- 1. Letter from D A Bixel (CP Co) to A Schwencer (NRC), dated February 8, 1977, spent fuel pool modifications, response to Question S6, (2510/0390).
- 2. VandeWalle, David J, Director, Nuclear Licensing, CP Co, to Director, Nuclear Reactor Regulation, USNRC, "Proposed Technical Specification Change Request Auxiliary Feedwater System," September 17, 1984.
- 3. Johnson, B D, CP Co, to Director, Nuclear Reactor Regulation, Attention Mr Dennis M Crutchfield, "Seismic Qualification of Auxiliary Feedwater System," August 19, 1981.
- 4. Mr Thomas V Wambach (NRC) to K W Berry (CP Co), letter dated July 24, 1987.
- 5. Facility change, FC-680, Spent Fuel Pool Rerack, (Cartridges 9576 through 9579).
- a. Safety Analysis Report, "Spent Fuel Storage Modification," October 16, 1986, amended December 19, 1986.
- b. Westinghouse, "Design Report of Region 2 Spent Fuel Storage Racks Palisades Plant," WNEP-8626, May 1, 1987, (9579/1589).
- 6. Letter from K W Berry (CP Co) to NRC, dated January 29, 1990, "Response to Generic Letter 89-13, Service Water System Problems Affecting Safety-Related Equipment."
- 7. Deviation Report Number D-PAL-89-061, "Post-Accident Operation of CCW System," initiated March 23, 1989.
- 8. Safety Evaluation By the Office of Nuclear Reactor Regulation, Withdrawal of Service Water Temperature Limit, Consumers Power Company, May 4, 1987.
- 9. Deleted.
- 10. Engineering Analysis, EA-A-PAL-96-011, "Calculation of Spent Fuel Pool Cooling System Design Heat Load," Rev. 2, April, 2000.
- 11. Deleted
- 12. Engineering Analysis, EA-DBD-1.02-003, Rev 1, "West ESG Room Flooding."
- 13. CPCo Internal Correspondence, DTPerry to GBSzczypka, "Heat Exchanger (CCW) Flow Limits," dated November 2, 1988.

- 14. El Final Report, "Analysis of Palisades Component Cooling and Service Water Systems," Revision 1, dated October 25, 1988 (D045\1075).
- 15. EA-GCP-93-01, "Review of Current Spent Fuel Pool Criticality Analysis and Bounding Conditions," March 19, 1993, Cart/Frame F474/0631.
- 16. Facility Change, FC-864, "Dry Storage of Spent Fuel Nuclear Fuel."
- 17. EA-FC-864-026, "Track Alley Load Distribution System (LDS): Evaluation of Load, Load Combination and Acceptance Criteria," Revision 1.
- 18. EA-FC-864-30, "FC-864 Heavy Loads," Revision 3.
- 19. Certificate of Compliance (C of C) for Dry Fuel Storage Casks, No 1007 Revision 0, effective 5/7/93.
- 20. Deleted
- 21. Safety Analysis Report for the Ventilated Storage Cask System, prepared by Pacific Sierra Nuclear Associates and Sierra Nuclear Corporation, October 1991.
- 22. EA-GOTHIC-CST-01, Rev 2, September 2009, "Determination of Initial Condensate Storage Tank (T-2) Indicated Level to Ensure 100,000 Gallons of Available Inventory."
- 23. EA-Bioshield Temp-01, "Effects of Plant Operating Temperatures on the Palisades Biological Shield Wall," Rev 1, October, 1995.
- 24. Engineering Change EC 8368, "Replace Existing Control Room HVAC Dampers with Bubble-Tight Dampers," dated June 2007.
- 25. EA-A-PAL-93-043-01, "NPSH Evaluation for Charging and Boric Acid Pumps," Rev 2, November 2003.
- 26. Safety Evaluation 94-0288, "Plant Operation With CV-0913 and CV-0950 in the Open Position."
- 27. EA-C-PAL-95-1299A, "Biological Shield Wall Temperature Profile Based On Measured Temperatures Inside the Reactor Vessel Cavity."
- 28. Condition Report C-PAL-97-1363, "Component Cooling Water System Not Analyzed for Present LOCA Containment Analysis."
- 29. EA-LOCA-2001-01, Revision 1, "Containment Response to a LOCA Using CONTEMPT-LT/28."

