UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 26, 2021

Mr. Don Moul
Vice President, Nuclear Division
and Chief Nuclear Officer
Florida Power & Light Company
Mail Stop: NT3/JW
15430 Endeavor Drive
Jupiter, FL 33478

SUBJECT: TURKEY POINT NUCLEAR GENERATING, UNIT NOS. 3 AND 4 – CORRECTION TO AMENDMENT CONCERNING EXTENSION OF CONTAINMENT LEAK RATE TESTING FREQUENCY AMENDMENTS (EPID L- 2020-LLA-0016)

Dear Mr. Moul:

By letter dated February 26, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21032A020), the U.S. Nuclear Regulatory Commission (NRC) issued amendments to Subsequent Renewed Facility Operating License Nos. DPR-31 and DPR-41 for Turkey Point Nuclear Generating, Unit Nos. 3 and 4 (Turkey Point), respectively. The amendments modify Section 6.8.4.h, "Containment Leakage Rate Testing Program," of the Turkey Point technical specifications (TS) to support the extension of the frequency of the Type A Integrated Leak Rate Test from 10 to 15 years and allow the extension of the containment isolation valves leakage test intervals (i.e., Type C tests) from their current 60-month frequency to 75 months.

The NRC staff has determined that the amendment number for the Unit 4 amendment should be 288. In addition, page 3 for both Unit Nos. 3 and 4 was not enclosed with the February 26, 2021 amendments. The appropriate page 3 for both Unit Nos. 3 and 4, the revised TS pages 6-9, and the Unit 4 amendment page enclosed with this letter are the correct pages to be used for the amendments issued on February 26, 2021.

We apologize for this oversight. If you have any questions, please contact me at 301-415-2315 or by e-mail to Eva.Brown@nrc.gov.

Sincerely,

Eva A. Brown

Digitally signed by Eva A. Brown
Date: 2021.05.26 13:11:32 -04'00'

Eva A. Brown, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosures:
1. Corrected Unit 4 Amendment pages
2. Corrected Safety Evaluation Replacement pages
3. Corrected page 3 to DPR-31
4. Corrected page 3 to DPR-41
5. Corrected TS page 6-9

cc: Listserv

ENCLOSURE 1

CORRECTED UNIT 4 AMENDMENT PAGES

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT NO. 4

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 288
Subsequent Renewed License No. DPR-41

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company (the licensee) dated January 27, 2020, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Subsequent Renewed Facility Operating License No. DPR-41 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 288 are hereby incorporated into this subsequent renewed operating license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into this subsequent renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Undine S. Shoop, Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Subsequent Renewed
Facility Operating License and
Technical Specifications

Date of Issuance: February 26, 2021

ENCLOSURE 2

UNIT 4 CORRECTED SAFETY EVALUATION REPLACEMENT PAGES

NOTICES AND ENVIRONMENTAL FINDINGS

RELATED TO AMENDMENT NO. 295 TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-31 AND AMENDMENT NO. 288 TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER & LIGHT COMPANY

TURKEY POINT NUCLEAR GENERATING UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

By application dated January 27, 2020 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML20034D803), Florida Power & Light Company (FPL, or the licensee) requested changes to the Technical Specifications (TSs) for Turkey Point Nuclear Generating Unit Nos. 3 and 4 (Turkey Point), which are contained in Appendix A of Subsequent Renewed Facility Operating License Nos. DPR-31 and DPR-41, respectively. The licensee proposed to modify Section 6.8.4.h, "Containment Leakage Rate Testing Program," of the Turkey Point TSs to support the extension of the frequency of the Type A Integrated Leak Rate Test from 10 to 15 years and allow the extension of the containment isolation valves leakage test intervals (i.e., Type C tests) from their current 60-month frequency to 75 months.

