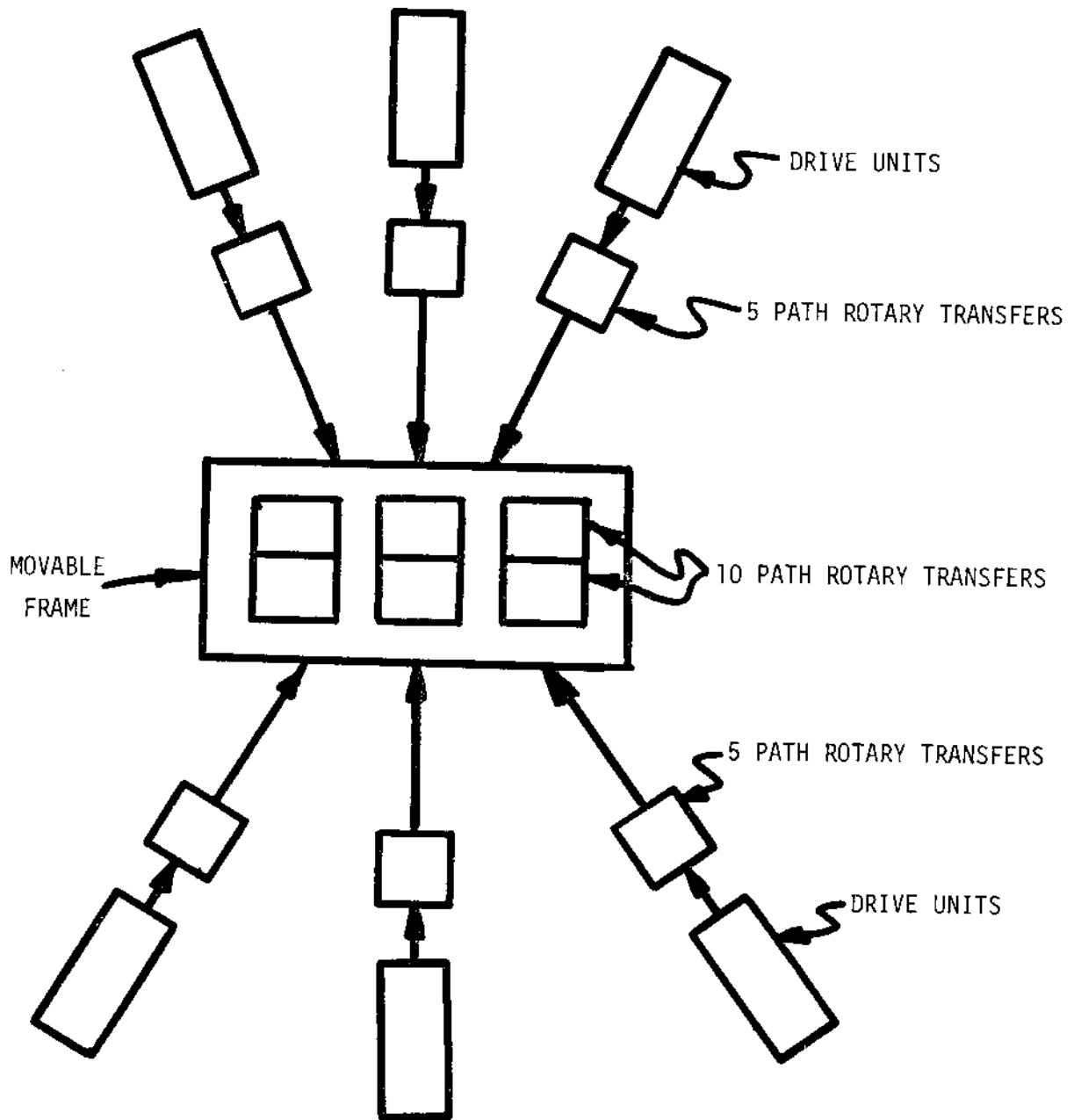
IN-CORE INSTRUMENTATION DETAILS
 FIGURE 7.6-1
 July, 1986