- 30. CPCo Letter to NRC, "Steam Binding of AFW Pumps Response to Generic Letter 88-3," dated May 9, 1988.
- 31. NRC Correspondence from Robert G. Schaaf to Thomas C. Bordine, Docket No. 50-255, October 28, 1997.
- 32. Technical Specification Surveillance Procedure RO-216, "Service Water Flow Verification."
- 33. Internal Memo GFPratt to BNYoung, "A-CMT-96-044 Ultimate Heat Sink Technical Specification Change Request," dated January 9, 1997.
- 34. CCP Discrepancy Report F-CG-89-058 dated October 31, 1990, "Failure of Nonseismic CCW Piping Inside Containment," (D309/0011).
- 35. Engineering Analysis EA-GWO7793-01, "CCW Piping Inside Containment HELBA," (C268/1281).
- 36. EA-JEG-97-01, Revision 0, "Operation of Charging Pumps P-55B & C Without CCW Flow to the Oil Coolers."
- 37. Letter from NRC to KMHaas, SER for "Change in Commitments to Perform Post-Accident Containment Hydrogen Concentration Analysis and Sump pH Analysis," dated March 27, 1995.
- 38. Letter from R L Hauter (Consumers) to J F O'Leary (NRC), "Docket No 50-255," dated December 1, 1972 (2504/0226)
- 39. Letter from R A Vincent (Consumers) to D M Crutchfield (NRC), "Docket No 50-255 License DPR-20 Palisades Plant Systematic Evaluation Program," dated March 31, 1982
- 40. SC-94-065, "Change Shield Cooling System Flow Rate to 134-154 gpm"
- 41. EAR-2000-0129, "Replacement of Shield Cooling Pumps"
- 42. EA-D-PAL-93-272F-01, Revision 1, "Engineered Safeguards Room Heatup Following LOCA in Conjunction with a LOOP" (44782402)
- 43. Design Basis Document, DBD-7.10, "NFPA 805 Fire Protection Program"
- 44. NRC Safety Evaluation and Amendment 202 to the Palisades License, June 4, 2001
- 45. EA-SFP-99-03, Revision 0, "New Fuel Storage Fuel Pool and Fuel Handling Criticality Safety Analysis" (4996/1664)

- 46. Facility Change FC-976, "Modify L-3 to Single Failure Proof and Increase Capacity from 100 Tons to 110 Tons"
- 47. EA-D-PAL-88-222B, Revision 0, "Minimum Lake Level Calculation for P-5 Suction With Five Discharge Tubes Open" (F043/1540)
- 48. Engineering Analysis EA-MOD-2003-019-05, "Evaluation of Heavy Loads for NUHOMS System," Revision 0
- 49. Standardized NUHOMS® Horizontal Modular Storage System Certificate of Compliance (C of C) Number 1004, Amendment No. 5
- 50. Stang, John F., Senior Project Manager, Project Directorate III-1, USNRC, to Daniel J. Malone, Palisades Nuclear Plant Site President, "Amendment No. 215 to the Palisades Operating Licensee No DPR-20: Palisades Spent Fuel Pool Crane Upgrade (TAC No. MC1862)," June 16, 2004
- 51. USNRC Generic Letter 96-06, Assurance of Equipment Operability and Containment Integrity During Design Basis Accident Conditions," September 30, 1996
- 52. EA-GL-96-06-SWS-02, "Service Water GL-96-06 Waterhammer Assessment," August 12, 2004
- 53. Generic Letter 96-06, Waterhammer Issues Resolution, Technical Basis Report, EPRI, Final Report April 2002
- 54. Generic Letter 96-06, Waterhammer Issues Resolution, User's Manual, EPRI, Final Report, April 2002
- 55. NRC Acceptance Letter of EPRI TR-113594, dated April 3, 2002, and enclosed Safety Evaluation Report, Evaluation of Electric Power Research Institute Report, TR-113594, "Resolution of Generic Letter 96-06 Waterhammer Issues," dated December 2000
- 56. 10 CFR 50.44, Combustible Gas Control for Nuclear Power Reactors, effective October 16, 2003
- 57. Engineering Analysis EA-EC-235-01, Revision 1, "Assessment of the High Pressure Air System's Capacity to Cycle Valves in the West Engineering Safeguards," August 2020
- 58. Bechtel Specification 5395-M-73, "Specification for the Reactor Crane," Revision 1, dated March 8, 1967
- 59. Letter from Dresser Crane, Hoist & Tower Division to Bechtel Corporation dated September 9, 1970 (Cart./Frame G728/1146)