2.0 STATE CONSULTATION

In accordance with the Commission's regulations, the U.S. Nuclear Regulatory Commission (NRC) staff notified the State of Florida official (Ms. Cynthia Becker, M.P.H., Chief of the Bureau of Radiation Control, Florida Department of Health) on January 25, 2021, of the proposed issuance of the amendments. The State official had no comments.

3.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to the use of facility components located within the restricted area as defined in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 20, "Standards for Protection Against Radiation." The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding, which was published in the *Federal Register* on March 10, 2020 (85 FR 13951) that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 26, 2021

TURKEY POINT NUCLEAR GENERATING UNIT NOS. 3 AND 4 - SAFETY EVALUATION CONCERNING EXTENSION OF CONTAINMENT LEAK RATE TESTING FREQUENCY (EPID L-2020-LLA-0016) (L-2020-003)

LICENSEE INFORMATION

Licensee: Florida Power and Light Company (FPL, the licensee)

Plant Name and Units: Turkey Point Nuclear Generating Unit Nos. 3 and 4

Subsequent Renewed Facility Operating Licenses: DPR-31 and DPR-41

Docket Nos.: 50-250 and 50-251

Amendment Numbers: 295 and 288

APPLICATION INFORMATION

Submittal Date: January 27, 2020

Submittal Agencywide Documents Access and Management System (ADAMS) Accession No.: ML20034D803

1.0 PROPOSED CHANGE

By application dated January 27, 2020, the licensee requested changes to the Technical Specifications (TSs) for Turkey Point Nuclear Generating Unit Nos. 3 and 4 (Turkey Point), which are contained in Appendix A of Subsequent Renewed Facility Operating License Nos. DPR-31 and DPR-41, respectively. The licensee proposed to modify Turkey Point, TS 6.8.4.h, "Containment Leakage Rate Testing Program," to support the extension of the frequency of the Type A Integrated Leak Rate Test (ILRT) from 10 years to 15 years and to allow the extension of the Containment Isolation Valves (CIVs) leakage test intervals (i.e., Type C tests) from their current 60-month frequency to 75 months. This testing supports the containment leak rate program described in Section 50.54(o) of Title 10 of the *Code of Federal Regulations* (10 CFR).

For the proposed change, the maximum surveillance interval for the Type A ILRTs will change to no longer than 15 years from the last ILRT based on satisfactory performance history. The proposed change would require the performance of the next Unit 3 ILRT by no later than July 2027, and the next Unit 4 ILRT by no later than February 2028.

ENCLOSURE 3

CORRECTED PAGE 3 TO DPR-31

applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified below:

A. Maximum Power Level

The applicant is authorized to operate the facility at reactor core power levels not in excess of 2644 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 295, are hereby incorporated into this subsequent renewed license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into this subsequent renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

C. Final Safety Analysis Report

The licensee's Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on November 1, 2001, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than July 19, 2012.

The Final Safety Analysis Report supplement as revised on November 1, 2001, described above, shall be included in the next scheduled update to the Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

D. Fire Protection

FPL shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the licensee amendment requests dated June 28, 2012 and October 17, 2018 (and supplements dated September 19, 2012; March 18, April 16, and May 15, 2013; January 7, April 4, June 6, July 18, September 12, November 5, and December 2, 2014; and February 18, 2015; October 24, and December 3, 2018; and January 31, 2019), and as approved in the safety evaluations dated May 28, 2015 and March 27, 2019. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would require prior NRC approval, the licensee may make changes to the fire protection program without prior approval of the Commission if those changes satisfy the provisions set forth in 10 CFR 50.48(a) and 10 CFR 50.48(c), the change does not require a change to a technical specification or a license condition, and the criteria listed below are satisfied.

Risk-Informed Changes that May Be Made Without Prior NRC Approval

A risk assessment of the change must demonstrate that the acceptance criteria below are met. The risk assessment approach, methods, and data shall be acceptable to the NRC and shall be appropriate for the nature and scope of the

ENCLOSURE 4

CORRECTED PAGE 3 TO DPR-41

A. Maximum Power Level

The applicant is authorized to operate the facility at reactor core power levels not in excess of 2644 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 288 are hereby incorporated into this subsequent renewed operating license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into this subsequent renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

C. Final Safety Analysis Report

The licensee's Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on November 1, 2001, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than April 10, 2013.