July, 1982

Typical Arrangement of Movable Miniature Neutron Flux Detector System
(Elevation View) **FIGURE 7.6-2**

UFSAR Revision 30.0



SCHEMATIC ARRANGEMENT OF IN-CORE
FLUX DETECTORS (PLAN VIEW)

FIGURE 7.6-3

July, 1982

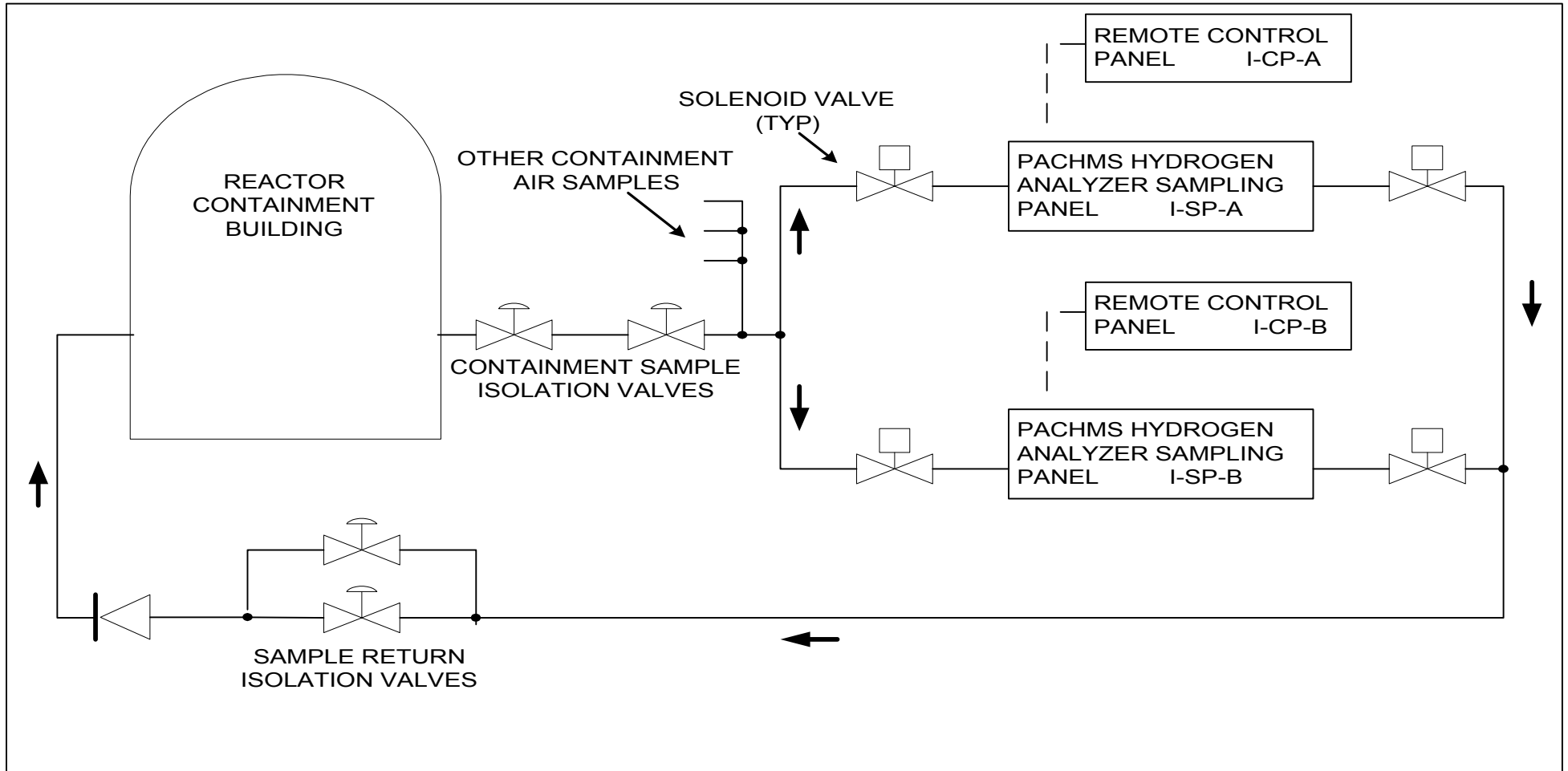
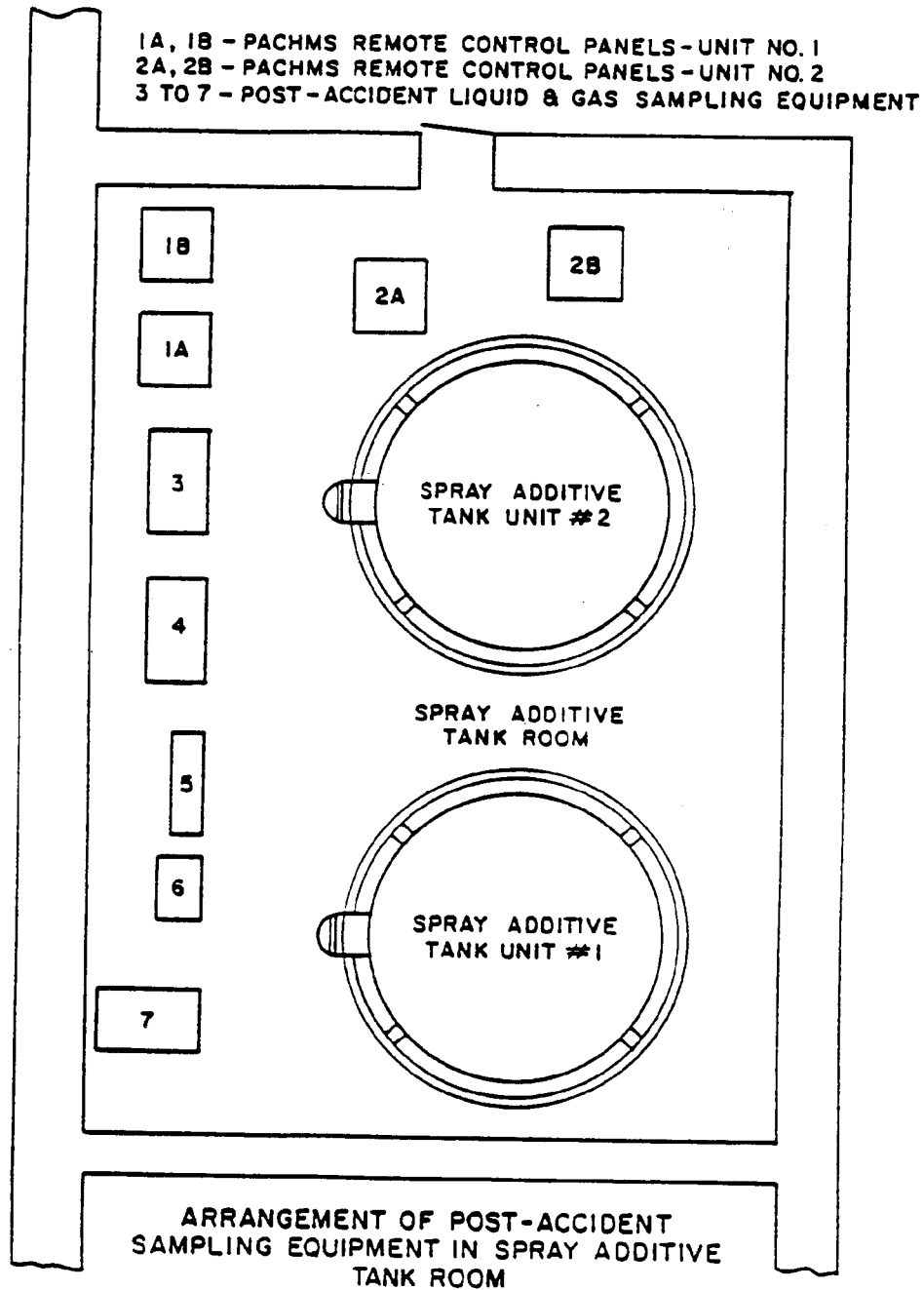