- 60. Bechtel Inter-office Memo dated December 18, 1969 (Cart/Frame 0447/2123)
- 61. Bechtel Field Procedure Number 3, Revision 6, Change #7, Load Test of Up-Rated Polar Crane, 2/22/1970 (Cart./Frame 0183/0181)
- 62. NRC letter from Dennis M. Crutchfield to David J. VandeWalle, "Control of Heavy Loads (Phase 1) NUREG-0612," dated November 9, 1983
- 63. Consumers Power Letter from Gerald B. Slade to the Nuclear Regulatory Commission, "Completion of Actions Made in Response to NRC Bulletin 88-04 Potential Safety Related Pump Loss," dated October 30, 1990.
- 64. Engineering Change EC516, "CCW System Temperature Re-Rate Design Change," dated January 25, 2007.
- 65. EA-CA025644-01, Revision 1, Evaluation of the Impact of 110% Emergency Diesel Generator Overload Operating Condition on Ambient Temperature, August 2007 (later).
- 66. EA-EC9600-01, Revision 1, Functionality of Electrical Equipment in the Emergency Diesel Room at an Elevated Temperature of 121°F, April 2007 (later).
- 67. EA-WJB-00-01, Revision 2, Spent Fuel Pool Dilution Analysis, EC0044749.
- 68. Engineering Analysis EA-CAP047706-01, "Palisades Reactor Head Drop Analysis," dated July 3, 2007.
- 69. AREVA NP Inc., ANP-2858P-003, Palisades SFP Region 1 Criticality Evaluation with Burnup Credit, January 2011.
- 70. EA-EEQ-TEMP-01, Revision 2, "EEQ Average Arrhenius Temperature Outside Containment Building Based on Temperature Monitoring"
- 71. EA-BHS-EQ-2000-01, Revision 4, "Results of Temperature Monitoring in Containment and Suggested Methods for Use of the Temperature Data in EQ Qualified Life Calculations"
- 72. EA-BDM-88-02, Revision 0, "Impact on EEQ Files Due to Increase of Containment Operating Temperature from 138°F to 145°F"
- 73. EA-EC46993-01, Revision 0, "E-53A/B Spent Fuel Pool Heat Exchanger Tube Plugging Calculation"
- 74. EA-CCW-87-01, Revision 3, "Spent Fuel Pool Heat Load and Required SFPHX Component Cooling Water Flow During a DBA"
- 75. EA-EC32159-05, Revision 0, "Criticality Safety Evaluation for Region 1 Spent Fuel Racks at Palisades (HI-2115017)"

- 76. PLP-RPT-12-00053, "NFPA Code Conformance Review"
- 77. EA-EC46465-09, Revision 0, "Evaluation of ADV Air Requirements for FLEX Event."
- 78. EA-EC32159-01, Revision 0, "Bulk Thermal-Hydraulic Analyses for the Palisades SFP and Tilt Pit with New Region 1 Racks (HI-2114999)"
- 79. Final Safety Analysis Report for the HI-STORM FW MPC Storage System, Holtec Report HI-2114830, Revision 4
- 80. Certificate of Compliance No. 1032, Amendment 1, Revision 1, for HI-STORM FW MPC Storage System
- 81. Engineering Change EC 63832, "Add an Air Accumulator to CV-0910 and Appendix J Program Update."
- 82. Certificate of Compliance for Dry Fuel Storage Casks, Certificate No. 1007 Revision 0 Renewal, effective September 20, 2017.
- 83. Final Safety Analysis Report for the VSC-24 Ventilated Storage Cask System, prepared by Energy Solutions, Revision 9, October 2017.
- NRC letter, "Palisades Nuclear Plant Issuance of Amendment Regarding Transition to Risk-Informed, Performance-Based Fire Protection Program in Accordance with 10 CFR 50.48(c)," dated February 27, 2015 (ADAMS Accession No. ML15007A191).