The Final Safety Analysis Report supplement as revised on November 1, 2001, described above, shall be included in the next scheduled update to the Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

D. Fire Protection

FPL shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the licensee amendment requests dated June 28, 2012 and October 17, 2018 (and supplements dated September 19, 2012; March 18, April 16, and May 15, 2013; January 7, April 4, June 6, July 18, September 12, November 5, and December 2, 2014; and February 18, 2015; October 24, and December 3, 2018; and January 31, 2019), and as approved in the safety evaluations dated May 28, 2015 and March 27, 2019. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would require prior NRC approval, the licensee may make changes to the fire protection program without prior approval of the Commission if those changes satisfy the provisions set forth in 10 CFR 50.48(a) and 10 CFR 50.48(c), the change does not require a change to a technical specification or a license condition, and the criteria listed below are satisfied.

Risk-Informed Changes that May Be Made Without Prior NRC Approval

A risk assessment of the change must demonstrate that the acceptance criteria below are met. The risk assessment approach, methods, and data shall be acceptable to the NRC and shall be appropriate for the nature and scope of the change being evaluated; be based on the as-built, as-operated, and maintained plant; and reflect the operating experience at the plant. Acceptable methods to assess the risk of the change may include methods that have been used in the

ENCLOSURE 5

CORRECTED TECHNICAL SPECIFICATION PAGE 6-9

ATTACHMENT TO LICENSE AMENDMENT NOS. 295 AND 288

TURKEY POINT NUCLEAR GENERATING UNIT NOS. 3 AND 4

SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NOS. DPR-31 AND DPR-41

DOCKET NOS. 50-250 AND 50-251

Replace page 3 of Subsequent Renewed Facility Operating License No. DPR-31 with the attached page 3. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Replace page 3 of Subsequent Renewed Facility Operating License No. DPR-41 with the attached page 3. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove	**Insert**
6-9	6-9

PROCEDURES AND PROGRAMS (Continued)

9. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives greater than 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;

10. Limitations on the annual dose or dose commitment to any member of the public, beyond the site boundary, due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.

The provisions of Specifications 4.0.2 and 4.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency.

g. DELETED

h. Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, and as modified by approved exemptions. This program shall be in accordance with Nuclear Energy Institute (NEI) 94-01, Revision 3-A, "Industry Guidance for Implementing Performance Based Option of 10 CFR 50 Appendix J," and the conditions and limitations specified in NEI 94-01, Revision 2-A, with the following deviations or exemptions:

1) A vacuum test will be performed in lieu of a pressure test for airlock door seals at the required intervals (Amendment Nos. 73 and 77, issued by NRC November 11, 1981).

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a, is defined here as the containment design pressure of 55 psig.

The maximum allowable containment leakage rate, L_a, at P_a, shall be 0.20% of containment air weight per day.

Leakage Rate acceptance criteria are:

1) The As-found containment leakage rate acceptance criterion is $\leq$ 1.0 L_a. Prior to increasing primary coolant temperature above 200°F following testing in accordance with this program or restoration from exceeding 1.0 L_a, the As-left leakage rate acceptance criterion is $\leq$0.75 L_a, for Type A test.

2) The combined leakage rate for all penetrations subject to Type B or Type C testing is as follows:

SUBJECT: TURKEY POINT NUCLEAR GENERATING UNIT NOS. 3 AND 4 – CORRECTION TO AMENDMENT CONCERNING EXTENSION OF CONTAINMENT LEAK RATE TESTING FREQUENCY AMENDMENTS (EPID L-2020-LLA-0016) DATED MAY 26, 2021

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