FIGURE 7.8-1

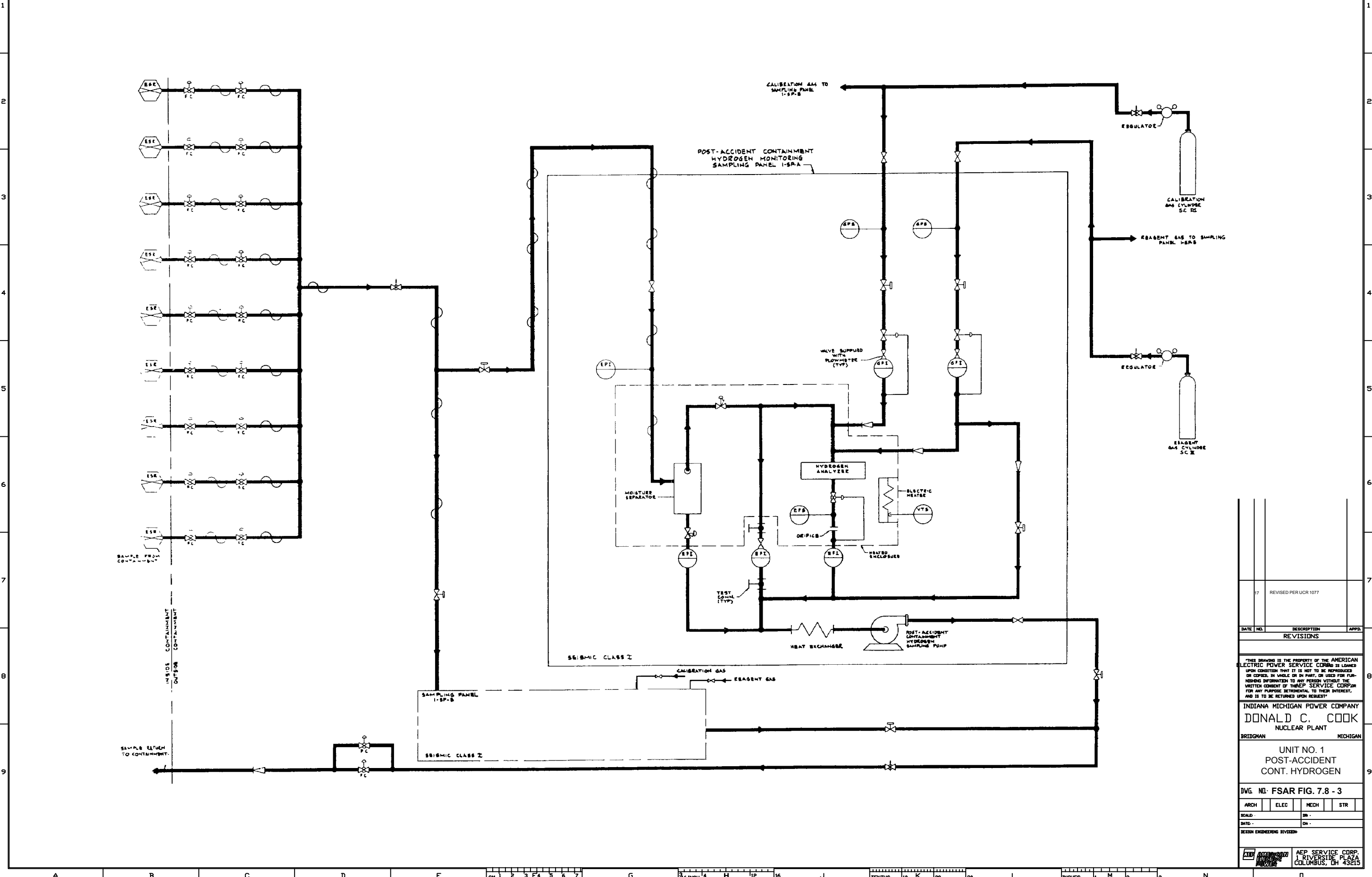
SCHEMATIC DIAGRAM OF POST-ACCIDENT CONTAINMENT HYDROGEN MONITORING SYSTEM (PACHMS)

Revision 17	Change Description: UCR-1077	
AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN	Title: SCHEMATIC DIAGRAM OF POST-ACCIDENT CONTAINMENT HYDROGEN MONITORING SYSTEM (PACHMS)	
	DWG. NO. UFSAR FIG 7.8-1	Sheet 1 of 1

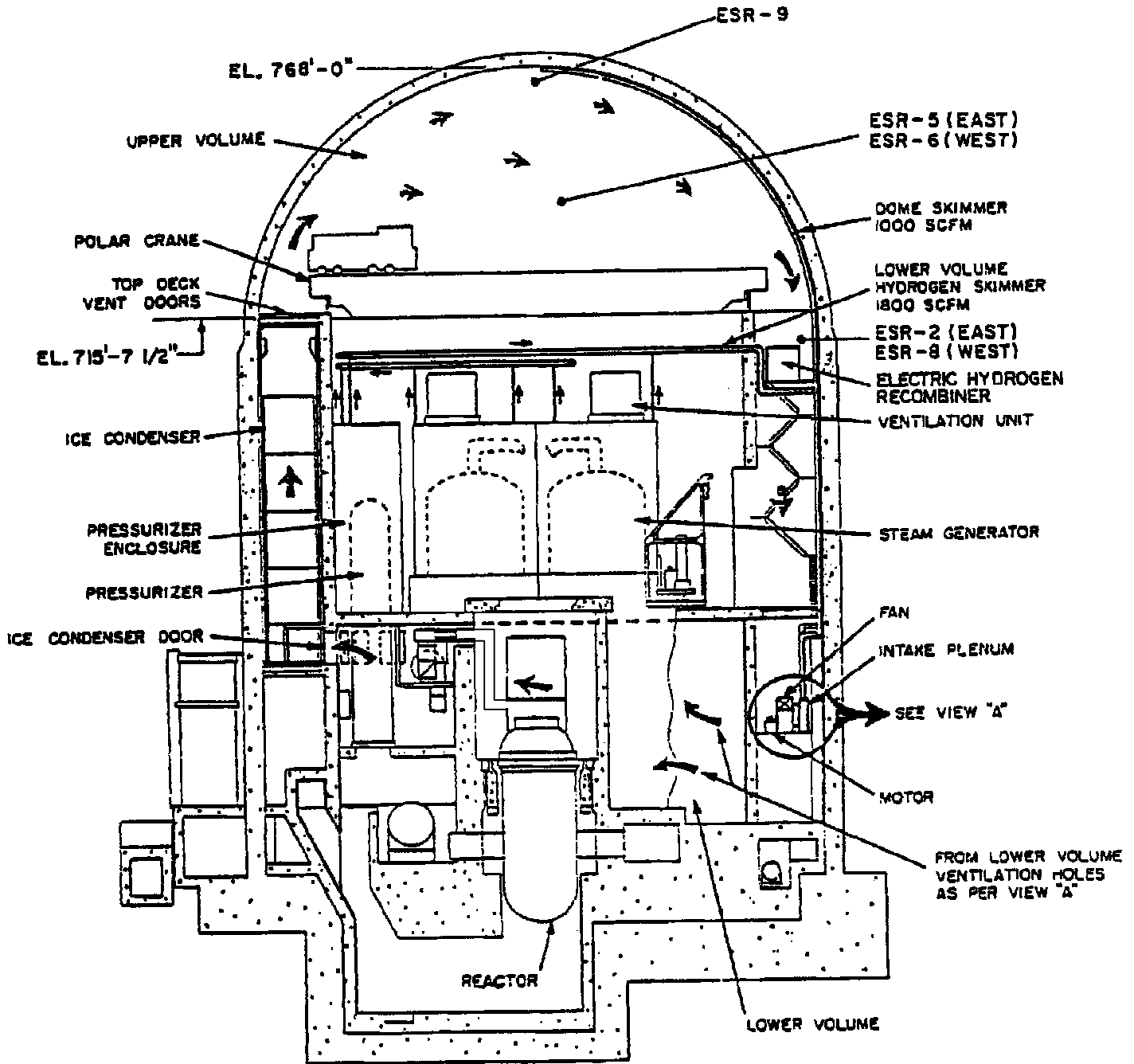
UFSAR Revision 30.0



17	REVISED PER UCR 1077	UNIT 1
REV. NO.	DESCRIPTION	
REVISIONS		
AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN	TITLE ARRANGEMENT OF POST-ACCIDENT SAMPLING EQUIPMENT IN SPRAY ADDITIVE TANK ROOM	
	DWG. NO. FSAR FIG. 7.8-2	SH 1 of 1

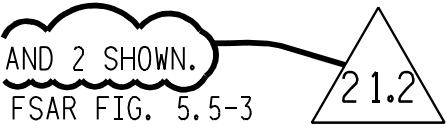


17	REVISED PER UCR 1077		
DATE	NO.	DESCRIPTION	APPR.
REVISIONS			
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INDIANA MICHIGAN POWER COMPANY DONALD C. COOK NUCLEAR PLANT BRIDGMAN MICHIGAN			
UNIT NO. 1 POST-ACCIDENT CONT. HYDROGEN			
DWG. NO. FSAR FIG. 7.8 - 3 ARCH. ELEC. MECH. STR. SCALE: 3/8" = 1" DATE: 04-			
DESIGN ENGINEERING DIVISION <small>AEP SERVICE CORP. 1 RIVERSIDE PLAZA COLUMBUS, OH 43260</small>			



NOTES:

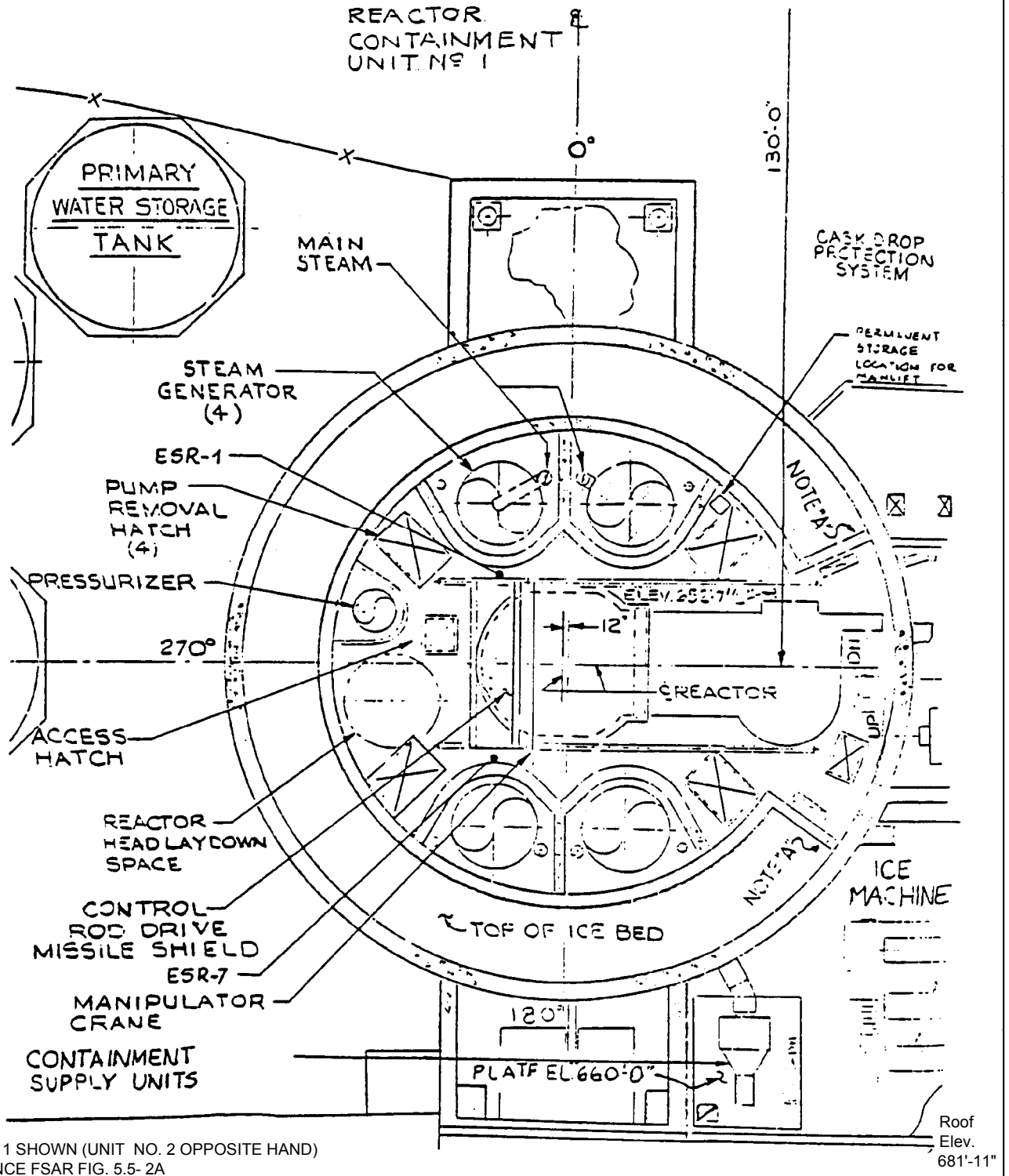
1. UNIT NO.1 AND 2 SHOWN.
2. REFERENCE FSAR FIG. 5.5-3



21.2	REVISED PER UCR-1837, REV.0.	UNIT 1
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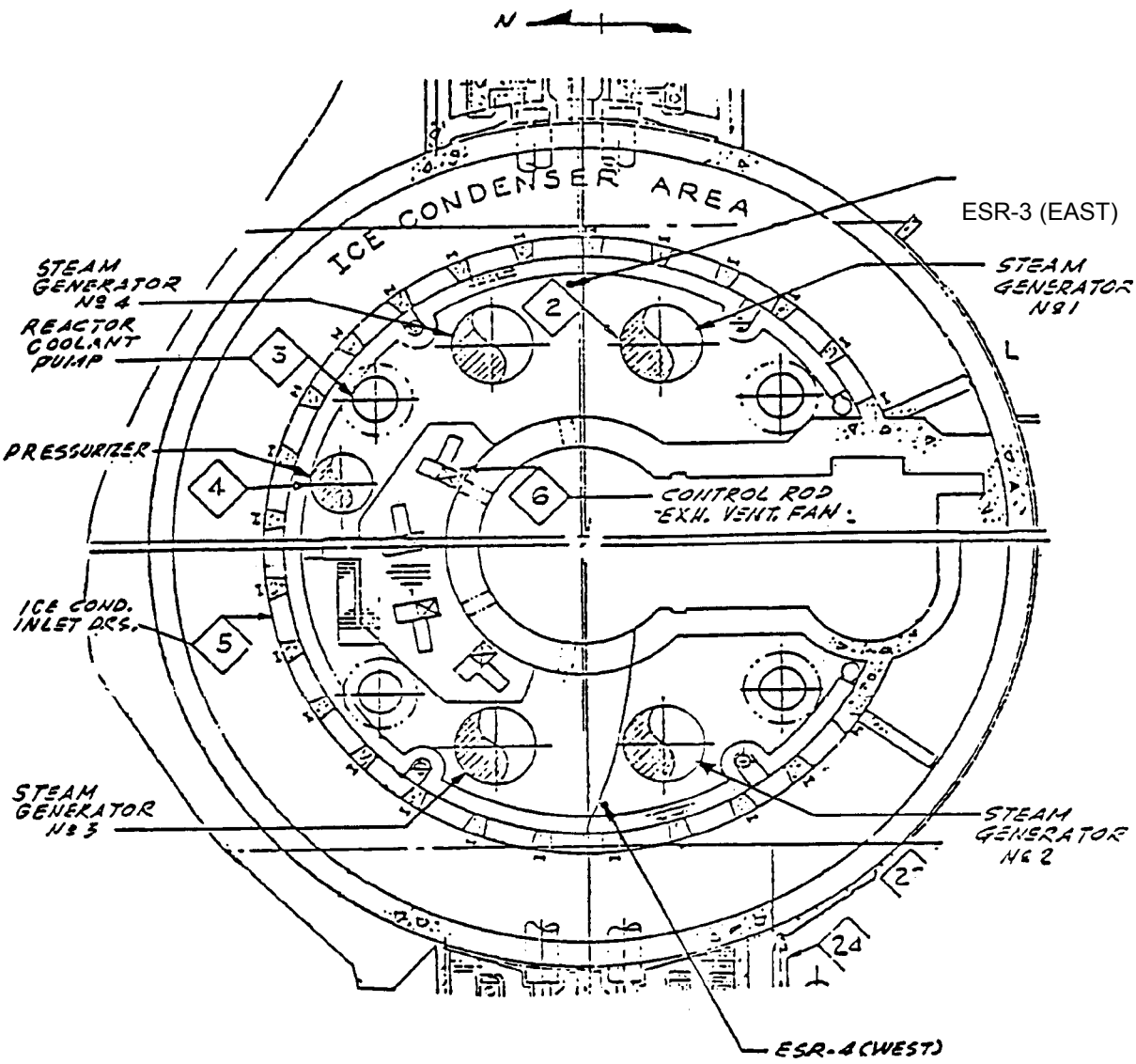
REV. NO.	DESCRIPTION	
REVISIONS		

AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN	TITLE SECTION OF CONTAINMENT DETAILING LOCATION OF HYDROGEN MONITORING SAMPLE PORTS ESR-2, 5, 6, 8, AND 9 DWG. NO. FSAR FIG. 7.8-4	SH 1 OF 1
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- NOTES:
 1. UNIT NO. 1 SHOWN (UNIT NO. 2 OPPOSITE HAND)
 2. REFERENCE FSAR FIG. 5.5- 2A

17	REVISED PER UCR 1077	UNIT 1
REV. NO.	DESCRIPTION	
REVISIONS		
AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN	TITLE Plan Elevation of Containment (Elevation - 650'-0") Detailing Location of Hydrogen Monitoring Sample Ports ESR- 1 & 7	
	DWG. NO. FSAR FIG. 7.8-5	SH 1 of 1



NOTES; 1. UNIT NO. 1 SHOWN (UNIT NO. 2 OPPOSITE HAND)
 2. REFERENCE DWG. NO. 12-5169

17	REVISED PER UCR 1077	UNIT 1
REV. NO.	DESCRIPTION	
REVISIONS		
AMERICAN ELECTRIC POWER COOK NUCLEAR PLANT NUCLEAR GENERATION GROUP BRIDGMAN, MICHIGAN	TITLE Plan Elevation of Containment (Elevation 633'-0") Detailing Location of Hydrogen Monitoring Sample Ports ESR-3 & 4	
	DWG. NO. FSAR FIG. 7.8-6	SH 1 of 1